Docket Nos.: 50-413
50-414

March 21, 1988

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No. 42 to Facility Operating License NPF-35 and Amendment No. 35 to Facility Operating License NPF-52-Catawba Nuclear Station, Units 1 and 2 (TACS 67115/67116)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 42 to Facility Operating License NPF-35 and Amendment No. 35 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. These amendments consists of changes to the Technical Specifications in response to your application dated February 10, 1988.

The amendments modify the Technical Specifications to reduce the required Ractor Coolant System total flow from 396,100 gpm to 387,600 gpm. The amendments are effective as of their date of issuance.

A copy of the related safety evaluation supporting Amendment No. 42 to Facility Operating License NPF-35 and Amendment No. 35 to Facility Operating License NPF-52 is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. 42 to NPF-35
2. Amendment No. 35 to NPF-52
3. Safety Evaluation

cc w/enclosure:
See next page

PD II-3	PD II-3	PD II-3
MRood	KJabbour:sw	Acting PD
3/7/88	3/7/88	3/7/88

8803280356 880321
PDR ADOCK 05000413
P PDR

DATED: March 21, 1988

AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NPF-52 - Catawba Nuclear Station, Unit 2
AMENDMENT NO. 42 TO FACILITY OPERATING LICENSE NPF-35 - Catawba Nuclear Station, Unit 1

DISTRIBUTION:
Docket File 50-414/413
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L. Harmon
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T. Barnhart (8)
ACRS (10)
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S. Varga/G. Lainas
E. Butcher
W. Hodges

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by Duke Power Company acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc., (licensees) dated February 10, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

8803280359 880321
PDR ADOCK 05000413
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 42, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into the license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:
Lawrence P. Crocker, Acting Director
Project Directorate II-3
Division of Reactor Projects I/II

Attachment:
Technical Specification Changes

Date of Issuance: March 21, 1988

PDII-3/DRPI/II	PDII-3/DRPI/II	SRXB	OGC-WF
MRood	KJabbour	WHodges	MYoung
03/7/88	03/7/88	03/7/88	03/8/88
PDII-DRPI/II	ADR2:NRR		
Acting PD	GLainas		
03/07/88	03/10/88		

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensee) dated February 10, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Pargraph 2.C. (2) of Facility Operating License No. NPF-52 is hereby changed to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 35 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into the license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:
Lawrence P. Crocker , Acting Director
Project Directorate II-3
Division of Reactor Projects I/II

Attachment:
Technical Specification Changes

Date of Issuance: March 21, 1988

PDII-3/DRPI/II	PDII-3/DRPI/II	SRXB	OGC-WF
MRood	KJabbour	WHodges	
03/ 7/88	03/ 7/88	03/ 7/88	03/ 8/88
PDII-DRPI/II	ADR2:NRR		
Acting PD	GLainas		
03/ 7/88	03/ 14/88		

ATTACHMENT TO LICENSE AMENDMENT NO. 42

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 35

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Amended Page	Overleaf Page
2-4	2-3
3/4 2-9	
3/4 2-10	
3/4 2-11	

2.2 LIMITING SAFETY SYSTEM SETTINGS

REACTOR TRIP SYSTEM INSTRUMENTATION SETPOINTS

2.2.1 The Reactor Trip System Instrumentation and Interlocks Setpoints shall be set consistent with the Trip Setpoint values shown in Table 2.2-1.

APPLICABILITY: As shown for each channel in Table 3.3-1.

ACTION:

a. With a Reactor Trip System Instrumentation or Interlock Setpoint less conservative than the value shown in the Trip Setpoint column but more conservative than the value shown in the Allowable Value Column of Table 2.2-1, adjust the Setpoint consistent with the Trip Setpoint value.

b. With the Reactor Trip System Instrumentation or Interlock Setpoint less conservative than the value shown in the Allowable Values column of Table 2.2-1, either:

1. Adjust the Setpoint consistent with the Trip Setpoint value of Table 2.2-1 and determine within 12 hours that Equation 2.2-1 was satisfied for the affected channel, or

2. Declare the channel inoperable and apply the applicable ACTION statement requirement of Specification 3.3.1 until the channel is restored to OPERABLE status with its Setpoint adjusted consistent with the Trip Setpoint value.

Equation 2.2-1 $Z + R + S \leq TA$

Where:

Z = The value from Column Z of Table 2.2-1 for the affected channel,

R = The "as measured" value (in percent span) of rack error for the affected channel,

S = Either the "as measured" value (in percent span) of the sensor error, or the value from Column S (Sensor Error) of Table 2.2-1 for the affected channel, and

TA = The value from Column TA (Total Allowance) of Table 2.2-1 for the affected channel.

TABLE 2.2.-1
REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
1. Manual Reactor Trip	N.A.	N.A.	N.A.	N.A.	N.A.
2. Power Range, Neutron Flux a. High Setpoint	7.5	4.56	0	≤109% of RTP*	≤111.1% of RTP*
b. Low Setpoint	8.3	4.56	0	≤25% of RTP*	≤27.1% of RTP*
3. Power Range, Neutron Flux, High Positive Rate	1.6	0.5	0	≤5% of RTP* with a time constant ≥ 2 seconds	≤6.3% of RTP* with a time constant ≥ 2 seconds
4. Power Range, Neutron Flux, High Negative Rate	1.6	0.5	0	≤5% of RTP* with a time constant ≥2 seconds	≤6.3% of RTP* with a time constant ≥2 seconds
5. Intermediate Range, Neutron Flux	17.0	8.4	0	≤25% of RTP*	≤31% of RTP*
6. Source Range, Neutron Flux	17.0	10	0	$\leq 10^5$ cps	$\leq 1.4 \times 10^5$ cps
7. Overtemperature ΔT	7.2(8.9#)	4.47(5.41#)	2.03(2.65#)	See Note 1	See Note 2
8. Overpower ΔT	4.3(4.9#)	1.3(1.24#)	1.2(1.7#)	See Note 3	See Note 4
9. Pressurizer Pressure-Low	4.0	2.21	1.5	≥1945 psig	≥1938 psig***
10. Pressurizer Pressure-High	7.5	4.96	0.5	≤2385 psig	≤2399 psig
11. Pressurizer Water Level-High	5.0	2.18	1.5	≤92% of instrument span	≤93.8% of instrument span
12. Reactor Coolant Flow-Low	2.5	1.77(1.41#)	0.6	≥90% of loop minimum measured flow**	≥89.2% (88.8%#) of loop minimum measured flow**

*RTP = RATED THERMAL POWER
**Loop minimum measured flow = 96,900 gpm
***Time constants utilized in the lead-lag controller for Pressurizer Pressure-Low are 2 seconds for lead and 1 second for lag. Channel calibration shall ensure that these time constants are adjusted to these values.
#Applicable upon deletion of RTD Bypass System.

Amendment No. 42 (Unit 1)
Amendment No. 35 (Unit 2)

POWER DISTRIBUTION LIMITS

3/4.2.3 REACTOR COOLANT SYSTEM FLOW RATE AND NUCLEAR ENTHALPY RISE HOT CHANNEL FACTOR

LIMITING CONDITION FOR OPERATION

3.2.3 The combination of indicated Reactor Coolant System total flow rate and R shall be maintained within the region of permissible operation shown on Figure 3.2-3 for four loop operation.

Where:

a. $R = \dfrac{F^N_{\Delta H}}{1.49\ [1.0 + 0.3\ (1.0 - P)]}$,

b. $P = \dfrac{\text{THERMAL POWER}}{\text{RATED THERMAL POWER}}$, and

c. $F^N_{\Delta H}$ = Measured values of $F^N_{\Delta H}$ obtained by using the movable incore detectors to obtain a power distribution map. The measured values of $F^N_{\Delta H}$ shall be used to calculate R since Figure 3.2-3 includes penalties for undetected feedwater venturi fouling of 0.1% and for measurement uncertainties of 2.1% for flow and 4% for incore measurement of $F^N_{\Delta H}$.

APPLICABILITY: MODE 1.

ACTION:

a. With the combination of Reactor Coolant System total flow rate and R within the region of restricted operation (flow rate less than 387,600 gpm), within 6 hours reduce the Power Range Neutron Flux-High Trip Setpoint to below the nominal setpoint by the same amount (% RTP) as the power reduction required by Figure 3.2-3.

b. With the combination of Reactor Coolant System total flow rate and R within the region of prohibited operation shown on Figure 3.2-3:

   1. Within 2 hours either:

      a) Restore the combination of Reactor Coolant System total flow rate and R to within the region of permissible operation, or

      b) Restore the combination of Reactor Coolant System total flow rate and R to within the region of restricted operation and comply with action a. above, or

      c) Reduce THERMAL POWER to less than 50% of RATED THERMAL POWER and reduce the Power Range Neutron Flux - High Trip Setpoint to less than or equal to 55% of RATED THERMAL POWER within the next 4 hours.

   2. Within 24 hours of initially being within the region of prohibited operation shown on Figure 3.2-3, verify through incore flux mapping and Reactor Coolant System total flow rate comparison that the combination of R and Reactor Coolant System total flow rate are restored to within the regions of restricted or permissible operation, or reduce THERMAL POWER to less than 5% of RATED THERMAL POWER within the next 2 hours.

LIMITING CONDITION FOR OPERATION

ACTION (Continued)

3. Identify and correct the cause of the out-of-limit condition prior to increasing THERMAL POWER above the reduced THERMAL POWER limit required by ACTION b.1.c) and/or b.2., above; subsequent POWER OPERATION may proceed provided that the combination of R and indicated Reactor Coolant System total flow rate are demonstrated, through incore flux mapping and Reactor Coolant System total flow rate comparison, to be within the regions of restricted or permissible operation shown on Figure 3.2-3 prior to exceeding the following THERMAL POWER levels:

   a) A nominal 50% of RATED THERMAL POWER,

   b) A nominal 75% of RATED THERMAL POWER, and

   c) Within 24 hours of attaining greater than or equal to 95% of RATED THERMAL POWER.

SURVEILLANCE REQUIREMENTS

4.2.3.1 The provisions of Specification 4.0.4 are not applicable.

4.2.3.2 The combination of indicated Reactor Coolant System total flow rate determined by process computer readings or digital voltmeter measurement and R shall be determined to be within the regions of restricted or permissible operation of Figure 3.2-3:

a. Prior to operation above 75% of RATED THERMAL POWER after each fuel loading, and

b. At least once per 31 Effective Full Power Days.

4.2.3.3 The indicated Reactor Coolant System total flow rate shall be verified to be within the regions of restricted or permissible operation of Figure 3.2-3 at least once per 12 hours when the most recently obtained value of R, obtained per Specification 4.2.3.2, is assumed to exist.

4.2.3.4 The Reactor Coolant System total flow rate indicators shall be subjected to a CHANNEL CALIBRATION at least once per 18 months. The measurement instrumentation shall be calibrated within 7 days prior to the performance of the calorimetric flow measurement.

4.2.3.5 The Reactor Coolant System total flow rate shall be determined by precision heat balance measurement at least once per 18 months.

PENALTIES OF 0.1% FOR UNDETECTED FEEDWATER VENTURI FOULING AND MEASUREMENT UNCERTAINTIES OF 2.1% FOR FLOW AND 4.0% FOR INCORE MEASUREMENT OF $F^N_{\Delta H}$ ARE INCLUDED IN THIS FIGURE.

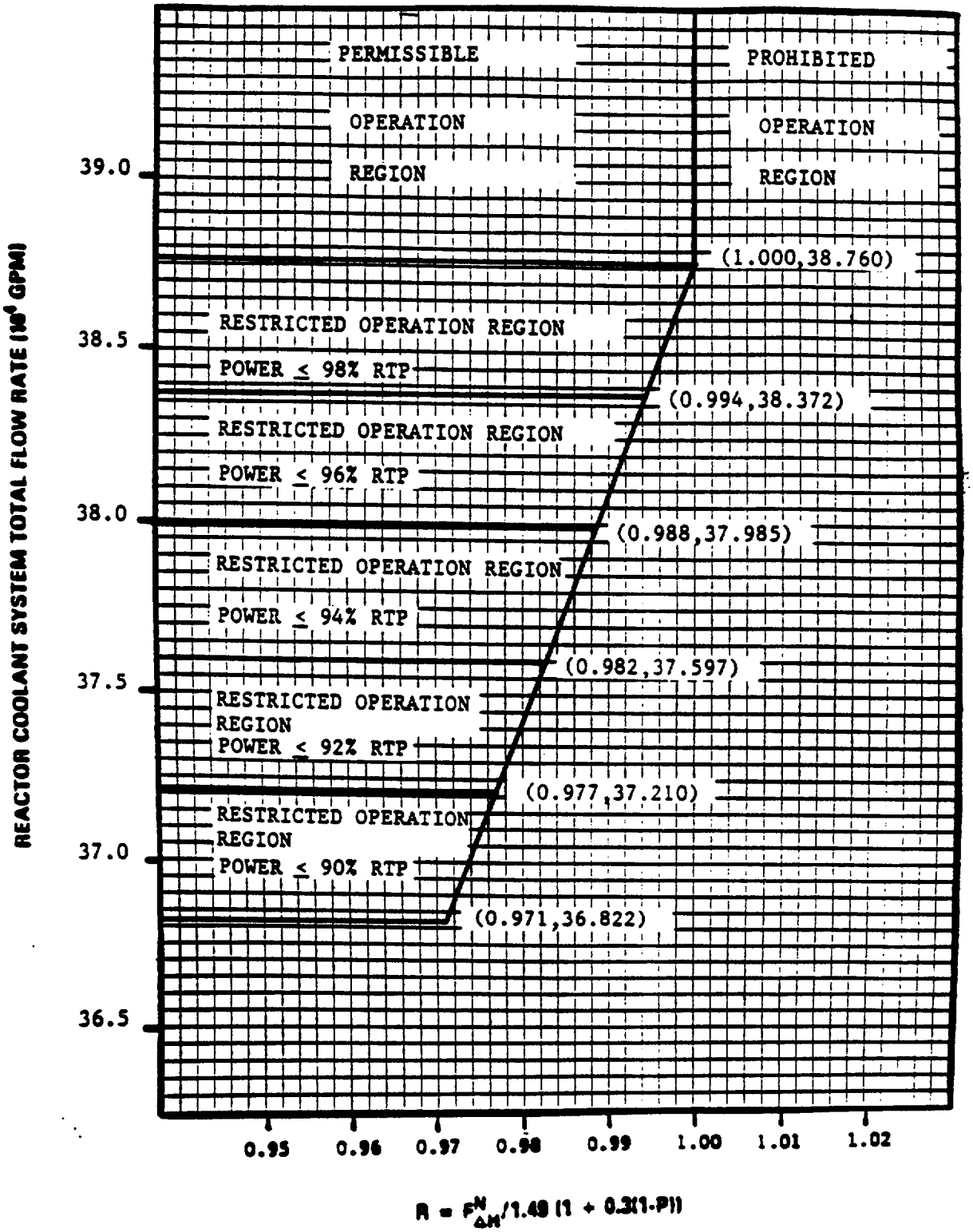

FIGURE 3.2-3
REACTOR COOLANT SYSTEM TOTAL FLOW RATE VERSUS R - FOUR LOOPS IN OPERATION

Amendment No. 42 (Unit 1)
Amendment No. 35 (Unit 2)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 42 TO FACILITY OPERATING LICENSE NPF-35

AND AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NPF-52

DUKE POWER COMPANY, ET AL.

DOCKET NOS. 50-413 AND 50-414

CATAWBA NUCLEAR STATION, UNITS 1 and 2

INTRODUCTION

By letter dated February 10, 1988, Duke Power Company, et al., (the licensee) proposed amendments to the operating licenses for Catawba Nuclear Station, Units 1 and 2, which would change Technical Specification (TS) Table 2.2-1, TS 3/4.2.3, and TS Figure 3.2-3 to reduce the required Reactor Coolant System (RCS) total flow from 396,100 gpm to 387,600 gpm.

EVALUATION

The licensee stated in its submittal requesting the TS changes that on January 13, 1988, following Catawba Unit 1 second refueling outage, a precision calorimetric test was conducted as required by TS surveillance requirement 4.2.3.5. This test resulted in the lowering of the RCS elbow tap flow coefficients which are used to convert elbow tap pressure drops to RCS flow rates. Upon insertion of the new constants into the operator aid computer, indicated RCS flow decreased to between 99.9% and 100.1% of the required flow. Because RCS flow was not consistently above 100% of the required flow, power was limited to 98% of the licensed power level of the unit in accordance with TS Figure 3.2-3. The fact that the RCS flow rate had remained constant throughout the past cycle and had returned to the same value (100.3%) following startup indicates that there is no degradation in actual RCS flow rate but that there is an amount of uncertainty attributuable to the RCS flow measurements.

An investigation into the indicated decreased RCS flow rate is being pursued by the licensee's and Westinghouse's personnel. One of the areas being investigated is the possibility that changes in RCS thermal streaming is causing a change in indicated hot and cold leg RTD temperatures. The precision heat balance calorimetric test is extremely sensitive to any uncertainty in this parameter.

All applicable FSAR postulated accidents and transients that have been analyzed used an assumed flow which is equal to, or conservative with respect to, the proposed TS flow of 387,600 gpm.

Certain Catawba FSAR Chapter 15 transients, those using the Improved Thermal Design Procedure (ITDP), are analyzed with a nominal flow rate of 387,600 gpm as outlined in the above submittal. The appropriate flow rate assumption for the Catawba FSAR Chapter 15 transients not using ITDP is the proposed TS

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PDR ADOCK 05000413
P PDR

minimum measured flow, 387,600 gpm, adjusted down by the flow uncertainty, 2.2%, to give 379,073 gpm. All of these transients are currently analyzed, as outlined in the above submittal, with flow rates less than this adjusted value and are therefore conservative.

The thermal hydraulic design analyses for the latest reload cores, Catawba 1 Cycle 3 and Catawba 2 Cycle 2, used the minimum measured flow of 387,600 gpm. It can be seen from this and from the preceding discussion of FSAR Chapter 15 analyses, that all applicable steady-state and transient core thermal-hydraulic analyses have been performed with flows equal to, or conservative with respect to, the proposed TS minimum measured flow.

RCS average temperature will remain unchanged with the change in minimum measured flow. This means that RCS initial fluid and metal stored energy will remain essentially unchanged. Further, a constant RCS average temperature implies that the driving temperature difference for primary-to-secondary heat transfer will remain essentially unchanged. These two parameters, initial energy content and rate of energy transfer across the steam generator tubes, are the means by which mass and energy releases influence containment response for the transients analyzed in Section 6.2.1 of the FSAR. Because the change in RCS flow is being made with a negligible change in RCS average temperature, the mass and energy release calculated in Sections 6.2.1.3 through 6.2.1.5 of the FSAR will not be affected.

On the basis of its review, the staff finds that the revision to the TSs will not adversely impact the accident analyses documented in Sections 6.2.1 and 15 of the FSAR nor the steady-state thermal-hydraulic reload design analyses discussed in Section 4.4 of the FSAR. Therefore, the changes to TS Table 2.2-1, TS 3/4.2.3, and TS Figure 3.2-3 are acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve changes to the use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes to surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposures. The NRC staff has made a determination that the amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Puruant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (53 FR 4793) on February 17, 1988. The Commission consulted with the state of South Carolina. No public comments were received, and the state of South Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Jabbour, PDII-3/DRPI/II

Dated: March 21, 1988

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 42, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into the license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:
Lawrence P. Crocker, Acting Director
Project Directorate II-3
Division of Reactor Projects I/II

Attachment:
Technical Specification Changes

Date of Issuance: March 21, 1988

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Acting PD	GLainas		
03/7/88	03/14/88		

Docket Nos.: 50-413
50-414

March 21, 1988

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No. 42 to Facility Operating License NPF-35 and Amendment No. 35 to Facility Operating License NPF-52-Catawba Nuclear Station, Units 1 and 2 (TACS 67115/67116)

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The amendments modify the Technical Specifications to reduce the required Ractor Coolant System total flow from 396,100 gpm to 387,600 gpm. The amendments are effective as of their date of issuance.

A copy of the related safety evaluation supporting Amendment No. 42 to Facility Operating License NPF-35 and Amendment No. 35 to Facility Operating License NPF-52 is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. 42 to NPF-35
2. Amendment No. 35 to NPF-52
3. Safety Evaluation

cc w/enclosure:
See next page

PD II-3	PD II-3	PD II-3
MRood	KJabbour:sw	Acting PD
3/7/88	3/7/88	3/7/88

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Date of Issuance: March 21, 1988

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Acting PD	GLainas		
03/ /88	03/ /88		

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