September 26, 1994

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Mr. J. T. Beckham, Jr.
Vice President - Plant Hatch
Georgia Power Company
P. O. Box 1295
Birmingham, AL 35201

SUBJECT: ISSUANCE OF AMENDMENT - EDWIN I. HATCH NUCLEAR PLANT, UNIT 1 (TAC NO. M90214)

Dear Mr. Beckham:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 194 to Facility Operating License DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit 1. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated August 16, 1994, as supplemented September 20, 1994.

The amendment would make a one-time change to TS 3.9.C for Hatch Unit 1 regarding the emergency diesel generator (DG) operability requirements during reactor shutdown conditions. Current TS 3.9.C requires that two DGs be operable during reactor shutdown when a core or containment cooling system is required to be operable. The proposed amendment would revise the current requirement such that only one emergency DG is required to be aligned to its associated core or containment cooling system during a specific time of the outage. During this time period, the decay heat removal (DHR) system will be in service. The DHR system, which is completely independent of the existing shutdown cooling system, is powered by the Baxley substation and has its own DG as a backup power supply.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-321

Enclosures:
1. Amendment No. 194 to DPR-57
2. Safety Evaluation

cc w/encl: See next page

*See previous concurrence

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DATE	9/26/94		9/26/94		09/19/94		9/27/94		09/22/94	

DOCUMENT NAME: G:\HATCH\HAT90214.AMD

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 26, 1994

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Vice President - Plant Hatch
Georgia Power Company
P. O. Box 1295
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Project Directorate II-3
Division of Reactor Projects - I/II
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Docket No. 50-321

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1. Amendment No. 194 to DPR-57
2. Safety Evaluation

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 194
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for an amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated August 16, 1994, as supplemented September 20, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by a page change to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 194 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: September 26, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 194

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Remove Page	Insert Page
3.9-6a	3.9-6a

C. Diesel Generator Requirements (Reactor in the Shutdown or Refuel Mode) (Continued)

1. Work is being done which has the potential for draining the reactor pressure vessel, or

2. Secondary containment is required, or

3. A core or containment cooling system is required.*

D. Electric Power Monitoring for the Reactor Protection System

Specifications:

1. When either of the RPS MG sets or the Alternate Source is in service, its power monitoring system shall be OPERABLE.

   (a) If the power monitoring system is not OPERABLE and Operability cannot be restored within 30 minutes of discovery, remove the power supply from service immediately thereafter.

   (b) One channel of a power monitoring system may be inoperable, as necessary for test or maintenance, not to exceed 8 hours per month.

4.9.D. Electric Power Monitoring for Reactor Protection System

Specifications:

1. The Electric Power Monitoring for the Reactor Protection System shall be demonstrated operable:

   (a) At least one per 6 months by performing a FUNCTIONAL TEST,

   (b) At least once per operating cycle by demonstrating the OPERABILITY of under-voltage, over-voltage and under-frequency protective instrumentation by performance of a CHANNEL CALIBRATION including simulated automatic actuation of the protective relays, tripping logic and output circuit breakers and verifying the following setpoints:

      (1) Over-voltage $\leq$132 VAC, with time delay relay set to 4 seconds maximum,

      (2) Under-voltage $\geq$108 VAC, with time delay relay set to 4 seconds maximum,

      (3) Under-frequency $\geq$57 Hz, with time delay relay set to 4 seconds maximum.

*For the Fall 1994 Unit 1 refueling/maintenance outage, two diesels must be operable per Specification 3.9.C. However, Specification 3.9.C.3 is not applicable when local leak rate testing is being performed on the Residual Heat Removal System loops and diesel generator 1A is removed from service for maintenance, provided the fuel pool gates are removed and the cavity is flooded. During this time, one of the two required diesels must be aligned to its corresponding core or containment cooling system.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 194 TO FACILITY OPERATING LICENSE DPR-57

GEORGIA POWER COMPANY, ET AL.

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-321

1.0 INTRODUCTION

By letter dated August 16, 1994, as supplemented September 20, 1994, Georgia Power Company, et al. (GPC or the licensee), proposed an amendment which would make a one-time change to Technical Specification (TS) 3.9.C for Hatch Unit 1 regarding the emergency diesel generator (DG) operability requirements during reactor shutdown conditions. The current TS 3.9.C requires that two DGs be operable during the reactor shutdown mode when a core or containment cooling system is required to be operable. The proposed amendment would revise the current requirements such that only one emergency DG is required to be aligned to its associated core or containment cooling system during a specific time of the outage. During this time period the decay heat removal (DHR) system will be in service. The DHR system, which is completely independent of the existing shutdown cooling system, is powered by the Baxley substation and has its own DG as a backup power supply. The September 20, 1994, letter provided additional information that did not change the scope of the August 16, 1994, application and the initial proposed no significant hazards consideration determination.

The licensee has requested this one-time change for the fall 1994 Unit 1 refueling/maintenance outage to perform, among other things, local leak rate testing on the residual heat removal system loops and maintenance on DG 1A. During the period that only one DG is operable, the reactor cavity will be flooded and the fuel pool gates will be removed.

2.0 EVALUATION

The DHR system is designed with a primary loop and a secondary loop. The primary loop consists of two pumps, two heat exchangers, and a strainer. These components are installed one elevation below the refueling floor in the reactor building. On the refueling floor, pipe spools allow the system to be aligned to either the Unit 1 or Unit 2 spent fuel pool, with suction from and discharge to the pools. The secondary loop consists of two cooling towers and two pumps located on the railroad airlock roof. Power is supplied from the Baxley, Georgia, substation. Furthermore, the DHR system has a dedicated nonsafety-related diesel to supply backup power, if necessary.

NRC Inspection Report 94-08, dated May 12, 1994, stated that during the Spring 1994 Unit 2 outage, the DHR system successfully demonstrated the ability to simultaneously provide adequate decay heat removal for the spent fuel pool and the reactor with no other decay heat removal system, other than reactor water cleanup, in service.

Early in the outage (i.e., from day 2 to day 5), the DHR system provides decay heat removal for the spent fuel pool. It provides core and the spent fuel pool cooling as soon as the gates between the reactor pressure vessel (RPV) and the fuel pool are removed (i.e., about the fifth day until the end of the outage). For the first 5 days, residual heat removal (RHR) loop B (which includes RHR pump B and its associated diesel generator 1C, and pump D and its associated DG 1B) provides core cooling. However, from the fifth day on, RHR loop B would be taken out of the shutdown cooling mode but will remain available and operable (with pump D and its associated DG 1B operable). Also, local leak rate testing (LLRT) will commence on RHR loop A, rendering it unavailable for shutdown cooling. From the third day until day 10, DG 1A would be taken out of service for maintenance. The LLRT will require approximately 44 hours to complete. Subsequent to the test, RHR loop A (with only pump C and its associated DG 1B) will then become available for shutdown cooling. The licensee expects that the maximum time period that this change is needed would likely be about 3 to 5 days.

In addition, the licensee stated that they plan to replace up to 2 RHR check valves, 1E11-F050A and/or B, depending on the results of the LLRTs. Replacement will take approximately 13 days to complete, and will be performed after the core is offloaded and before it is reloaded.

Under the above proposed configuration, a loss-of-offsite power (LOOP), coupled with a failure of DG 1B, will render the RHR shutdown cooling system inoperable. However, the DHR system will remain available, with power being supplied from either its normal supply or the backup diesel. Furthermore, the reactor cavity is flooded and the gates between the RPV and the spent fuel pool are open.

Based on the above evaluation, the NRC staff finds that this one-time revision has no adverse impact on safety, and does not pose an undue risk to public health and safety. Therefore, it is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 44196, August 26, 1994). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Kahtan N. Jabbour, NRR

Date: September 26, 1994