November 1, 1991

Docket Nos. 50-321
and 50-366

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Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT FOR EDWIN I. HATCH NUCLEAR PLANT - COMPLIANCE WITH GENERIC LETTER 88-01 (TAC NOS. M69138 AND M69139)

The Commission has issued the enclosed Amendment No. 176 to Facility Operating License DPR-57 and Amendment No. 117 to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated April 12, 1991.

The amendments change the TSs for Units 1 and 2 to comply with Generic Letter (GL) 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping," and NRC Safety Evaluation dated February 16, 1990.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 176 to DPR-57
2. Amendment No. 117 to NPF-5
3. Safety Evaluation

cc w/enclosures:
See next page

PDII-3:LA	PDII-3:PM	PDII-3:PM	EMCB:PM	OGC	D:PDII-3
LBerry	FRinaldi:cw	KJabbour	CYCheng		DMatthews
10/9/91	10/10/91	10/21/91	10/1/91	10/25/91	11/1/91

DOCUMENT NAME: INSERVICE INSPECTION 69138/9

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P PDR

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DATED: November 1, 1991

AMENDMENT NO. 176 TO FACILITY OPERATING LICENSE DPR-57 - Hatch Nuclear Plant, Unit 1
AMENDMENT NO. 117 TO FACILITY OPERATING LICENSE NPF-5 - Hatch Nuclear Plant, Unit 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 176
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees) dated April 12, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 176, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Eileen M McKenna for

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: November 1, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 176

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages
3.6-7	3.6-7
3.6-8	3.6-8
3.6-10	3.6-10

3.6.F.2.c. When the time limits or maximum conductivity or chloride concentration limits are exceeded, an orderly shutdown shall be initiated and the reactor shall be in the Cold Shutdown Condition within 24 hours.

G. Reactor Coolant Leakage*

1. Unidentified and Total

Any time irradiated fuel is in the reactor vessel and reactor coolant temperature is above 212°F:

a. reactor coolant system leakage into the primary containment from unidentified sources shall not exceed 5 gpm;

b. reactor coolant system leakage into the primary containment from unidentified sources shall not increase more than 2 gpm within a 24-hour period or less; and

c. the total reactor coolant system leakage into the primary containment shall not exceed 25 gpm within any 24-hour period.

when checked in accordance with 4.6.G.

2. Leakage Detection Systems

a. At least one of the leakage measurement instruments associated with each sump shall be operable and two of the other three leakage detection systems identified in Table 3.2-10, note c shall be operable when irradiated fuel is

4.6.F.2.c.3. Primary coolant pH shall be measured at least once every 8 hours whenever reactor coolant conductivity is > 2.0μmho/cm at 25°C.

d. Whenever the reactor is not pressurized, a sample of the reactor coolant shall be analyzed at least every 96 hours for chloride ion content and pH.

G. Reactor Coolant Leakage

Unidentified sources of reactor coolant system leakage shall be checked by the drywell floor drain sump system and recorded at least once per 8 hours. Identified sources of reactor coolant system leakage shall be checked by the equipment drain sump system and recorded at least once per 8 hours. The readings provided by the primary containment atmosphere particulate radioactivity monitoring system, the primary containment radioiodine monitoring system, and the primary containment gaseous radioactivity monitoring system shall also be recorded at least once per 8 hours.

*Not required during performance of an inservice hydrostatic or leakage test even if reactor coolant temperature is above 212°F.

G. Reactor Coolant Leakage

2. Leakage Detection Systems (Cont'd)

a. (Continued)

in the reactor vessel and reactor coolant temperature is above 212°F. Further, the primary containment atmosphere particulate radioactivity monitoring system shall be among these two operable systems, or samples shall be obtained and analyzed at least once each 8 hours.

b. From and after the date that any two of the four systems identified in Table 3.2-10, note c are made or found to be inoperable, but with the primary containment atmosphere particulate radioactivity monitoring system operable, reactor power operation may continue for the succeeding 30 days provided the primary containment atmosphere particulate radioactivity monitoring system reading is checked and recorded at least once each 8 hours.

c. From and after the date that any two of the four systems, including the primary containment atmosphere particulate radioactivity monitoring system, identified in Table 3.2-10, note c are made or found to be inoperable, reactor power operation may continue for the succeeding 30 days provided samples of the containment atmosphere are obtained and analyzed at least once each 8 hours.

3.6.K. STRUCTURAL INTEGRITY

1. Normal Condition

The structural integrity of ASME Code Class 1, 2, and 3 (equivalent) components shall be maintained in accordance with the Surveillance Requirements of Specification 4.6.K.

2. Off-Normal Conditions

a. With the structural integrity of any ASME Code Class 1 component not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDT considerations.

b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 212°F.

c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.

4.6.K. STRUCTURAL INTEGRITY

Surveillance Requirements for in-service inspection and testing of ASME Code Class 1, 2, and 3 (equivalent) components shall be applicable as follows:

1. In-service inspection of ASME Code Class 1, 2, and 3 (equivalent) components and in-serivce testing of ASME Code Class 1, 2, and 3 (equivalent) pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where specific relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g) (6) (i).

2. Performance of the above in-service inspection and testing activities shall be in addition to other specified Surveillance Requirements.

3. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

4. The Inservice Inspection Program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the staff positions on schedule, methods and personnel, and sample expansion included in the generic letter, except where specific written relief has been granted by the Commission.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 117
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees) dated April 12, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 117 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Eileen M McKenna for
David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: November 1, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 117

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages
3/4 0-2	3/4 0-2
3/4 0-3	3/4 0-3
3/4 4-5	3/4 4-5
3/4 4-6	3/4 4-6

SURVEILLANCE REQUIREMENTS (Continued)

b. A total maximum combined interval time for any 3 consecutive surveillance intervals not to exceed 3.25 times the specified surveillance interval.

4.0.3 Performance of a Surveillance Requirement within the specified time interval shall constitute compliance with OPERABILITY requirements for a Limiting Condition for Operation and associated ACTION statements unless otherwise required by the specification. Surveillance requirements do not have to be performed on inoperable equipment.

4.0.4 Entry into an OPERATIONAL CONDITION or other specified applicable state shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation have been performed within the applicable surveillance interval or as otherwise specified.

4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2, & 3 components shall be applicable as follows:

a. Inservice inspection of ASME Code Class 1, 2, and 3 components and inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section XI 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).

b. Surveillance intervals specified in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for the inservice inspection and testing activities required by the ASME Boiler and Pressure Vessel Code and applicable Addenda shall be applicable as follows in these Technical Specifications:

SURVEILLANCE REQUIREMENTS (Continued)

ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for inservice inspection and testing activities	Required frequencies for performing inservice inspection and testing activities
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Yearly or annually	At least once per 366 days

c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.

d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.

e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

f. The Inservice Inspection Program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the staff positions on schedule, methods and personnel, and sample expansion included in the generic letter, except where specific written relief has been granted by the Commission.

3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGE

LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.3.1 The following reactor coolant system leakage detection systems shall be OPERABLE:

a. The primary containment atmosphere particulate radioactivity monitoring system,

b. The primary containment floor drain and equipment sump level and flow monitoring systems, and

c. The primary containment gaseous radioactivity monitoring system.

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

a. With either the primary containment atmosphere particulate radioactivity monitoring system or the primary containment gaseous radioactivity monitoring system inoperable, operation may continue for 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 8 hours;

b. With at least one leakage monitoring instrument OPERABLE for both the primary containment floor drain sump and the equipment sump, operation may continue for 30 days;

c. Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.1 The leakage detection systems shall be demonstrated OPERABLE by:

a. Primary containment atmosphere gaseous and particulate monitoring system-performance of a CHANNEL CHECK at least once per 8 hours, a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.

b. Primary containment sump level and flow monitoring system-performance of a sensor check at least once per 8 hours, CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.3.2 Reactor coolant system leakage shall be limited to:

a. No PRESSURE BOUNDARY LEAKAGE,

b. 5 gpm UNIDENTIFIED LEAKAGE,

c. 25 gpm total leakage averaged over any 24-hour period, and

d. 2 gpm increase in UNIDENTIFIED LEAKAGE within a 24-hour period or less.

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.

b. With any reactor coolant system leakage greater than the limits specifically in b or c above, reduce the leakage rate to within the limits within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

c. With any reactor coolant system leakage greater than the limits specified in d above, reduce the leakage rate to within the limits within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.2 The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:

a. Monitoring the primary containment floor drain sump and equipment sump levels and flow rates at least once per 8 hours, and

b. Monitoring the primary containment atmospheric particulate and gaseous radioactivity at least once per 8 hours.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 176 TO FACILITY OPERATING LICENSE DPR-57

AND AMENDMENT NO. 117 TO FACILITY OPERATING LICENSE NPF-5

GEORGIA POWER COMPANY, ET AL.

EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-321 AND 50-366

1.0 INTRODUCTION

By letter dated April 12, 1991, the Georgia Power Company, et al. (the licensee), proposed changes to the Technical Specifications (TSs) for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The proposed changes incorporate NRC staff position on Inservice Inspection (ISI) in Generic Letter (GL) 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping," and the NRC Safety Evaluation (SER) of February 16, 1990. The first proposed change revises Unit 2 TSs 3.4.3.1, 3.4.3.2, 4.4.3.1, and 4.4.3.2 regarding the reactor coolant system (RCS) leakage monitoring. The second proposed change modifies Unit 1 TSs 3.6.G.1 and 3.6.G.2 regarding reactor coolant leakage. The last proposed change revises Unit 1 TS 4.6.K and Unit 2 TS 4.0.5 to state the ISI Program for piping, as addressed in GL 88-01, is in conformance with NRC staff position except where a specific written relief has been granted, and to delete ASME Section XI requirements related to the start of commercial operation.

2.0 DISCUSSION

NRC GL 88-01, issued January 5, 1988, provided guidance in the form of NRC positions regarding Intergranular Stress Corrosion Cracking (IGSCC) problems in Boiling Water Reactor (BWR) piping made of austenitic stainless steel that is four inches or larger in nominal diameter and contains reactor coolant at a temperature above 200 degrees Fahrenheit during reactor power operation regardless of ASME code classification. NRC GL 88-01 requested licensees of operating BWRs and holders of construction permits for BWRs to provide information regarding conformance with the NRC positions. One of the items which the GL requested licensees to address was a TS change to include a statement that the ISI program for piping covered by the scope of NRC GL 88-01 will be in conformance with the NRC positions on schedule, methods, personnel, and sample expansion included in this GL, or in accordance with alternate measures approved by the NRC staff.

3.0 EVALUATION

The NRC staff previously completed an evaluation of the licensee's ISI programs outlined in response to GL 88-01. By letter dated February 16, 1990, we informed the licensee that their programs were fully acceptable and satisfied all of the requirements in GL 88-01 except for the TS on the ISI program and identified leakage.

The licensee has proposed the following changes to the TSs in conformance with the guidance in GL 88-01 and the February 16, 1990 Safety Evaluation:

- Revise Unit 2 TS 3/4.4.3 to incorporate the requirements of an NRC Confirmatory Order dated July 8, 1983, and an NRC Safety Evaluation dated February 16, 1990. Specifically, TS 3.4.3.1 will now require, under certain conditions, that grab samples be obtained and analyzed every 8 hours in lieu of 4 hours. Similarly, in TSs 4.4.3.1.a and 4.4.3.1.b channel or sensor checks on the leakage detection systems will be performed at least every 8 hours. The same interval will apply for the surveillance requirements in TSs 4.4.3.2.a and 4.4.3.2.b. Also, this TS revision will require monitoring of floor and equipment sump levels and flow rates, primary containment atmospheric particulate and gaseous radioactivity at least once every 8 hours. Further, TS 3.4.3.2.d will state that conformance to the 2 gpm increase in RCS unidentified leakage will be applicable, "within a 24-hour period or less." These changes comply with NRC positions stated in the February 16, 1990 SER, and therefore, are acceptable.

- Revise Unit 1 TS 4.6.G applicable to reactor coolant leakage requirements and the surveillance frequency associated with the primary containment atmosphere particulate radioactivity monitoring system, the primary containment radioiodine monitoring system, and the primary containment gaseous radioactivity monitoring system. The change will state, "at least once per 8 hours," instead of "at least once per 4 hours," as currently specified. Also, TS 3.6.G.1.a no longer specifies averaging over a 24-hour period for the 5 gpm unidentified RCS leakage limit, and TS 3.6.G.1.b specifies that the limit for 2 gpm increase will not be exceeded "within a 24-hour period or less." Further, a phrase in TS 3.6.G.1.c was changed from, "a 24-hour period," to "any 24-hour period." Surveillance Requirements under 4.6.G will also be changed to reflect the 8 hours requirement. These changes follow the same requirements stated in the equivalent Unit 2 TSs, and therefore, are acceptable.

- Add TS Section 4.6.K.4 for Unit 1 and 4.0.5.f for Unit 2 to incorporate the Inservice Inspection Program for piping covered by GL 88-01. Also, delete Unit 2 TS 4.0.5.a.1 and renumber other TSs. This action removes obsolete ASME Section XI requirements applicable prior to the start of commercial operation of the facility.

The above changes to TSs 4.6.K.4 and 4.0.5.f are generally in accordance with the model TS and wording in GL 88-01 and NRC SER dated February 16, 1990. These changes are administrative in nature because they amend the TSs by adding statements which incorporate a reference to the existing NRC-approved Hatch ISI program. The existing Hatch ISI program ensures that surveillance is performed on piping susceptible to IGSCC in a manner that increases the probability of detecting any defects or flaws. The revision to Unit 2 TS 4.0.5.a.1 is considered administrative in nature, and therefore, acceptable to the NRC staff. The conclusions of the Hatch plant's accident analyses as documented in the FSAR or the NRC staff's SER are not altered by this change to the TSs. Therefore, the NRC staff has concluded that the TS changes satisfy the intent and objective of GL 88-01 and are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 29275). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Frank Rinaldi, PDII-3/DRPE

Date: November 1, 1991