Docket No.: 50-413

JAN 10 1986

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Enclosed for your information is a copy of a "Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing," related to your requests dated March 15, August 7, October 30, November 7, December 17, December 20, and December 23, 1985. This notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

/S/

B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A

Enclosure: As stated

cc w/enclosures: See next page

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-35, issued to Duke Power Company, et al., the licensee, for the operation of the Catawba Nuclear Station, Unit 1, located in York County, South Carolina.

In accordance with Duke Power Company's applications for amendment dated March 15, 1985, August 7, 1985, October 30, 1985, November 7, 1985, December 17, 1985, December 20, 1985 and December 23, 1985, the proposed changes would revise the Catawba Unit 1 Technical Specifications to eliminate typographical errors, provide additional clarification, improve consistency, adjust nomenclature, bring portions of the Specifications into conformance with current NRC staff positions, incorporate Unit 2 information where appropriate, and make other minor changes. The NRC staff renumbered many pages to accommodate the additional information needed for Unit 2. The items included in this notice do not cover all the items requested by Duke Power Company; some of the items not included in this notice are still under consideration by the NRC staff.

A primary objective of the amendment request is to achieve a single Technical Specifications document that is common for Units 1 and 2 with individual specifications for each Unit clearly identified, as appropriate.

The following is a description of the proposed changes to the current Unit 1 Technical Specifications:

1. Page V of the Table of Contents is revised to change page numbers for 2 figures that had been rearranged for a more consistent format.

2. Page VII of the Table of Contents is revised to indicate that a new section 3/4.4.11 has been added.

3. Page 2-2, Specification 2.1, Figure 2.1-1, "1700 psia" is changed to "1775 psia" to correct a typographical error.

4. Pages 2-5, 3/4 3-21, and 3-31, Specifications 2.2.1 and 3.3.2, Tables 2.2-1 and 3.3-3 are revised to incorporate Unit 2 trip setpoints.

5. Page 2-8, Specification 2.2.1, Table 2.2-1 is revised to read "590.8^{o}F (Nominal T_{avg} allowed by Safety Analysis)" for clarification of a parenthetical statement.

6. Page 2-8, Specification 2.2.1, Table 2.2-1 is revised to read "$q_t - q_b$ is more negative than -43%," and to read "$q_t - q_b$ is more positive than -6.5%" to clarify the nomenclature by replacing the word "exceeds."

7. A new page 3/4 0-2, Specification 3.0.5 is added as a clarification to specify the applicability of all specifications to Units 1 and 2 individually and jointly.

8. Pages 3/4 1-12, Specification 3.1.2.6 is revised to correct a typographical error in the specification number.

9. Page 3/4 2-7, Specification 4.2.2.2 is revised to replace "$F^L_{xy} = F^{RTP}_{xy}$ [1+0.3(1-P)]" with "$F^L_{xy} = F^{RTP}_{xy}$ [1+0.2(1-P)]" to correct a typographical error.

10. Pages 3/4 2-6 through 2-8 are rearranged and renumbered for a more consistent format.

11. Pages 3/4 2-10 and 2-11 are rearranged and renumbered for a more consistent format.

12. Pages 3/4 3-3, 3-8, and 3-10, Specifications 3.3.1 and 4.3.1.1, Tables 3.3-1, 3.3-2 and 4.3-1 are revised to incorporate Unit 2 specific values.

13. Pages 3/4 3-15, 3-23, 3-27, 3-34, 3-37, 3-38, 3-39, 3-42, 3-49, and 3-53, Specifications 3/4.3.2, and 3/4.3.3, Tables 3.3-3, 3.3-4, 3.3-5, 4.3-2, and 3.3-6 are revised to delete the word "ventilation" for a functional unit to correct and clarify the nomenclature of the functional unit.

14. Pages 3/4 3-18 and 3-19, Specification 3.3.2, Table 3.3-3 is revised to replace the word "loop" with "steam generator" to clarify the nomenclature for the actual system.

15. Page 3/4 3-21, Specification 3.3.2, Table 3.3-3 is revised to correct a typographical error by replacing an instrument number "5231" with "5232".

16. Page 3/4 3-26, Specification 3.3.2, Table 3.3-3 is revised to add the words "with flow through the HEPA filters and carbon adsorbers" at the end of ACTION 24 to correct an inconsistency with other specifications.

17. Page 3/4 3-29 through 3-32, Specification 3.3.2, Table 3.3-4 is revised to replace the symbol for "less than" with the symbol for "less than or equal to" to correct a typographical error and to incorporate Unit 2 specific trip setpoints.

18. Page 3/4 3-40, Specification 3.3.2, Table 3.3-5 is revised to delete "N/A" from line containing "13. Loss-of-Offsite Power" to correct a typographical error.

19. Page 3/4 3-41, Specification 3.3.2, Table 3.3-5 is revised to clarify that the KC valves apply to both Units 1 and 2 to correct an inconsistency for equipment designation.

20. Pages 3/4 3-52 and 3-54, Specification 3.3.3.1, Tables 3.3-6 and 4.3-3 are revised to replace "EMF-15" with "1EMF-15, 2EMF-4" to incorporate a Unit 2 design difference and to replace "All" with "1,2,3,4" under applicable modes for functional unit item 4 to correct an inconsistency with other specifications.

21. Page 3/4 3-53, Specification 3.3.3.1, Table 3.3-6 is revised to replace "charcoal" with "carbon" in ACTIONs 31, 34, and 35 to ensure the consistency of the nomenclature for adsorbers.

22. Pages 3/4 3-62 and 3-65, Specifications 3.3.3.5 and 3.3.3.6 are revised to replace "HOT SHUTDOWN within the next 12 hours" with "HOT STANDBY within the the next 6 hours and in HOT SHUTDOWN within the following 6 hours" to correct an inconsistency in the ACTION item with other specifications.

23. Pages 3/4 3-67 and 3-69, Specifications 3.3.3.6 and 4.3.3.6, Tables 3.3-10 and 4.3-7 are revised to replace "(EMF-26, 27, 28, or 29)" with "(1EMF-26, 27, 28, or 29 and 2EMF-10, 11, 12, or 13)" to incorporate Unit 2 design differences.

24. Page 3/4 3-70, Specification 3.3.3.7, is revised to replace "system" with "System" to correct a typographical error in reference to the Chlorine Detection System.

25. Page 3/4 3-71, Specification 4.3.3.8, is revised to replace "fire detection" with "smoke detection or flame detection," to replace the words "Fire detectors" with "Detectors," and to replace the words "fixed temperature/rate of rise" with "heat" to make the specification consistent with the correct nomenclature used in Table 3.3-11.

26. Pages 3/4 3-73 to 3-76, Specification 3.3.3.8, Table 3.3-11 is revised to add multiple fire detection instruments and locations for Unit 2 and to correct typographical errors.

27. Pages 3/4 3-79, Specification 3.3.3.10, Table 3.3-12 is revised to replace "1" with "1 per station" in 1.a., 2., 3.a., 3.b., and 3.c. to clarify applicability of instrumentation on a station basis.

28. Page 3/4 3-82, Specification 4.3.3.10, Table 4.3-8 is revised to correct a spelling error in "CHANNEL".

29. Page 3/4 3-84, Specification 3.3.3.11, Table 3.3-13 is revised to replace "1" with "1 per station" in 1.a. and 1.b., and to replace "1/train" with "1/train per station" in 2.a., and "2/train" with "2/train per station" in 2.b. to clarify the applicability of systems to both units.

30. Pages 3/4 3-77 to 3-84 have been renumbered to accommodate additional equipment for Unit 2 incorporated into pages 3/4 3-73 to 3-76.

31. Page 3/4 4-10, Specification 3.4.4, ACTION c. is revised to replace "With both PORV(s)" with "With more than one PORV" to clarify the action statement for a plant containing more than 2 PORVs.

32. Page 3/4 4-17, Specification 4.4.5.1, Table 4.4-1 has multiple revisions to clarify the information regarding Catawba as a plant with 4 steam generators.

33. Pages 3/4 4-22 and 4-23, Specification 3.4.6.2, Table 3.4-1 is revised to clarify the applicability of the valves to both units, and to correct typographical errors.

34. Page 3/4 4-40, a new Specification 3.4.11 is added to address Reactor Coolant System Vents.

35. Page 3/4 5-4, Specification 4.5.1.2 is revised to replace "...the water level is 93.2 [plus or minus] 2.7 inches above...." with "...the water level is 93.2 [plus or minus] 2.7 inches (Unit 1) and 93.1 [plus or minus] 2.7

inches (Unit 2) above..." to incorporate Unit 2 specifications.

36. Pages 3/4 5-6 and 5-7, Specification 4.5.2 is revised to replace "1NI" with "NI" to clarify the applicability of the valves to both units.

37. Page 3/4 6-7, Specification 3.6.1.2, Table 3.6-1 is revised to include a Unit 2 specific penetration and a footnote regarding its applicability to Unit 1.

38. Pages 3/4 6-14, 6-15, 7-13, 7-14, 7-15, 9-5, 9-6, 9-14, and 9-15, Specifications 4.6.1.8, 3.7.6, 4.7.6, 4.9.4.2, 4.9.11.1, and 4.9.11.2 are revised to replace the word "charcoal" with "carbon" to ensure the consistency of the nomenclature for adsorbers.

39. Page 3/4 6-17, Specification 4.6.1.9.4 is revised to replace "Containment Air Release and Addition System" with "Containment Air Release and Addition System valves" to correct a typographical omission.

40. Page 3/4 6-15, 7-14, 7-17, 9-6, and 9-15, Specifications 4.6.1.8, 4.7.6, 4.7.7, 4.9.4.2, and 4.9.11.2 are revised to delete "when tested in accordance with ANSI N510-1980" because this ANSI standard does not have test procedures for heat dissipators.

41. Page 3/4 7-1, Specification3.7.1.1 is revised to replace "COLD SHUTDOWN within the following 30 hours" with "HOT SHUTDOWN within the following 6 hours" to make the ACTION statement consistent with the APPLICABILITY statement for this Specification and the Standard Technical Specifications.

42. Page 3/4 7-3, Specification 3.7.1.1, Table 3.7-2 is revised to replace "1SV" with "SV" to clarify the applicability of the valves to both units.

43. Page 3/4 7-5, Specification 4.7.1.2.1 is revised to correct a typographical error by moving "and" from before paragraph 4) to before paragraph 5).

44. New page 3/4 7-9, Specification 3/4.7.1.3 is added to implement a Limiting Condition for Operation and Surveillance Requirements for the Unit 2 Condensate Storage Tank. Pages 7-9 through 7-41 are renumbered to reflect this addition, and Table of Contents Page IX is corrected to include the new Specification. This Specification does not apply to Unit 1; therefore, for Unit 1 noticing purposes, this change is purely administrative.

45. Page 3/4 7-12 and 7-13, Specifications 3.7.5 and 3.7.6 are revised to indicate that the specifications apply to both units.

46. Page 3/4 7-14, Specification 4.7.6e.3) is revised to clarify the relative pressure measurement requirement across the HEPA filter in the control room area ventilation system.

47. Page 3/4 7-27, Specification 4.7.10.1 is revised to correct a spelling error for "distribution".

48. Page 3/4 7-28, Specification 3.7.10.2 is revised to incorporate spray/sprinkler systems for Unit 2.

49. Pages 3/4 7-33 and 7-34, Specification 3.7.10.4, Table 3.7-3 has multiple revisions to incorporate Unit 2 additions, and to correct a typographical error.

50. Pages 3/4 8-12, 8-13, 8-14, 8-16, and 8-17, Specifications 3.8.2.1, 4.8.2.1.1, 3.8.2.2, and 3.8.3.1 are revised to delete reference "1" from Bus and Battery Numbers to clarify the applicability to both units.

51. Pages 3/4 8-19 and 8-21 through 8-43, Specification 3.8.4 and Table 3.8-1 have been revised and new pages 3/4 8-44 through 8-66 have been added to establish separate tables for Units 1 and 2 and to correct typographical errors.

52. Pages 3/4 8-24 and 8-26, Specification 3.8.4, Table 3.8-1 are revised to delete containment penetration conductor overcurrent protective devices to match the as-built plant.

53. Pages 3/4 8-35 and 8-43, Specification 3.8.4, Table 3.8-1 are revised to identify additional containment penetration conductor overcurrent protective devices originally omitted from the table.

54. Page 3/4 8-40, Specification 3.8.4, Table 3.8-1 is revised to correct a typographical error.

55. Page 3/4 11-2, Specification 4.11.1.1.1, Table 4.11-1 is revised to correct a typographical error by removing "a." under 2.

56. Page 3/4 11-18, Specification 3/4.11.3 is revised to correct a typographical error in the heading of the Specification.

57. Page 5-3, Specification 5.1.2, Figure 5.1-2 is revised to indicate that the Low Population Zone is 3.8 miles instead of 5.0 miles to conform with the Final Safety Analysis Report.

58. Page 6-1, Specification 6.2.2 is revised to include standard wording for a two unit station.

59. Pages 6-3 and 6-4, Specifications 6.2.1 and 6.2.2, Figures 6.2-1 and 6.2-2 have been revised to incorporate the current structures for offsite and unit organizations.

60. Page 6-5, Specification 6.2.2, Table 6.2-1 is revised to incorporate wording for a two unit station with a common control room.

61. Page 6-17, Specification 6.9.1.7 is revised to clarify classification of shipped solid waste as it pertains to 10 CFR Part 61.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendments would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated, or (2) create the possibility of a new or different kind of accident

from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

As stated above, the Commission proposes to determine that the proposed changes do not involve a significant hazards consideration. In this regard, the Commission has provided guidance concerning the application of standards for determining whether or not a significant hazards consideration exists by providing certain examples (48 FR 14870) of amendments considered not likely to involve significant hazards considerations. The examples include: (i) A purely administrative change to technical specifications; for example, a change to achieve consistency throughout the technical specifications, correction of an error, or a change in nomenclature; and (ii) A change that constitutes an additional limitation, restriction, or control not presently included in the technical specifications; for example, a more stringent surveillance requirement.

The proposed changes to the Technical Specifications are similar to these examples in that they are either administrative (i), or are more restrictive (ii). On this basis, the Commission proposes that these changes do not involve significant hazards considerations. The following is a description of how the proposed change items are similar to the examples of 48 FR 14870.

With the exception of item 34, all of the proposed changes in the Technical Specifications listed above are for eliminating typographical errors, correcting punctuation, adjusting nomenclature for consistency with usage at the plant, providing additional clarification, improving consistency within the specifications, making minor changes, and including Unit 2 Specifications as necessary

to reflect operation of Units 1 and 2. Therefore, they are encompassed by example (i) of actions not likely to involve significant hazards considerations.

Proposed change 34 introduces an additional Technical Specification to address reactor coolant system vents. Requirements for reactor coolant system vents are found in NUREG-0737 as post-TMI requirements. This Specification details the Limiting Condition for Operation and Surveillance Requirements for the vents. This proposed change is encompassed by example (ii) of 48 FR 14870 in that it provides additional restrictions and controls not presently included in the Technical Specifications. On this basis, the Commission proposes to determine that the change does not involve a significant hazards consideration since it incorporates the addition of restrictions and controls that are not currently included in the Technical Specifications.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for hearing.

Comments should be addressed to the Rules and Procedures Branch, Division of Rules and Records, Office of Administration, U. S. Nuclear Regulatory Commission, Washington, D.C. 20555.

By February 6, 1986, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interests may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Request for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to

the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment involves a significant hazards consideration, any hearing would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely fashion would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street NW, Washington, D.C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to B.J. Youngblood, Director, PWR Project Directorate #4, Division of PWR Licensing-A: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this

Federal Register notice. A copy of the petition should also be sent to the Executive Legal Director, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to Mr. William L. Porter, Duke Power Company, P.O. Box 33189, Charlotte, North Carolina 28242, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board, that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the applications for amendment dated March 15, 1985, August 7, 1985, October 30, 1985, November 7, 1985, December 17, 1985, December 20, 1985, and December 23, 1985, which are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, D.C., and at the York County Library, 138 East Black Street, Rock Hill, South Carolina 29730.

Dated at Bethesda, Maryland, this 30th day of December 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

B.J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A