Docket Nos.: 50-413
and 50-414

November 24, 1987

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No. 34 to Facility Operating License NPF-35 and Amendment No. 25 to Facility Operating License NPF-52 - Catawba Nuclear Station, Units 1 and 2 (TACs 59641/60928)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 34 to Facility Operating License NPF-35 and Amendment No. 25 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated July 31, 1985, and supplemented November 8 and December 26, 1985, and March 7, 1986.

The amendments would add an action statement to Technical Specification 3/4.2.3, and would modify Figure 3.2-3 to delete the DNB limit line and add a graduated scale to allow a tradeoff of reactor coolant system flow against reactor thermal power level. The amendments are effective as of the date of issuance.

A copy of the related safety evaluation supporting Amendment No. 34 to Facility Operating License NPF-35 and Amendment No. 25 to Facility Operating License NPF-52 is enclosed.

Notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

/S/

Kahtan Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. 34 to NPF-35
2. Amendment No. 25 to NPF-52
3. Safety Evaluation

cc w/encl:
See next page

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02/04/87	02/04/87	11/19/87
11/5/87	11/5/87	

November 24, 1987

AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NPF-35 -
CATAWBA NUCLEAR POWER STATION, UNIT 1
AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NPF-52 -
CATAWBA NUCLEAR POWER STATION, UNIT 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc., (licensees) dated July 31, 1985, and supplemented November 8 and December 26, 1985, March 7, 1986, and July 1, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 34 , and the Environmental Protection Plan

contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

Lawrence P. Crocker, Acting Director
Project Directorate II-3
Division of Reactor Projects I/II

Attachment:
Technical Specification Changes

Date of Issuance: November 24, 1987

*SEE PREVIOUS CONCURRENCE

PWR#4/DPWR-A	PWR#4/DPWR-A	OGC-Bethesda	PDII-3/DRPI/II
*MDuncan:mac	*KJabbour		Acting Director
02/4/87	02/4/87	11/10/87	11/19/87
11/5/87	11/5/87		

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency, (licensees) dated July 31, 1985, and supplemented November 8 and December 26, 1985, March 7, 1986, and July 1, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 25, and the Environmental Protection Plan

contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Lawrence P. Crocker, Acting Director
Project Directorate II-3
Division of Reactor Projects I/II

Attachment:
Technical Specification Changes

Date of Issuance: November 24, 1987

*SEE PREVIOUS CONCURRENCE

PWR#4/DPWR-A	PWR#4/DPWR-A	OGC-Bethesda	PDII-3/DRPI/II
*MDuncan:mac	*KJabbour		Acting Director
02/4/87	02/4/87	11/10/87	11/19/87
11/5/87	11/5/87		

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 25

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

Amended Page	Overleaf Page
3/4 2-9	
3/4 2-10	
3/4 2-11	3/4 2-12

3/4.2.3 REACTOR COOLANT SYSTEM FLOW RATE AND NUCLEAR ENTHALPY RISE HOT CHANNEL FACTOR

LIMITING CONDITION FOR OPERATION

3.2.3 The combination of indicated Reactor Coolant System total flow rate and R shall be maintained within the region of allowable operation shown on Figure 3.2-3 for four loop operation.

Where:

a. $R = \dfrac{F^N_{\Delta H}}{1.49\,[1.0 + 0.3\,(1.0 - P)]}$,

b. $P = \dfrac{\text{THERMAL POWER}}{\text{RATED THERMAL POWER}}$, and

c. $F^N_{\Delta H}$ = Measured values of $F^N_{\Delta H}$ obtained by using the movable incore detectors to obtain a power distribution map. The measured values of $F^N_{\Delta H}$ shall be used to calculate R since Figure 3.2-3 includes penalties for undetected feedwater venturi fouling of 0.1% and for measurement uncertainties of 2.1% for flow and 4% for incore measurement of $F^N_{\Delta H}$.

APPLICABILITY: MODE 1.

ACTION:

a. With the combination of Reactor Coolant System total flow rate and R within the region of acceptable operation with the flow rate less than 396100 gpm, within 6 hours reduce the Power Range Neutron Flux-High Trip Setpoint to below the nominal setpoint by the same amount (% RTP) as the power reduction required by Figure 3.2-3.

b. With the combination of Reactor Coolant System total flow rate and R outside the region of acceptable operation shown on Figure 3.2-3:

   1. Within 2 hours either:

      a) Restore the combination of Reactor Coolant System total flow rate and R to within the above limits, or

      b) Reduce THERMAL POWER to less than 50% of RATED THERMAL POWER and reduce the Power Range Neutron Flux - High Trip Setpoint to less than or equal to 55% of RATED THERMAL POWER within the next 4 hours.

   2. Within 24 hours of initially being outside the above limits, verify through incore flux mapping and Reactor Coolant System total flow rate comparison that the combination of R and Reactor Coolant System total flow rate are restored to within the above limits, or reduce THERMAL POWER to less than 5% of RATED THERMAL POWER within the next 2 hours.

LIMITING CONDITION FOR OPERATION

ACTION (Continued)

3. Identify and correct the cause of the out-of-limit condition prior to increasing THERMAL POWER above the reduced THERMAL POWER limit required by ACTION a.2. and/or b., above; subsequent POWER OPERATION may proceed provided that the combination of R and indicated Reactor Coolant System total flow rate are demonstrated, through incore flux mapping and Reactor Coolant System total flow rate comparison, to be within the region of acceptable operation shown on Figure 3.2-3 prior to exceeding the following THERMAL POWER levels:

   a) A nominal 50% of RATED THERMAL POWER,

   b) A nominal 75% of RATED THERMAL POWER, and

   c) Within 24 hours of attaining greater than or equal to 95% of RATED THERMAL POWER.

SURVEILLANCE REQUIREMENTS

4.2.3.1 The provisions of Specification 4.0.4 are not applicable.

4.2.3.2 The combination of indicated Reactor Coolant System total flow rate determined by process computer readings or digital voltmeter measurement and R shall be determined to be within the region of acceptable operation of Figure 3.2-3:

a. Prior to operation above 75% of RATED THERMAL POWER after each fuel loading, and

b. At least once per 31 Effective Full Power Days.

4.2.3.3 The indicated Reactor Coolant System total flow rate shall be verified to be within the region of acceptable operation of Figure 3.2-3 at least once per 12 hours when the most recently obtained value of R, obtained per Specification 4.2.3.2, is assumed to exist.

4.2.3.4 The Reactor Coolant System total flow rate indicators shall be subjected to a CHANNEL CALIBRATION at least once per 18 months. The measurement instrumentation shall be calibrated within 7 days prior to the performance of the calorimetric flow measurement.

4.2.3.5 The Reactor Coolant System total flow rate shall be determined by precision heat balance measurement at least once per 18 months.

PENALTIES OF 0.1% FOR UNDETECTED FEEDWATER VENTURI FOULING AND MEASUREMENT UNCERTAINTIES OF 2.1% FOR F V AND 4.0% FOR INCORE MEASUREM T OF $F^N_{\Delta H}$ ARE INCLUDED IN THIS FIGURE.

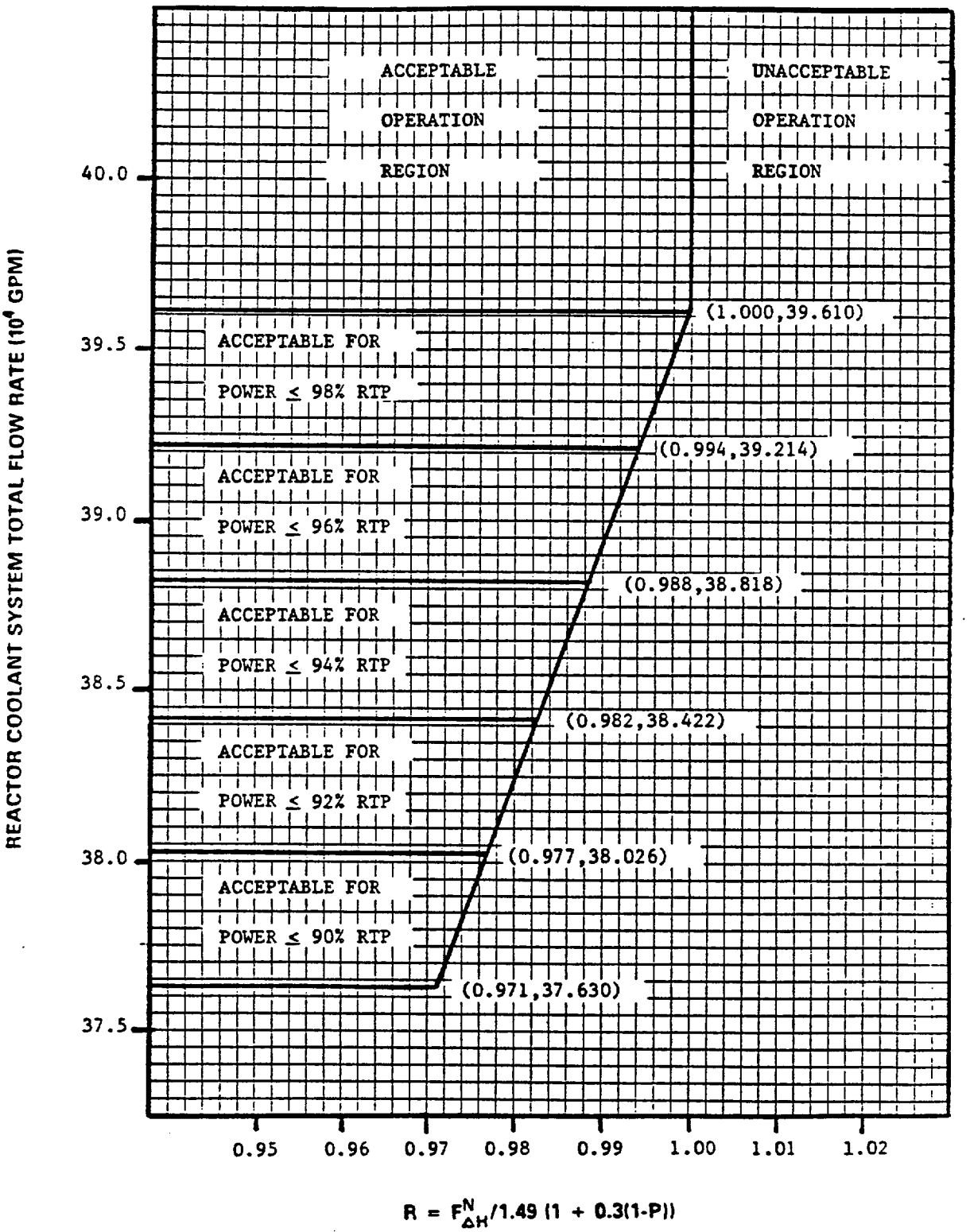

FIGURE 3.2-3
REACTOR COOLANT SYSTEM TOTAL FLOW RATE VERSUS R - FOUR LOOPS IN OPERATION

3/4.2.4 QUADRANT POWER TILT RATIO

LIMITING CONDITION FOR OPERATION

3.2.4 The QUADRANT POWER TILT RATIO shall not exceed 1.02 above 50% of RATED THERMAL POWER.

APPLICABILITY: MODE 1.*

ACTION:

a. With the QUADRANT POWER TILT RATIO determined to exceed 1.02 but less than or equal to 1.09:

1. Calculate the QUADRANT POWER TILT RATIO at least once per hour until either:

a) The QUADRANT POWER TILT RATIO is reduced to within its limit, or

b) THERMAL POWER is reduced to less than 50% of RATED THERMAL POWER.

2. Within 2 hours either:

a) Reduce the QUADRANT POWER TILT RATIO to within its limit, or

b) Reduce THERMAL POWER at least 3% from RATED THERMAL POWER for each 1% of indicated QUADRANT POWER TILT RATIO in excess of 1 and similarly reduce the Power Range Neutron Flux-High Trip Setpoints within the next 4 hours.

3. Verify that the QUADRANT POWER TILT RATIO is within its limit within 24 hours after exceeding the limit or reduce THERMAL POWER to less than 50% of RATED THERMAL POWER within the next 2 hours and reduce the Power Range Neutron Flux-High Trip Setpoints to less than or equal to 55% of RATED THERMAL POWER within the next 4 hours; and

4. Identify and correct the cause of the out-of-limit condition prior to increasing THERMAL POWER; subsequent POWER OPERATION above 50% of RATED THERMAL POWER may proceed provided that the QUADRANT POWER TILT RATIO is verified within its limit at least once per hour for 12 hours or until verified acceptable at 95% or greater RATED THERMAL POWER.

*See Special Test Exceptions Specification 3.10.2.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NPF-35

AND AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NPF-52

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DUKE POWER COMPANY, ET AL.

I. INTRODUCTION

By letter dated July 31, 1985, and supplemented November 8 and December 26, 1985, and March 7, 1986, Duke Power Company, et al., (the licensee) proposed changes to Figure 3.2-3 of Technical Specification (TS) 3/4.2.3 for Catawba Nuclear Station, Units 1 and 2. The changes would delete the DNB limit line and add a graduated scale to allow tradeoff of reactor coolant system (RCS) flow against reactor thermal power (RTP) level.

In response to NRC staff concerns, on July 1, 1987, the licensee supplemented its earlier request to provide: (a) an additional action statement for reductions in the high flux trip setpoint at various operating conditions, (b) additional information concerning the adequacy of the overtemperature Delta-T setpoint for various DNBR events, and (c) additional information concerning the F Delta-H equation. Because the July 1, 1987, submittal provided clarification of, proposed additional restrictions on, certain aspects of the request, the substance of the changes noticed in the Federal Register and the proposed no significant hazards determination were not affected.

II. EVALUATION

1. Operation with RCS flow less than 396,100 gpm

The licensee proposes that Figure 3.2-3 titled "Reactor Coolant System Total Flow Rate Versus R-Four Loops in Operation" be modified to permit operation at a reduced flow rate (i.e., about 95% of full flow) which would be compensated by an appropriate reduction in reactor power level. The reduced power level ensures that the thermal margin is maintained at a value equivalent to that for full flow. In addition, the licensee proposes to delete the DNB limit line to prevent operation misinterpretation because the rod bow factor (R_2) is not associated with TS 3/4.2.3 or Figure 3.2-3 and thus the DNB limit line serves no useful purpose.

The current best estimate flow at Catawba Unit 1 is 399,122 gpm which is only 0.76 of 1% above the TS limit of 396,100 gpm. Considering an expected variance in flow measurement of about 2.2%, the licensee believes the proposed revision is needed because of the severity of the existing TS if any flow measurement indicates a flow slightly less than 396,100 gpm.

The licensee proposes to reduce RTP level by 2% for each 1% reduction in the measured RCS flow below the TS flow requirement for 100% power which is equal to 396,100 gpm. Westinghouse calculations show, with penalties added for conservatism, that a 5% reduction in flow results in a 4.4% reduction in power. The staff's independent audit (using W-3 correlation sensitivity factors reported in References (1) and (2)) show that, for the worst case reported in Reference (2), a 5% reduction in flow requires a 4.32% reduction in power. The licensee's plan to use a 10% reduction in power for a 5% reduction in flow is conservative. Furthermore, this ratio of RTP level to RCS flow was used for the McGuire Nuclear Station TS and has been reviewed and approved by the NRC staff.

Based on the above evaluation, the staff concludes that the proposed modification to Figure 3.2-3 of TS 3/4.2.3 for using a 2/1 power to flow relationship for a maximum of 5% reduction in flow is acceptable.

The staff also finds the elimination of the DNB limit line in Figure 3.2-3 to be acceptable as it was originally for an "R_2" value associated with rod bow penalty. In the November 8, 1985, letter, the licensee stated that a rod bow penalty is not applied to Catawba as there is approximately 10% margin between the safety analysis and design limit DNBR values. This is partially used to offset a 2.7% DNBR rod bow penalty. The staff agrees with the licensee's statements and licensee's treatment of rod bow penalty.

2. Reduction in High Flux Trip Setpoint

The justification for the 2% RTP reduction per 1% RCS flow reduction was based upon the sensitivities of DNBR to power and flow determined by several independent sources using different assumptions and computer codes. The licensee stated that the plant response to an FSAR Chapter 15 transient initiated from the reduced flow/reduced power operating regions would continue to satisfy the applicable acceptance criteria based upon margins available in the Chapter 15 analyses, protection system setpoint calculations, and thermal-hydraulic analyses including the use of a minimum allowable DNBR greater than that justified by the Improved Thermal Design Procedure.

The licensee confirmed that adequate margins exist in the related analyses to justify the proposed TS changes. The limiting events in regard to the proposed operating regions are the reactivity insertion transients which increase reactor power. The RCCA group withdrawal analyses provide the most limiting cases with the reactor trip signal assumed to be due to either the Overtemperature Delta-T or the Power Range Neutron Flux-High Trip Setpoint trip functions. The licensee determined that the low flow penalties for the RCCA group withdrawals initiated from the proposed operating regions would require dedication of most of the available DNBR margin. Thus, the use of protection system setpoint reductions and more restrictive limits on core power distributions, as described below, to compensate for the potential RCS flow reduction was appropriate.

By letter dated July 1, 1987, the licensee provided a revision to the proposed TS changes which includes an action statement requiring a reduction in the Power Range Neutron Flux-High Trip Setpoint to below the normal setpoint by the same amount (% RTP) as the power reduction required by Figure 3.2-3 (2% RTP per 1% RCS flow). By maintaining the 2% RTP per 1% RCS flow trade-off at the trip conditions as well as the steady-state conditions, the high flux setpoint reduction ensures the DNBR margin existing in the FSAR analyses is maintained for those events assumed to utilize the Power Range Neutron Flux-High Trip Setpoint trip function.

Based on its review, the staff finds that the protection system setpoint reduction is acceptable because it appropriately compensates for the potential RCS flow reduction and is sufficiently conservative for DNBR events.

3. Overtemperature Delta-T Trip Function

For those FSAR Chapter 15 events which assume Overtemperature Delta-T trip function protection, the licensee concluded that an administrative control on a term within the setpoint equation and the restriction of core power distributions are adequate to ensure that margins in the analyses are maintained. The term Delta-T_o in the setpoint equation (TS Table 2.2-1) is defined as the indicated Delta-T at rated thermal power. Delta-T_o is defined as the equivalent Delta-T at 100% RTP. The effect of the greater of the measured RCS flow or 396,100 gpm, results in an effective Overtemperature Delta-T setpoint reduction and maintains DNBR margins for those cases reliant upon the overtemperature Delta-T trip. The ratio of the measured Delta-T to Delta-T_o is thus equivalent to %RTP and is compared to the Overtemperature Delta-T trip setpoint. By maintaining Delta-T_o at or below the equivalent Delta-T at 100% RTP and 396,100 gpm, any flow deficiency will result in an increase in the actual Delta-T and an overestimation of power level. The overestimation of power is equivalent to a trip setpoint reduction of approximately 1% RTP per 1% flow deficiency.

Based on its review, the staff finds that this compensation is acceptable for maintaining the effectiveness of the Overtemperature Delta-T protection.

4. F Delta-H Equation

The F Delta-H equation is in TS 3/4.2.3 which states that "the combination of indicated RCS total flow rate and R shall be maintained in the region of allowable operation shown in Figure 3.2-3, for four loop operation."

The licensee has factored the 2/1 power to flow tradeoff into the F Delta-H equation and in Figure 3.2-3. For example, at the maximum power reduction of 10% corresponding to a flow reduction of 5% and with F Delta-H maintained at the full power value of 1.49, the value of R is 0.971. This condition is shown in Figure 3.2-3 where RTP equals 90% and the RCS flow is 95% of the required flow or .95 X (396,100 gpm) = 376,3000 gpm.

The limitation placed on F Delta-H for reduced RCS flow conditions prevents the usual allowance for increasing F Delta-H at reduced power levels. Reductions in power levels combined with a RCS flow deficiency and increased radial peaking do not maintain DNBR margin and thus the F Delta-H restrictions in Figure 3.2-3 are required.

Based on its review, the staff finds that Figure 3.2-3 was appropriately adjusted and, therefore, it is acceptable.

III. ENVIRONMENTAL CONSIDERATION

The amendments involve a change in use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposures. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there have been no public comments on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

IV. CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (51 FR 30563) on August 27, 1986, and consulted with the state of South Carolina. No public comments were received, and the state of South Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

References

(1) Chelemer, H., L. H. Boman, and D. R. Sharp, WCAP-8567, "Improved Thermal Design Procedure," Westinghouse Electric Corporation, Pittsburgh, Pennsylvania, July 1975.

(2) Hesson, G. M., and J. M. Cuta, FATE-79-101, "Analysis of the Sensitivity of Calculated MDNBR to Eight Selected DNB Parameters," Battelle Pacific Northwest Laboratories, Richland, Washington, March 1979.

Principal Contributors: Kahtan Jabbour, PDII-3/DRPI/II
Harry Balukjian, SRXB/DEST

Dated: November 24, 1987