

March 18, 1999

Mr. H. L. Sumner, Jr.
Vice President - Nuclear
Hatch Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: CORRECTION TO AMENDMENT NO. 211 - EDWIN I. HATCH NUCLEAR
PLANT, UNIT 1 (TAC NO. M99073)

Dear Mr. Sumner:

On June 2, 1998, the Commission issued Amendment No. 211 to Facility Operating License No. DRP-57 for the Edwin I. Hatch Nuclear Plant, Unit 1, in response to your application dated May 30, 1997, as supplemented by letter dated April 1, 1998. The amendment revises the Technical Specifications (TS), requirements to reflect a design modification that changes the power sources to valves associated with the low pressure coolant injection mode of the residual heat removal system.

After issuance, it was discovered that TS page 3.5-4 was inadvertently issued with Amendment No. 211. TS page 3.5-4 did not reflect changes made by Amendment No. 204. Amendment No. 204 changed the location of SR 3.5.1.5 to TS page 3.5-3. Therefore, TS page 3.5-4 should not have been changed by Amendment No. 211. Enclosed is a corrected TS page TS 3.5-4 reflecting its original Amendment No. 204.

We regret any inconvenience this may have caused you.

Sincerely,
Original signed by:
Leonard N. Olshan, Senior Project Manager
Project Directorate II-2
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-321

Enclosure: TS 3.5-4

cc w/encl: See next page

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DOCUMENT NAME: G:\HATCH\99073COR.WPD

OFC	PM/PD2-2	LA/PD2-2	D/PD2-2
NAME	L. Olshan:cn	C. Hawes <i>CMH</i>	H. Berkow
DATE	3/18/99	3/18/99	3/18/99
COPY	YES/NO	YES/NO	YES/NO

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in dark ink, appearing to read "L. N. Olshan".

Leonard N. Olshan, Senior Project Manager
Project Directorate II-2
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-321

Enclosure: TS 3.5-4

cc w/encl: See next page

Edwin I. Hatch Nuclear Plant

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SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY												
SR 3.5.1.6	<p>-----NOTE----- Only required to be performed prior to entering MODE 2 from MODE 3 or 4, when in MODE 4 > 48 hours. -----</p> <p>Verify each recirculation pump discharge valve cycles through one complete cycle of full travel or is de-energized in the closed position.</p>	31 days												
SR 3.5.1.7	<p>Verify the following ECCS pumps develop the specified flow rate against a system head corresponding to the specified reactor pressure.</p> <table border="1"> <thead> <tr> <th>SYSTEM</th> <th>FLOW RATE</th> <th>NO. OF PUMPS</th> <th>SYSTEM HEAD CORRESPONDING TO A REACTOR PRESSURE OF</th> </tr> </thead> <tbody> <tr> <td>CS</td> <td>≥ 4250 gpm</td> <td>1</td> <td>≥ 113 psig</td> </tr> <tr> <td>LPCI</td> <td>≥ 17,000 gpm</td> <td>2</td> <td>≥ 20 psig</td> </tr> </tbody> </table>	SYSTEM	FLOW RATE	NO. OF PUMPS	SYSTEM HEAD CORRESPONDING TO A REACTOR PRESSURE OF	CS	≥ 4250 gpm	1	≥ 113 psig	LPCI	≥ 17,000 gpm	2	≥ 20 psig	In accordance with the Inservice Testing Program
SYSTEM	FLOW RATE	NO. OF PUMPS	SYSTEM HEAD CORRESPONDING TO A REACTOR PRESSURE OF											
CS	≥ 4250 gpm	1	≥ 113 psig											
LPCI	≥ 17,000 gpm	2	≥ 20 psig											
SR 3.5.1.8	<p>-----NOTE----- Not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. -----</p> <p>Verify, with reactor pressure ≤ 1058 psig and ≥ 920 psig, the HPCI pump can develop a flow rate ≥ 4250 gpm against a system head corresponding to reactor pressure.</p>	92 days												

(continued)

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