June 27, 1996

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Mr. J. T. Beckham, Jr.
Vice President - Plant Hatch
Georgia Power Company
P. O. Box 1295
Birmingham, AL 35201

SUBJECT: ISSUANCE OF AMENDMENTS - EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2 (TAC NOS. M94863 AND M94864)

Dear Mr. Beckham:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 201 to Facility Operating License DPR-57 and Amendment No. 142 to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated February 21, 1996, as supplemented by letters dated May 1 and June 4, 1996.

The amendments revise the Drywell Air Temperature Limiting Condition for Operation (LCO) from less than or equal to 135 degrees F to less than or equal to 150 degrees F. The proposed change provides a margin for the primary containment Drywell Air Temperature LCO when prolonged summer and high river temperatures are experienced. Also, a strictly editorial correction to a Final Safety Analysis Report (FSAR) reference has been made.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Kahtan N. Jabbour, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

Enclosures: 1. Amendment No. 201 to DPR-57
2. Amendment No. 142 to NPF-5
3. Safety Evaluation

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cc w/encl: See next page

*SEE PREVIOUS CONCURRENCE

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 27, 1996

Mr. J. T. Beckham, Jr.
Vice President - Plant Hatch
Georgia Power Company
P. O. Box 1295
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2. Amendment No. 142 to NPF-5
3. Safety Evaluation

cc w/encl: See next page

cc:
Mr. Ernest L. Blake, Jr.
Shaw, Pittman, Potts and Trowbridge
2300 N Street, NW.
Washington, DC 20037

Mr. D. M. Crowe
Manager Licensing - Hatch
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Mr. L. Sumner
General Manager, Nuclear Plant
Georgia Power Company
11030 Hatch Parkway North
Baxley, Georgia 31513

Resident Inspector
U.S. Nuclear Regulatory Commission
11030 Hatch Parkway North
Baxley, Georgia 31513

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street, NW. Suite 2900
Atlanta, Georgia 30323

Mr. Charles H. Badger
Office of Planning and Budget
Room 610
270 Washington Street, SW.
Atlanta, Georgia 30334

Harold Reheis, Director
Department of Natural Resources
205 Butler Street, SE., Suite 1252
Atlanta, Georgia 30334

Mr. Thomas P. Mozingo
Program Manager
Nuclear Operations
Oglethorpe Power Corporation
2100 East Exchange Place
P. O. Box 1349
Tucker, Georgia 30085-1349

Charles A. Patrizia, Esquire
Paul, Hastings, Janofsky & Walker
10th Floor
1299 Pennsylvania Avenue
Washington, DC 20004-9500

Mr. Jack D. Woodard
Senior Vice President
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.201
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated February 21, 1996, as supplemented by letters dated May 1 and June 4, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 201 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Enviromental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: June 27, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 201

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the areas of change.

Remove Page	Insert Page
3.6-17	3.6-17

3.6 CONTAINMENT SYSTEMS

3.6.1.5 Drywell Air Temperature

LCO 3.6.1.5 Drywell average air temperature shall be ≤ 150°F.

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Drywell average air temperature not within limit.	A.1 Restore drywell average air temperature to within limit.	8 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3. AND B.2 Be in MODE 4.	12 hours 36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.1.5.1	Verify drywell average air temperature is within limit.	24 hours

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated February 21, 1996, as supplemented by letters dated May 1 and June 4, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 142 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: June 27, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 142

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the areas of change.

Remove Page	Insert Page
3.6-17	3.6-17

3.6 CONTAINMENT SYSTEMS

3.6.1.5 Drywell Air Temperature

LCO 3.6.1.5 Drywell average air temperature shall be ≤ 150°F.

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Drywell average air temperature not within limit.	A.1 Restore drywell average air temperature to within limit.	8 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3. AND B.2 Be in MODE 4.	12 hours 36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.1.5.1	Verify drywell average air temperature is within limit.	24 hours

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE DPR-57

AND AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NPF-5

GEORGIA POWER COMPANY, ET AL.

EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-321 AND 50-366

1.0 INTRODUCTION

By letter dated February 21, 1996, as supplemented by letters dated May 1, 1996 and June 4, 1996, Georgia Power Company, et al. (the licensee), proposed license amendments to change the Technical Specifications (TS) for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The proposed revision would change the Drywell Air Temperature Limiting Condition for Operation (LCO) from less than or equal to 135°F to less than or equal to 150°F. The proposed change would provide a margin for the primary containment Drywell Air Temperature LCO when prolonged summer and high river temperatures are experienced. Also, a strictly editorial correction to a Final Safety Analysis Report (FSAR) reference would be made. The reference to FSAR Section 6.2 would be changed to FSAR Section 5.2.3.2 for Hatch Unit 1. The May 1 and June 4, 1996, letters provided clarifying information that did not change the scope of the February 21, 1996, application and the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The licensee stated that, as part of the Plant Hatch Power Uprate Program, the pertinent design basis analyses for the containment were performed assuming an initial drywell average air temperature of 150°F, an initial drywell pressure of 1.75 psig, and a suppression pool temperature of 100°F.

Of the analyses that were performed, those that are affected by the proposed increase in drywell temperature include:

- short-term containment pressure/temperature response to the design basis loss-of-coolant accident (DBA-LOCA)
- DBA-LOCA containment dynamic loads
- long-term containment pressure/temperature response to DBA-LOCA
- drywell pressure/temperature response to small steamline breaks

The short-term LOCA response evaluation utilized the M3CPT computer code. The long-term LOCA response and the small steamline break evaluations utilized the SHEX computer code. The results of the licensee's analyses are:

(1) Operation with the drywell air temperature less than or equal to 150°F will not result in any safety concerns associated with primary containment system performance.

(2) Peak drywell pressures will remain below design drywell pressures, and drywell structure temperatures will remain below design temperatures.

(3) For Unit 2, the peak ambient drywell air temperature is below the drywell structure design temperature of 340°F.

(4) For Unit 1, the peak ambient drywell air temperature is slightly above the drywell structure design temperature of 281°F during the initial 15 seconds of the limiting accident. An evaluation concluded that the actual drywell structure design temperature is not exceeded. This condition was previously approved by the NRC staff during its review of the power uprate submittal per the staff's safety evaluation Section 3.7.1.2 dated August 31, 1996.

The licensee's evaluation of the effects of increased initial drywell temperature on equipment qualification found that drywell temperature will increase by only 1°F during the postulated worst-case scenario (i.e., small steamline break), and that the resultant drywell temperature remains below the existing equipment qualification temperature envelopes for Hatch Units 1 and 2. The licensee will continue to monitor ambient drywell temperatures and the qualified lifetimes of components will be adjusted as necessary based on the actual temperatures that exist. The licensee's evaluation also concluded that the proposed increase in drywell temperature will not have an adverse effect on the primary containment structure and pipe supports.

The licensee evaluated reactor water level instrument calibration assuming drywell temperatures up to 170°F. The results showed that a change in calibration endpoints from 135°F to 170°F had a negligible effect upon setpoint available margins. This temperature bounds the expected actual temperature in the vicinity of the instrument sensing lines, assuming the drywell average allowable temperature is less than 150°F. The licensee concluded that since the instrument calibration impact is negligible, no changes in instrument setpoints are required.

The NRC staff has reviewed the information submitted by the licensee in support of the proposed increase in drywell temperature for Hatch Units 1 and 2, as discussed above. The licensee has evaluated the effects of increased drywell temperature on the containment and equipment located in the drywell, and has found that the effects are insignificant and of no consequence to public health and safety. Based on the information that has been provided, the NRC staff agrees with the licensee's assessment and the use of computer codes SHEX and M3CPT for the above purpose . Therefore, the proposed increase in drywell temperature is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 18167 dated April 24, 1996). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: J. Tatum
R. Goel
K. Jabbour

Date: June 27, 1996