

November 5, 2001

Mr. Otto L. Maynard  
President and Chief Executive Officer  
Wolf Creek Nuclear Operating Corporation  
Post Office Box 411  
Burlington, KS 66839

SUBJECT: FINAL CLOSEOUT OF RESPONSES TO GENERIC LETTER 96-06 FOR WOLF  
CREEK GENERATING STATION (TAC NO. M96887)

Dear Mr. Maynard:

The U. S. Nuclear Regulatory Commission (NRC) staff issued Generic Letter (GL) 96-06, "Assurance of Equipment Operability and Containment Integrity During Design-basis Accident Conditions," on September 30, 1996. In the GL, the NRC staff requested that you determine, for postulated accident conditions at Wolf Creek Generating Station (WCGS), if (1) containment air cooler cooling water systems are susceptible to either waterhammer or two-phase flow conditions, and (2) piping systems that penetrate containment are susceptible to thermal expansion of fluid so that overpressurization of piping could occur. You responded in letters of October 25, 1996 (ET 96-0083), January 29, 1997 (ET 97-0004), July 1, 1997 (ET 97-0059), September 28, 1998 (WM 98-0100), December 10, 1998 (ET 98-0099), June 29, 1999 (WM 99-0042), February 29, 2000 (ET 00-0010), and February 12, 2001 (ET 01-0006). Meetings were held with your staff on June 25 and September 24, 1997; and summaries of the meetings were issued July 22 and October 15, 1997, respectively. The staff's letter of July 9, 2001, closed out the first part of GL 96-06 that concerned waterhammer and two-phase flow.

This letter addresses the second part of the GL that concerns piping systems that penetrate containment and are susceptible to thermal expansion of fluid. You provided an assessment for WCGS in your letter of January 29, 1997, where you stated that 23 penetrations and 9 other piping segments were potentially vulnerable to a water solid volume that may be subjected to an increase in pressure due to heating of trapped fluid. In this submittal, you stated that an analysis was performed which indicated that all penetrations and other piping segments were operable, but that additional evaluations were needed to determine if additional corrective actions were necessary.

In the application dated December 10, 1998, you requested approval to use American Society of Mechanical Engineers (ASME) Code Case N-611, "Use of Stress Limits as an Alternative to Pressure Limits; Section III, Division 1, Subsection NC/ND-3500," to resolve the second part of GL 96-06. The Code Case deletes a requirement to limit the maximum fluid pressure in valves provided that the ASME Code stress limits are met. Because the staff approved the use of Code Case N-611 in its letter of March 4, 1999, and you stated in the December 10, 1998, submittal that the ASME Code allowable stresses in piping systems inside containment are not exceeded for any case under postulated design-basis accident conditions, the staff concludes that the necessary provisions of the ASME Code are met for the 23 penetrations and 9 other piping segments, and that the second part of the GL is acceptably resolved for WCGS.

This letter and the staff's letter of July 9, 2001, closes out the staff's review of your responses to GL 96-06. As stated in your letter of December 10, 1998, the analyses of the 23 penetrations and 9 other piping segments were completed; therefore, the implementation of the second part of GL 96-06 has been completed at WCGS. In the Safety Evaluation attached to the staff's July 9, 2001, letter it was stated that all necessary analyses and modifications for the first part of the GL had been completed by your letter of February 12, 2001. Therefore, all analyses and modifications for GL 96-06 have been completed for WCGS.

Sincerely,

*/RA/*

Jack Donohew, Senior Project Manager, Section 2  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-482

cc: See next page

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*/RA/*

Jack Donohew, Senior Project Manager, Section 2  
Project Directorate IV  
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Wolf Creek Generating Station

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