

September 5, 1997

Mr. H. L. Sumner, Jr.
Vice President - Nuclear
Hatch Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: ISSUANCE OF AMENDMENTS - EDWIN I. HATCH NUCLEAR PLANT,
UNITS 1 AND 2 (TAC NOS. M97702 AND M97703)

Dear Mr. Sumner:

The Nuclear Regulatory Commission has issued the enclosed Amendment No.208 to Facility Operating License DPR-57 and Amendment No. 150to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated January 7, 1997, as supplemented July 2, 1997.

The amendments revise the plant TS associated with surveillance requirement testing that requires manually actuating every safety/relief valve during each unit startup from a refueling outage.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Ngoc B. (Tommy) Le, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

Enclosures:

1. Amendment No.208to DPR-57
2. Amendment No.150to NPF-5
3. Safety Evaluation

DISTRIBUTION

Docket File	OGC
PUBLIC	ACRS
PDII-2 RF	Ghill(4)
BBoger	JJohnson,RII
WBeckner,TSB	PSkinner,RII
THarris (e-mail SE only)	
DTerao	GHammer

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cc w/encl: See next page

DOCUMENT NAME: G:\HATCH\HAT97702.AMD

OFFICE	DRPE/PD22/PM	DRPE/PD22/LA	OGC <i>WAB</i>	DRPE/PD22/D
NAME	NB.LE:cn <i>le</i>	LBERRY <i>LB</i>	R.Bachmann	H.BERKOW
DATE	8/19/97	8/19/97	8/20/97	9/4/97
COPY	(YES) NO	(YES) NO	YES NO	YES (NO)

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cc w/encl: See next page

DOCUMENT NAME: G:\HATCH\HAT97702.AMD

OFFICE	DRPE/PD22/PM	DRPE/PD22/LA	OGC <i>WAB</i>	DRPE/PD22/D
NAME	NB.LE:cn <i>Le</i>	LBERRY <i>LB</i>	<i>R.Bachmann</i>	<i>H.BERKOW</i>
DATE	<i>8/19</i> /97	<i>8/19</i> /97	<i>8/20</i> 197	<i>9/4</i> 197
COPY	<input checked="" type="checkbox"/> YES NO	<input checked="" type="checkbox"/> YES NO	YES NO	YES <input checked="" type="checkbox"/> NO

OFFICIAL RECORD COPY



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 5, 1997

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Vice President - Nuclear
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The amendments revise the plant TS associated with surveillance requirement testing that requires manually actuating every safety/relief valve during each unit startup from a refueling outage.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Ngoc B. Le".

Ngoc B. (Tommy) Le, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

Enclosures:

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2. Amendment No. 150 to NPF-5
3. Safety Evaluation

cc w/encl: See next page

Edwin I. Hatch Nuclear Plant

cc:

**Mr. Ernest L. Blake, Jr.
Shaw, Pittman, Potts
and Trowbridge
2300 N Street, NW.
Washington, DC 20037**

**Mr. D. M. Crowe
Manager, Licensing
Southern Nuclear Operating
Company, Inc.
P. O. Box 1295
Birmingham, Alabama 35201-1295**

**Resident Inspector
U.S. Nuclear Regulatory Commission
11030 Hatch Parkway North
Baxley, Georgia 31513**

**Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30303**

**Mr. Charles H. Badger
Office of Planning and Budget
Room 610
270 Washington Street, SW.
Atlanta, Georgia 30334**

**Harold Reheis, Director
Department of Natural Resources
205 Butler Street, SE., Suite 1252
Atlanta, Georgia 30334**

**Steven M. Jackson
Senior Engineer - Power Supply
Municipal Electric Authority
of Georgia
1470 Riveredge Parkway, NW
Atlanta, Georgia 30328-4684**

**Charles A. Patrizia, Esquire
Paul, Hastings, Janofsky & Walker
10th Floor
1299 Pennsylvania Avenue
Washington, DC 20004-9500**

**Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513**

**Mr. W. G. Harriston, III
President and Chief Executive
Officer
Southern Nuclear Operating
Company, Inc.
P. O. Box 1295
Birmingham, Alabama 35201-1295**

**Mr. J. D. Woodard
Executive Vice President
Southern Nuclear Operating
Company, Inc.
P. O. Box 1295
Birmingham, Alabama 35201-1295**

**Mr. P. W. Wells
General Manager, Edwin I. Hatch
Nuclear Plant
Southern Nuclear Operating
Company, Inc.
P. O. Box 439
Baxley, Georgia 31513**

**Mr. R. D. Barker
Program Manager
Fossil & Nuclear Operations
Oglethorpe Power Corporation
2100 East Exchange Place
P. O. Box 1349
Tucker, Georgia 30085-1349**



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.208
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by the Georgia Power Company and Southern Nuclear Operating Company, Inc. (Southern Nuclear), acting for themselves, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated January 7, 1997, as supplemented by letter dated July 2, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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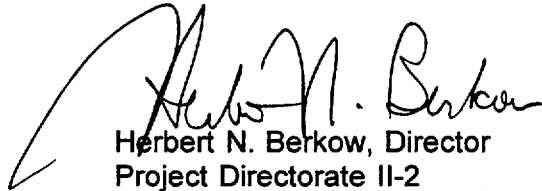
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

- (2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 208, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: September 5, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 208

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove

Insert

3.4-8

3.4-8

3.5-6

3.5-6

3.6-19

3.6-19

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY								
SR 3.4.3.1	<p>Verify the safety function lift setpoints of the S/RVs are as follows:</p> <table style="margin-left: 40px;"> <thead> <tr> <th style="text-align: center;"><u>Number of S/RVs</u></th> <th style="text-align: center;"><u>Setpoint (psig)</u></th> </tr> </thead> <tbody> <tr> <td style="text-align: center;">4</td> <td style="text-align: center;">1110 ± 33.3</td> </tr> <tr> <td style="text-align: center;">4</td> <td style="text-align: center;">1120 ± 33.6</td> </tr> <tr> <td style="text-align: center;">3</td> <td style="text-align: center;">1130 ± 33.9</td> </tr> </tbody> </table> <p>Following testing, lift settings shall be within ± 1%.</p>	<u>Number of S/RVs</u>	<u>Setpoint (psig)</u>	4	1110 ± 33.3	4	1120 ± 33.6	3	1130 ± 33.9	<p>In accordance with the Inservice Testing Program</p>
<u>Number of S/RVs</u>	<u>Setpoint (psig)</u>									
4	1110 ± 33.3									
4	1120 ± 33.6									
3	1130 ± 33.9									

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.5.1.9 -----NOTE----- Not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. -----</p> <p>Verify, with reactor pressure \leq 165 psig, the HPCI pump can develop a flow rate \geq 4250 gpm against a system head corresponding to reactor system pressure.</p>	<p>18 months</p>
<p>SR 3.5.1.10 -----NOTE----- Vessel injection/spray may be excluded. -----</p> <p>Verify each ECCS injection/spray subsystem actuates on an actual or simulated automatic initiation signal.</p>	<p>18 months</p>
<p>SR 3.5.1.11 -----NOTE----- Valve actuation may be excluded. -----</p> <p>Verify the ADS actuates on an actual or simulated automatic initiation signal.</p>	<p>18 months</p>
<p>SR 3.5.1.12 Verify each ADS valve relief mode actuator strokes when manually actuated.</p>	<p>18 months</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.1.6.1	Verify each LLS valve relief mode actuator strokes when manually actuated.	18 months
SR 3.6.1.6.2	<p>-----NOTE----- Valve actuation may be excluded. -----</p> <p>Verify the LLS System actuates on an actual or simulated automatic initiation signal.</p>	18 months



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GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 150
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by the Georgia Power Company and Southern Nuclear Operating Company, Inc. (Southern Nuclear), acting for themselves, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated January 7, 1997, as supplemented by letter dated July 2, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

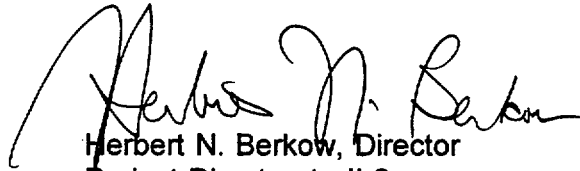
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

- (2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 15 are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: September 5, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 150

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove

3.4-8

3.5-6

3.6-19

Insert

3.4-8

3.5-6

3.6-19

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY								
SR 3.4.3.1	<p>Verify the safety function lift setpoints of the S/RVs are as follows:</p> <table border="1"> <thead> <tr> <th><u>Number of S/RVs</u></th> <th><u>Setpoint (psig)</u></th> </tr> </thead> <tbody> <tr> <td>4</td> <td>1120 ± 33.6</td> </tr> <tr> <td>4</td> <td>1130 ± 33.9</td> </tr> <tr> <td>3</td> <td>1140 ± 34.2</td> </tr> </tbody> </table> <p>Following testing, lift settings shall be within ± 1%.</p>	<u>Number of S/RVs</u>	<u>Setpoint (psig)</u>	4	1120 ± 33.6	4	1130 ± 33.9	3	1140 ± 34.2	<p>In accordance with the Inservice Testing Program</p>
<u>Number of S/RVs</u>	<u>Setpoint (psig)</u>									
4	1120 ± 33.6									
4	1130 ± 33.9									
3	1140 ± 34.2									

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.5.1.9 -----NOTE----- Not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. -----</p> <p>Verify, with reactor pressure \leq 165 psig, the HPCI pump can develop a flow rate \geq 4250 gpm against a system head corresponding to reactor pressure.</p>	<p>18 months</p>
<p>SR 3.5.1.10 -----NOTE----- Vessel injection/spray may be excluded. -----</p> <p>Verify each ECCS injection/spray subsystem actuates on an actual or simulated automatic initiation signal.</p>	<p>18 months</p>
<p>SR 3.5.1.11 -----NOTE----- Valve actuation may be excluded. -----</p> <p>Verify the ADS actuates on an actual or simulated automatic initiation signal.</p>	<p>18 months</p>
<p>SR 3.5.1.12 Verify each ADS valve relief mode actuator strokes when manually actuated.</p>	<p>18 months</p>

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.6.1.6.1 Verify each LLS valve relief mode actuator strokes when manually actuated.	18 months
SR 3.6.1.6.2 -----NOTE----- Valve actuation may be excluded. ----- Verify the LLS System actuates on an actual or simulated automatic initiation signal.	18 months



UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 208 TO FACILITY OPERATING LICENSE DPR-57
AND AMENDMENT NO. 150 TO FACILITY OPERATING LICENSE NPF-5

SOUTHERN NUCLEAR OPERATING COMPANY, INC., ET AL.

EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-321 AND 50-366

1.0 INTRODUCTION

By letter dated January 7, 1997, as supplemented July 2, 1997, Georgia Power Company and Southern Nuclear Operating Company, Inc. et al. (the licensee) proposed license amendments to change the Technical Specifications (TS) for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The proposed changes are associated with surveillance testing that would require manually actuating every safety/relief valve (S/RV) during each unit startup from a refueling outage. The proposed changes provide an alternate method of testing of the S/RV during shutdown conditions rather than during unit startup as is currently done. The July 2, 1997, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

In its letter dated January 7, 1997, the licensee proposed a change to modify the S/RV TS surveillance requirement to perform manual actuations once every 18 months as part of startup testing activities. The licensee supplemented this request with a letter dated July 2, 1997, which provided additional information regarding the testing proposed for the plant S/RVs. The specific TS change evaluated herein is for TS Surveillance Requirements (SRs) 3.4.3.2, 3.5.1.12, and 3.6.1.6.1. Current relief mode SR 3.5.1.12, for the Automatic Depressurization System (ADS), and SR 3.6.1.6.1, for the Low-Low Set (LLS) system, require that each S/RV be actuated at pressure conditions. The licensee proposes to revise these SRs to require the S/RVs to be manually actuated in the relief mode during an outage before steam is generated. The licensee also proposes to delete safety mode SR 3.4.3.2, which requires that each S/RV be manually actuated.

3.0 BACKGROUND

Each plant S/RV is a Target Rock 2-Stage pilot-operated S/RV with an attached pneumatic actuator. There are a total of 11 S/RVs installed on each of the Hatch units' main steam systems, all of which operate in the safety mode for overpressure protection. In the safety mode, each S/RV opens when system pressure exceeds the self-actuating setpoint pressure, which is controlled by the setpoint spring acting down on the pilot disk. When the pilot disk

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opens, the resulting differential pressure across the main piston opens the main disk to relieve system overpressure. The relief mode ADS and LLS functions are accomplished by an automatic control circuit that applies electric power to solenoids which provide control air to the pneumatic diaphragm assembly (i.e., auxiliary actuating device) that removes the pilot spring force allowing the pilot disk to open. Once the pilot disk is open, steam pressure provides the necessary force to open the main S/RV disk. In both Hatch, Units 1 and 2, seven S/RVs are for the ADS, and the remaining four are for the LLS.

Currently, both Hatch Units 1 and 2 TS SRs 3.4.3.2, 3.5.1.12, and 3.6.1.6.1 require that, at least once every 18 months, the S/RVs be functionally exercised with reactor steam pressure. This testing is performed with system pressure of at least 920 psig during reactor startup following an outage. The licensee has linked this testing to leakage of the valves, providing several examples of instances where these S/RVs began to leak after the in situ stroke testing was performed. The licensee states that, if the pilot stage leakage is severe enough, the S/RV setpoint could drift and lead to spurious actuation and/or failure of the valve to reseal. The licensee also states that S/RV leakage can cause increases in suppression pool temperature and level and increased use of the Residual Heat Removal (RHR) system for suppression pool cooling. Further, the licensee states that S/RV leakage reduces electrical generating capacity and could increase radiation hazard for personnel.

3.1 Proposed Technical Specification Changes

The licensee proposes to revise TS SRs 3.5.1.12 and 3.6.1.6.1 and to eliminate TS SR 3.4.3.2. This would allow S/RV functional exercising to be performed before reactor steam is generated. The licensee states that with the proposed changes, the solenoid valves would be energized, the actuator would stroke, and the pilot disk rod lift would be measured, but that, because there would be no steam pressure, the pilot disk would not be lifted. The licensee states that the ability of the pilot disk to open would be shown by the required safety mode actuation performed by a bench test. Further, the licensee states that all 11 S/RVs and three or four main disk assemblies are sent to Wyle Labs and tested with steam pressure each refueling outage. The licensee states that as a result, the main disks are fully stroked and stroke timed at approximately a 5-year frequency. This testing also verifies the reseal pressure and closure of the S/RVs. The licensee added that due to the test facility limitations, flow through the main stage is limited such that full flow is not discharged. However, the discharge restriction is not accomplished by impeding the movement of the main disk, but by restricting the size of the discharge path downstream of the main disk discharge. The licensee states that the combination of both the testing at Wyle Labs and that performed after the valves are reinstalled, completely demonstrates operability of the S/RVs.

3.2 Staff Evaluation

The staff has reviewed the licensee's proposed TS changes and agrees that the current TS requirement to perform the in situ stroke testing of the S/RVs may contribute to undesirable S/RV leakage and could result in spurious actuation of the valves during power operation and/or failure to reseal, increased use of RHR for suppression pool cooling, decreased generating capacity, and increased radiation hazard. The testing proposed by the licensee

provides periodic verification of all of the individual S/RV components, which are currently being tested except that some tests are to be performed at a test facility instead of in situ with reactor steam. The staff agrees that the proposed surveillance and testing of the ADS S/RVs and associated components provide assurance of adequate valve operation.

One difference between the current TS-required stroking of the S/RV main stages during plant startup and the licensee's proposal is that, with the proposed testing, there would be less frequent stroking of the S/RV main stages. Instead of stroke testing the S/RV main stages after each refueling outage, only three or four main stages would be stroked each refueling outage, which would result in an approximate 5-year frequency. However, because the main stage disks of these valves have a proven history of reliable performance at boiling water reactor plants, the staff agrees that the proposed stroking of three or four main stage disks each refueling is adequate.

Another difference between the current TS-required stroking and the licensee's proposal is that, when performing the testing in situ as required by the current TS, the testing verifies that the S/RV discharge line is not blocked. However, the licensee stated that there are foreign material exclusion controls in place at the plant which, together with the horizontal orientation of the discharge line mating connections, provide reasonable assurance that no obstruction exists in the lines. The staff agrees that the likelihood of blockage of an S/RV discharge line is remote as demonstrated by operational history and that the licensee has acceptably addressed this concern.

Based on the above evaluation, the staff concludes that the licensee has demonstrated the adequacy of the proposed changes to the Hatch Nuclear Plant, Units 1 and 2, TS. The proposed changes provide for testing of the S/RVs to demonstrate proper operation without the need for in situ stroking of the S/RV main stages with reactor steam. Therefore, the proposed changes to TS SRs 3.4.3.2, 3.5.1.12, and 3.6.1.6.1 for Hatch Nuclear Plant, Units 1 and 2, are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (62 FR 4350 dated January 29, 1997). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Hammer

Date: September 5, 1997