

# **FINAL SUBMITTAL**

**BROWNS FERRY EXAM  
50-259, 260, 296/2001-301**

**SEPTEMBER 17-21, 2001**

**FINAL AS GIVEN  
OPERATOR ACTIONS**

**F.1.g - FORM ES-D-2  
OPERATOR ACTIONS**

Facility: Browns Ferry

Scenario No.: NRC-1

Op-Test No.: A

Examiners: \_\_\_\_\_ Operators: \_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

Initial Conditions: Unit At 85%, With Power Ascension in progress. 2C RHR Pump is tagged out for minor maintenance (6 hours into a 7 day LCO). HPCI Flow Test at Rated Pressure, 2-SR-3.5.1.7 is in progress and complete up to Step 7.10.

Turnover: Continue to 100%. Severe Thunder Storm Warning in effect.

Event No.	Malf. No.	Event Type*	Event Description
1	N/A	N-BUO	Alternate Unit 2 EH pumps.
2	N/A	R-all	Continue power ascension to 100%.
3	Imfhp08	N-duo C-duo	Continue HPCI Flow Test 2-SR-3.5.1.7 (Ruptured HPCI steam line with a failure to auto isolate).
4	imf swo2a	I-duo	"A" RBCCW pump trip and FCV-70-48 failure to close.
5	imf ad01n 100	C-buo	SRV-1-180 Fails Open.
6	Bat RRPAVIB dmf th12A	C-buo /all	High Vibrations result in 2A Recirc Pump Seal Leak, and Power Oscillations that will Lead to Reactor Scram.
7	imf th22	M-all	High Pressure in Drywell.
7b	imf edo1	M-all	LOOP/ After entry into procedure for High Pressure in DW, with a failure of the "D" D/G to autostart.

\* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

## SIMULATOR EVALUATION GUIDE

TITLE : NRC-1  
REVISION : 0  
DATE : August 30, 2001  
PROGRAM : BFN Operator Training - Hot License

PREPARED BY: \_\_\_\_\_ / \_\_\_\_\_  
(Operations Instructor) Date

REVIEWED BY: \_\_\_\_\_ / \_\_\_\_\_  
(LOR Lead Instructor or Designee) Date

REVIEWED BY: \_\_\_\_\_ / \_\_\_\_\_  
(Operations Training Manager or Designee) Date

CONCURRED : \_\_\_\_\_ / \_\_\_\_\_  
(Operations Superintendent or Designee) Date

VALIDATION : \_\_\_\_\_ / \_\_\_\_\_  
BY (Operations SRO: Required for Exam Scenarios Only) Date

TASK LIST : N/A \_\_\_\_\_ / \_\_\_\_\_  
UPDATED (Operations Training) Date

LOGGED IN : N/A \_\_\_\_\_ / \_\_\_\_\_  
(Librarian) Date

NUCLEAR TRAINING  
REVISION/USAGE LOG

REVISION NUMBER	DESCRIPTION OF REVISION	DATE	PAGES AFFECTED	REVIEWED BY
0	INITIAL	8/30/01	All	

I. Program: BFN Operator Training

II. Course: Hot License Training

III. Title: NRC 1

IV. Length of Scenario:

V. Examination Objectives:

A. Terminal Objective

1. Perform routine shift turnover, plant assessment and routine shift operation in accordance with BFN procedures.
2. Given abnormal conditions, the operating crew will place the unit in a stabilized condition per normal, abnormal, annunciator and emergency procedures.

B. Enabling Objectives:

1. The operating crew will alternate EHC pumps.
2. The operating crew will continue power ascension from 85% power.
3. The operating crew will experience a HPCI steam line break during performance of 2-SR-3.5.1.7, HPCI Flow Rate, with a failure of HPCI to auto isolate.
4. The operating crew will recognize and respond to a loss of an RBCCW pump and failure of FCV-70-48 to automatically close.
5. The operating crew will recognize and respond to a safety-relief valve failed open.
6. The operating crew will recognize and respond to a high vibration and trip of 2A Recirc pump.
7. The operating crew will recognize and respond to reactor power oscillations by scrambling the reactor.
8. The operating crew will recognize and respond to a high drywell pressure condition.
9. The operating crew will recognize and respond to a loss of all offsite power.
10. The operating crew will respond to a failure of 'D' diesel generator to auto start.

VI. References:

The procedures used in the simulator are controlled copies and are used in development and performance of simulator scenarios. Scenarios are validated prior to use, and any procedure differences will be corrected using the procedure revision level present in the simulator. Any procedure differences noted during presentation will be corrected in the same manner. As such, it is expected that the references listed in this section need only contain the reference material which is not available in the simulator.

VII. Training Materials:

- A. This Lesson Plan (NRC 1)
- B. Simulator
- C. Control Rod Insertion Sheets
- D. Marked up copy of 2-SR-3.5.1.7, HPCI Flow Rate

# VIII. Console Operators Instructions

## A. Scenario File Summary

- |                |  |  |
|----------------|--|--|
| 1. IC 39       | 85% MOC<br>bat 7048ftc<br>bat 7048-1<br><br>bat torhrc<br>imf hp09<br>imf dg01d                        | Fails FCV-70-48 Open<br>Allows FCV-70-48 to be closed manually<br>Prepares RHR C for tagout<br>Failure of HPCI to auto isolate<br>Failure of D diesel gen to auto start        |
| 2. bat app16fg |  | Defeats RHR Injection Valve Timers   |
| 3. bat RRPVIB  | imf th12a<br>imf th10a (none 1: )<br>imf th11a (none 2: )<br>imf cr02a (e5 20) 25 2:00<br>ior zdihs681 | Inserts Vibration Alarm<br>Fails Recirc Pump A Inboard Seal<br>Fails Recirc Pump A Outboard Seal<br>Inserts Power Oscillations<br>Prevents Recirc Pump A Suction Valve Closure |

## B. Console Operators Manipulations

<u>ELAP. TIME</u>	<u>IC/MF/RF#</u>	<u>DESCRIPTION/ACTION</u>
Sim setup	rst 39	85% Power MOC
Sim setup	manual	Hang HO tags on 'C' RHR pump
Sim setup	manual	Place TESTING/MAINT frames on Panel 9-3F, Windows 5, 11, 26 for HPCI SR
When HPCI is at rated pressure and flow	imf hp08	Steam leak into HPCI room
When directed by Lead Examiner	imf sw02a	Trips 'A' RBCCW pump
Unit 1 Operator 1 minutes after requested	mrf sw03	Places spare RBCCW pump in service
When directed by Lead Examiner	imf ad01n 100	Fails SRV-1-180 open
When dispatched, wait four minutes	mrf ad02n out	Removes power from SRV-1-180

**More Follows**

B. Console Operators Manipulations (Continued)

**When dispatched to check 2A Recirc Vibration, wait 3 minutes and report back 15 mils**

When directed by Lead Examiner	bat RRPAVIB	Recirc Pump A high vibration, seal failure, suction valve fails to close and power oscillations.
When 'A' Recirc trips	dmf th12a	Deletes vibration high alarm
After scram	imf th22 (none :30) 100	60 GPM seal leakage
When requested, wait 4 minutes	bat app16fg	Defeats RHR injection valve timers
When directed by Lead Examiner	imf ed01	Loss of offsite power
When requested, wait 4 minutes	bat eecw bat eecw-1 mrf rp01 mrf rp02 mrf ia05a mrf ia05d	Resets EECW to CAC and RBCCW Returns EECW to auto Resets A RPS Circuit Protectors Resets B RPS Circuit Protectors Resets A CAC Resets D CAC



IX. Scenario Summary

Given Unit 2 at 85% power, the crew will alternate EHC pumps and resume power ascension to 100%. As 2-SR-3.5.1.7, HPCI Flow Rate, is continued the crew will experience a ruptured HPCI steam line with a failure of HPCI to automatically isolate. Manual HPCI isolation will be possible. Subsequently, an RBCCW pump trips with the resultant failure of the 70-48 valve to close. When the 70-48 valve is manually closed it isolates non-essential RBCCW loads including the reactor water cleanup system. As power ascension is continued, an SRV fails open but can be closed as steps of AOI-1-1 are performed. Finally the crew experiences high vibration with a subsequent trip and seal leakage on the 2A Recirc Pump resulting in high drywell pressure. When the diesel generators automatically start the D diesel generator fails to auto start but can be manually started.

X. Information to Floor Examiners:

- A. Ensure recorders are inking and recording and ICS is active and updating.
- B. Assign Crew Positions based on the required rotation.
  - 1. SRO: Unit Supervisor
  - 2. BUO: Board Unit Operator
  - 3. DUO: Desk Unit Operator
- C. Conduct a shift turnover with the Unit Supervisor.
- D. Direct the shift crew to review the control board and take note of present conditions, alarms, etc.
- E. Terminate the scenario when the following conditions are satisfied are at the request of the floor/lead instructor/evaluator.
  - 1. RPV water level +2" to +51"
  - 2. Drywell pressure under control
  - 3. Critical systems restored from loss of offsite power

XI. Simulator Event Guide

Event 1: Alternate EHC Pumps

<u>POSITION</u>	<u>EXPECTED ACTION(S)</u>	<u>NOTES/REMARKS</u>
BUO/DUO	Receive crew briefing and walk boards down	_____
SRO	Directs DUO to alternate EHC pumps	_____
DUO	Alternates EHC Pumps in accordance with 2-OI-47A	_____
	<ul style="list-style-type: none"> <li>• Starts 2B EHC Pump</li> </ul>	_____
	<ul style="list-style-type: none"> <li>• Verifies EHC header pressure 1550 to 1650 psig</li> </ul>	_____
	<ul style="list-style-type: none"> <li>•</li> </ul>	_____
	<ul style="list-style-type: none"> <li>• Verifies 2B EHC motor amps &lt;140</li> </ul>	_____
	<ul style="list-style-type: none"> <li>•</li> </ul>	_____
	<ul style="list-style-type: none"> <li>• Stops 2A EHC Pumps</li> </ul>	_____

XI. Simulator Event Guide (Continued)

Event 2: Continue power ascension

<u>POSITION</u>	<u>EXPECTED ACTION(S)</u>	<u>NOTES/REMARKS</u>
SRO	Directs power ascension per GOI-100-12 and OI-68	_____
BUO	Raises reactor power at 8 to 10 MW/min in accordance with GOI-100-12 and OI-68	_____
DUO	Performs as peer checker for recirc flow changes	_____

XI. Simulator Event Guide (Continued)

Event 3: HPCI Steam Line Break

<u>POSITION</u>	<u>EXPECTED ACTION(S)</u>	<u>NOTES/REMARKS</u>
SRO	Directs DUO to continue with 2-SR-3.5.1.7 at step 7.11	_____
BUO/DUO	Makes plant announcement HPCI is to be started	_____
DUO	Responds to Reactor Bldg Hi Rad alarm per the ARP	_____
DUO	Determines HPCI area source of hi rad	_____
DUO	Responds to HPCI Leak Detection Temp Hi alarm per the ARP	_____
DUO	Recognizes HPCI not isolated when isolation lights are illuminated	_____
SRO	Directs HPCI manually isolated	_____
DUO	Manually isolates HPCI steam supply	_____
BUO/DUO	Evacuates HPCI area	_____
BUO/DUO	Notifies Rad Con	_____
DUO	Monitors for lowering temperature and radiation levels in HPCI area	_____
SRO	Sends personnel to investigate	_____
SRO	Determines unit in 72 hour LCO (TS 3.5.1 - HPCI and C RHR Inop)	_____

XI. Simulator Event Guide (Continued)

Event 4: RBCCW Pump Trip and FCV-70-48 Failure to Close

<u>POSITION</u>	<u>EXPECTED ACTION(S)</u>	<u>NOTES/REMARKS</u>
CREW	Recognizes 'A' RBCCW pump trip	_____
SRO	Directs response per 2-AOI-70-1	_____
DUO	Attempts manual restart of 'A' RBCCW Pump	_____
DUO/BUO	Directs Unit 1 Operator to place spare RBCCW pump in service to Unit 2	_____
DUO	Determines FCV-70-48 did not close	_____
DUO	Manually closes FCV-70-48	_____
DUO	Verifies all available drywell cooling in service	_____
DUO/BUO	Secures Reactor Water Cleanup System	_____
CREW	Monitors drywell and recirc pump temperatures	_____

XI. Simulator Event Guide (Continued)

Event 5: SRV-1-180 Fails Open

<u>POSITION</u>	<u>EXPECTED ACTION(S)</u>	<u>NOTES/REMARKS</u>
CREW	Recognizes SRV open <ul style="list-style-type: none"> <li>• Main Steam Relief Valve Open alarm</li> <li>• lowering generator output</li> </ul>	
SRO	Directs response per AOI-1-1	
DUO	Determines SRV-1-180 from accoustic monitor	
DUO	Places SRV-1-180 control switch from close to open to close several times	
DUO	Determines SRV still open	
DUO	Places SRV Tailpipe Flow Monitor power switch to OFF and then back ON	
DUO	Determines SRV still open	
DUO	Places MSRV Auto Actuation Logic Inhibit, 2-XS-1-202, in INHIBIT	
DUO	Determines SRV has closed	
SRO	Directs power removed from SRV-1-180	
DUO	Dispatches personnel to remove power from SRV-1-180	
DUO	After power removed from SRV-1-180, returns MSRV Auto Actuation Logic Inhibit, 2-XS-1-202, to AUTO	
SRO	Determines TS 3.4.3 - Non-ADS valve, safety function of 12 required, No further LCO	

XI. Simulator Event Guide (Continued)

Event 6: Recirc Vibration, Seal Leakage, Power Oscillations and Scram

<u>POSITION</u>	<u>EXPECTED ACTIONS</u>	<u>COMMENTS</u>
BUO/DUO	Announces Recirc A high vibration alarm	_____
BUO/DUO	Consults ARP for Panel 9-4	_____
BUO/DUO	Directs AUO to Local Panel to check vibration	_____
DUO	Monitors Recirc Pump Temperatures	_____
SRO	Contacts Reactor Engineer	_____
SRO	Directs BUO to reduce speed of 2A RRP to reduce vibration	_____
BUO	Reduces 2A RRP speed with peer check to clear vibration alarm	_____
BUO/DUO	Announces Recirc A Seal Leakage Alarm	_____
BUO/DUO	Identifies Seal Failure via Instrumentation	_____
BUO/DUO	Recognizes lowering pressure on Recirc Pump A #1 seal	_____
SRO	Directs crew to watch for signs of increased leakage	_____
DUO	Acknowledges Recirc Pump A seal leakoff high alarm; informs SRO; consults ARP	_____
BUO/DUO	Recognizes lowering pressure on Recirc Pump A outboard seal; informs SRO	_____
DUO	Monitors drywell parameters; notes pressure and temperature increasing; informs SRO	_____
SRO	When vibration report received or dual seal failure is reported, directs 'A' Recirc Pump tripped	_____
BUO/DUO	Trips Recirc A and closes the discharge valve	_____
SRO	Directs actions per 2-AOI-68-1	_____



XI. Simulator Event Guide (Continued)

Event 6: Recirc Vibration, Seal Leakage, Power Oscillations and Scram (Continued)

<u>POSITION</u>	<u>EXPECTED ACTIONS</u>	<u>COMMENTS</u>
BUO/DUO	Directs AUO to Recirc MG Set to monitor oil temp.	_____
BUO/DUO	Checks Power to flow map to verify within safe region	_____
BUO	Checks APRMs and LPRMs for indication of power oscillations	_____
BUO	Informs SRO of Power Oscillations	_____
SRO	When APRM oscillation >10% peak to peak, directs reactor scram	_____
BUO	Scrams the reactor	_____
SRO	Directs AOI-100-1	_____
BUO/DUO	Carry out actions of AOI-100-1	_____
SRO	Directs 'A' Recirc Isolated	_____
DUO	Notes that Recirc Pump A suction isolation valve will not close; informs SRO	_____
SRO	Directs DUO to monitor drywell temperature and pressure	_____
DUO	Directs AUO to close Recirc Pump suction valve locally at Board.	_____
SRO	Directs venting per OI-64	_____
DUO	Vents per OI-64	_____
DUO	Directs Logs person to monitor release rates	_____
DUO	Keeps SRO informed as drywell pressure approaches 2.45 psig	_____
SRO	Enters EOI-2 at 2.45 psig drywell pressure	_____
BUO/DUO	Directs venting per Appendix 12	_____

XI. Simulator Event Guide (Continued)

Event 7: LOSS OF OFFSITE POWER

<u>POSITION</u>	<u>EXPECTED ACTIONS</u>	<u>INST. INFO/NOTES</u>
BUO/DUO	Recognize loss of offsite power; informs SRO	_____
SRO	Directs BOP to verify all 4 diesel generators started and tied on (0-AOI- 57-1A immediate action)	_____
DUO	Recognizes that DG D did not auto start	_____
DUO	Manually starts DG D	_____
DUO	Informs DRO DG D was manually started and tied to board	Critical Task _____
SRO	Directs actions per 0-AOI-57-1A:	_____
DUO	Carries out actions of 0-AOI-57-1A	_____
	Reset EECW Valves	_____
	Start RBCCW pumps and drywell cooling	_____
	Start diesel driven fire pump [Unit 1 operator]	_____
	Realign electrical distribution system	_____
	Reset RPS MG sets	_____
	Reset A, D & G Air Compressors.	_____
DUO	Starts drywell coolers	_____
BUO	Starts RBCCW pumps	_____
SRO	Directs RPS busses be restored	_____
SRO	Directs 1B CRD pump started	_____
BUO	Starts 1B CRD pump and aligns to Unit 2	_____

XIV. Shift Turnover Information

Equipment out of service/LCOs: 2C RHR Pump is out of service. T.S 3.5.1.A.1 has been  
entered. Unit 2 is 6 hours into a seven day LCO

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Operation/Maintenance for the Shift: Unit 2 is at 85% power, 2-SR-3.5.1.7 in progress  
Complete up to Step 7.11 (HPCI Main and Booster Pump Set Developed Head and Flow Rate  
Test at Rated Reactor Pressure). Swap EHC Pumps per section 6.3 of OI47A. Increase reactor  
power to 90% using Recirc flow (GOI-100-12 step 5.132) and complete HPCI SR.

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Unusual Conditions/Problem Areas: None

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Facility: Browns Ferry	Scenario No.: NRC 2	Op-Test No.: A	
Examiners: _____		Operators: _____	
_____		_____	
_____		_____	
<p>Initial Conditions: Unit At 100%; 2C RHR Pump is tagged out for minor maintenance (6 hours into a 7 day LCO)</p> <p>Turnover: Power reduction Planned in order to perform 2-SR-3.3.1.1.8(9) turbine control valve fast closure. Severe Thunder Storm Warning in effect.</p>			
Event No.	Mal. No.	Event Type*	Event Description
0	imf rp06	C all	Prevent Reactor Scram
0	bat atws75	M	75% ATWS
1	N/A	R buo	Power Reduction for 2-SR-3.3.1.1.8(9)
2	imf rp01a	I buo	Failure of "A" RPS.
3	Imf og04a	C duo	Failure of "A" SJAE.
4	Imf rd01a	C buo	2A CRD Pump Trip. B pump starts
5	imf pc02 dmf pc02	C duo	A Reactor Zone Fans Trip.
6	Bat ms1b hilevel	C all	Fail level control for 2B1 moisture sep. creating high level
7a	N/A	M all	Reactor Fails to Scram. (ATWS)
7b	imf pc05a	M all	Steam Leak in Primary and Secondary Containment with MSIV Closure.

\* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

SIMULATOR EVALUATION GUIDE

TITLE : NRC-2  
REVISION : 0  
DATE : August 30, 2001  
PROGRAM : BFN Operator Training - Hot License

PREPARED BY: \_\_\_\_\_ / \_\_\_\_\_  
(Operations Instructor) Date

REVIEWED BY: \_\_\_\_\_ / \_\_\_\_\_  
(LOR Lead Instructor or Designee) Date

REVIEWED BY: \_\_\_\_\_ / \_\_\_\_\_  
(Operations Training Manager or Designee) Date

CONCURRED : \_\_\_\_\_ / \_\_\_\_\_  
(Operations Superintendent or Designee) Date

VALIDATION : \_\_\_\_\_ / \_\_\_\_\_  
BY (Operations SRO: Required for Exam Scenarios Only) Date

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NUCLEAR TRAINING REVISION/USAGE LOG				
REVISION NUMBER	DESCRIPTION OF REVISION	DATE	PAGES AFFECTED	REVIEWED BY
0	INITIAL	8/30/01	All	

I. Program: BFN Operator Training

II. Course: Hot License Training

III. Title: NRC 2

IV. Length of Scenario:

V. Examination Objectives:

A. Terminal Objective

1. Perform routine shift turnover, plant assessment and routine shift operation in accordance with BFN procedures.
2. Given abnormal conditions, the operating crew will place the unit in a stabilized condition per normal, abnormal, annunciator and emergency procedures.

B. Enabling Objectives:

1. The operating crew will reduce power to perform 2-SR-3.3.1.1.8(9).
2. The operating crew will recognize and respond to a failure of 'A' RPS Bus.
3. The operating crew will recognize and respond to an isolation of 'A' SJAE.
4. The operating crew will recognize and respond to a control rod drive pump trip.
5. The operating crew will recognize and respond to a trip of the reactor zone ventilation fans.
6. The operating crew will recognize and respond to a high level in a moisture separator.
7. The operating crew will recognize and respond to an ATWS.
8. The operating crew will recognize and respond to a main steam line break inside primary containment.
9. The operating crew will recognize and respond to an MSIV closure due to a steam line break in the tunnel.

VI. References:

The procedures used in the simulator are controlled copies and are used in development and performance of simulator scenarios. Scenarios are validated prior to use, and any procedure differences will be corrected using the procedure revision level present in the simulator. Any procedure differences noted during presentation will be corrected in the same manner. As such, it is expected that the references listed in this section need only contain the reference material which is not available in the simulator.

VII. Training Materials:

- A. This Lesson Plan (NRC 2)
- B. Simulator
- C. Control Rod Insertion Sheets



## VIII. Scenario Summary

Given Unit 2 at 100% power a power reduction will be performed to perform 2-SR-3.3.1.1.8(9), Turbine Control Valve Fast Closure. A failure of 'A' RPS bus will occur. 'A' steam jet air ejector will isolate requiring the crew to place the 'B' steam jet air ejector in service. The operating CRD pump will subsequently trip requiring response in accordance with the Abnormal Operating Procedure. The crew will then experience a reactor zone ventilation fan trip and will be required to place the B set of fans in service.. A level control failure on 2B1 moisture separator will trip the main turbine and the operating crew will experience a failure to auto scram and an ATWS is experienced when the manual scram is inserted. When all rods are in the crew will experience leaks in primary and secondary containment resulting in an MSIV closure.

IX. Console Operator Instructions

A. Scenario File Summary

		MF/RF/IO#	<u>Description</u>
1.	File	bat app08ace	
a.		mrf rp06a byp	Bypasses MSIV Group I closure
		mrf rp06b byp	on low RPV level, Group 6
		mrf rp06c byp	ventilation, and drywell control
		mrf rp06d byp	air isolation
		mrf rp14a byp	
		mrf rp14b byp	
		mrf ia08 byp	
2.	File:	bat app16fg	
		MF/RF/IO#	<u>Description</u>
		mrf rh14 byp	Bypasses LPCI Loop I and II
		mrf rh15 byp	injection valve timers
3.	File	bat app02	
		MF/RF/IO#	<u>Description</u>
		mrf rp12a test	Bypasses ARI signals
		mrf rp12b test	
4.	File	bat app01f	
		MF/RF/IO#	<u>Description</u>
		mrf rp13a byp	Bypasses auto scram signals
		mrf rp13b byp	
		mrf rp13c byp	
		mrf rp13d byp	
5.	File	bat atws-1	
		MF/RF/IO#	<u>Description</u>
		dmf rd09a	Allows sdv to drain
		dmf rd09b	

**B. Console Operator Manipulations**

<b>ELAP. TIME</b>	<b>IC/MF/RF #</b>	<b>DESCRIPTION/ACTION</b>
Sim. Setup	rst 38	100%, MOC
Sim. Setup	bat atws75	75% ATWS
Sim Setup	imf rp06	Failure to auto scram
SIM Setup	imf rp14b	Failure to auto ARI
Sim. Setup	bat torhrc	Tags out 2C RHR Pump
Sim. Setup	Manual	Place HO tag on 2C RHR Pump
When directed by lead examiner	imf rp01a	Trip 'A' RPS MG Set
Three minutes after being dispatched	mrf rp03a	Places 'A' RPS on alternate
When directed by lead examiner	imf og04a	Isolates 'A' SJAE
When directed by lead examiner	imf rd01a	Trip 'A' CRD Pump
When directed by lead examiner	imf pc02 dmf pc02	Trip running reactor zone ventilation fans and allow restart
When directed by lead examiner	bat ms1bhilevel	Fails 2B1 moisture separator level control
When reactor scrams	bat sdv	Removes SDV level switch failure
When requested, wait 5 minutes	bat app02	Defeats ARI
When requested, wait 5 minutes	bat app01f	Bypasses scram signals
When requested, wait 5 minutes	bat app08ace	Bypasses Group 6 low level isolations
When scram reset and SDV drains open	bat atws-1	Allows SDV to drain
When all rods are in	imf th33a 100 10:00 imf th32a 100 5:00	Main steam line break inside primary containment and in the steam tunnel
When requested, wait 5 minutes	bat app16fg	Defeats RHR injection timers

Terminate the scenario when the following conditions are satisfied or upon request of the Chief Examiner:

1. All rods in
2. RPV water level restored +2" to +51" and stable
3. Drywell pressure under control

- X. Information to Floor Examiners:
- A. Ensure recorders are inking and recording and ICS is active and updating.
  - B. Assign Crew Positions based on the required rotation.
    - 1. SRO: Unit Supervisor
    - 2. BUO: Board Unit Operator
    - 3. DUO: Desk Unit Operator
  - C. Conduct a shift turnover with the Unit Supervisor.
  - D. Direct the shift crew to review the control board and take note of present conditions, alarms, etc.
  - E. Terminate the scenario when the following conditions are satisfied are at the request of the floor/lead instructor/evaluator.
    - 1. All rods in
    - 2. Water level restored +2" to +51"
    - 3. Drywell pressure under control

XI. Simulator Event Guide

Event 1: Power Reduction for SR

POSITION	EXPECTED ACTIONS	SAT/UNSAT/COMMENTS
SRO	Directs load reduction for 2-SR-3.3.1.1.8(9)	Turbine Control Valve Fast Closure
BUO	Determines latest copy of procedure.	_____
BUO	Reduces reactor power to <80% in accordance with 2-GOI-100-12 & 2-OI-68.	_____
BUO	Reduces recirc flow in accordance with OI-68, Section 6.0	_____
BUO	Lowers setpoint of individual recirc pump controls to reduce power to 80% at 8 to 10 mw/min	_____
BUO	Performs SR 3.3.1.1.I	Core Thermal Hydraulic Stability

Event 2: Failure of 'A' RPS

<u>POSITION</u>	<u>EXPECTED ACTIONS</u>	<u>SAT/UNSAT/COMMENTS</u>
CREW	Announce half scram	_____
B/D/U	Reports RPS 2A failure	_____
SRO	Directs BUO/DUO to carry out actions of AOI-99-1	_____
SRO	Dispatch O/S SRO to restore RPS and to determine reason for loss of MG set	_____
CREW	Monitors steam Tunnel Temp	_____
SRO	Informs crew that RPS 2A has been placed on Alt. power supply	_____
SRO	Directs half scram reset	_____
	Directs PCIS logic reset	_____
	Directs system restoration per 2-OI-99	_____
SRO	Contacts Elect. Maint. to Troubleshoot RPS MG set	_____
BUO	Resets half scram	_____
	Reset PCIS logic	_____
DUO	Places Rx/RF fans in service in SLOW	_____
	Secures SBTGT "C"	_____
	Notifies Unit 1 to stop SBTGT "A & B"	_____
	Verifies/Opens DWCA suction valves	_____
	Places PSC head tank pumps in service	_____
	Restores DW DP compressor to automatic operation	_____
	Restores DW floor and equip. drain systems to normal operation	_____
	Restores Radiation monitoring system (DW CAM)	_____
	Restores H2/O2 analyzer to operation	_____

Restores RWCU system to service

\_\_\_\_\_

Secures CB Emergency Press system to standby  
(optional)

\_\_\_\_\_

Resets TIP isolation

\_\_\_\_\_

Reports systems returned to normal.

\_\_\_\_\_

Event 2:

Failure of 'A' RPS (Continued)

BUO/DUO

Reports Steam Tunnel Temp. lowering when Rx  
Bldg. Vent. restored

\_\_\_\_\_

DUO

Places Rx/RF fans in service in FAST  
5 minutes after restart

\_\_\_\_\_



Event 3: Failure of 'A' SJAE

POSITION	EXPECTED ACTIONS	SAT/UNSAT/COMMENTS
CREW	Determines 'A' SJAE isolated in response to 9-53 Off-Gas Flow low alarm	_____
SRO	Directs 'B' SJAE placed into service per OI-66	_____
DUO	Places 'B' SJAE in service per OI-66, Section 8.14 <ul style="list-style-type: none"> <li>• Verifies condensate inlet and outlet open</li> <li>• Verifies SJAE inlet valve open</li> <li>• Opens SJAE outlet valve</li> <li>• Places pressure control handswitch in open</li> </ul>	_____

Event 4: 2A CRD Pump Trip

POSITION	EXPECTED ACTION (S)	COMMENTS
CREW	Determines 2A CRD Pump Trip	_____
SRO	Directs actions per AOI-85-3	_____
BUO	- Places FIC in Man at zero setting.	_____
BUO	- Start 1B CRD Pump	_____
BUO	Adjusts CRD Sys flow and pressures	_____
BUO	Balance CRD FIC and place in auto	_____
SRO	Dispatches AUO/OS SRO to check CRD pump 2A	_____

Event 5: 'A' Reactor Zone Fan trip

POSITION	EXPECTED ACTIONS	INST. INFO/NOTES
CREW	Recognizes Rx Bldg Ventilation abnormal alarm	_____
SRO	Directs response per ARP	_____
DUO	Determines 'A' Reactor Zone Fans have tripped	_____
SRO	Directs 'B' reactor Zone Fans placed in service	_____
DUO	Places 'B' reactor Zone fans in SLOW	_____
DUO	Reports 'B' reactor zone fans in service	_____
CREW	Monitors steam tunnel temperature on TIS-1-60	_____
DUO	After five minutes, places 'B' Reactor Zone fans in FAST	_____

Event 6: MSDP and LCV for 2B1 failure

POSITION	EXPECTED ACTIONS	INST. INFO/NOTES
BUO/DUO	Respond to annunciators utilizing ARPs	_____
BUO/DUO	Recognizes MSDP B1 tripped	_____
CREW	Recognizes potential for turbine trip. With both MSDP and LCV failure turbine trip will occur on high level	_____
SRO	Directs SRO/AUO to turbine bldg. to attempt to restore level control. Attempt to start MSDP &/or get high level control.	_____
CREW	Makes determination that alarms are valid	_____
SRO	Directs Load reduction in accordance with GOI-100-12. Will extend time before turbine trip	_____

Event 7A: Turbine trip, failure to auto scram, ATWS		
POSITION	EXPECTED ACTIONS	INST. INFO/NOTES
CREW	Recognizes Turbine trip and failure of Rx to scram	_____
SRO	Directs manual Scram	_____
BUO	Inserts Manual scram as directed	_____
	Reports all rods not full in	_____
	Places mode switch in S/D	_____
	Initiates ARI	_____
SRO	Monitors Rx power	_____
	Enters EOI-1, & C5	_____
	Directs ADS inhibited	_____
SRO	Directs power control per RC/Q	_____
	Direct EOI App. 2, 1f, 1d, 8a, 8c, 8e	_____
	Directs App. 4	_____
	Directs level lower to < -50"	_____
BUO/DUO	Performs App. 4	_____
	Lowers level to < -50"	_____
	Inhibits ADS and verifies via ann.	Critical Task _____
BUO	Resets Scram when App. 1F & 2 complete Drives Rods when 85-586 closed.	_____
BUO	Scrams reactor when SDV alarms clear	Critical Task _____
BUO	Reports all rods in	_____
SRO	Directs level restored +2" to +51"	_____
SRO	Exits C5 and directs restoration per AOI-100-1	_____
BUO	Restores vessel level +2" to +51"	_____

Event 7B,C      Leak in Primary and Secondary Containment with MSIV Closure		
POSITION	EXPECTED ACTIONS	INST. INFO/NOTES
CREW	Recognizes MSIV closure	_____
BUO/DUO	Controls Rx pressure in specified band (800 - 1000)	_____
DUO	Evacuates Turbine Bldg.	_____
CREW	Recognizes drywell pressure and temperature rising	_____
SRO	@ 2.45# drywell pressure Directs pressure control per EOI 2 and re-enter EOI-1	_____
	Vent per App 12 and H2 O2 Avail. in service.	_____
SRO	Directs Supp. Chamber sprays initiated prior to reaching 12 psig Supp. Chamber pressure.	_____
DUO	Initiates Supp. Chamber Sprays.	_____
	Vents Containment per App. 12	_____
	Places H2 O2 Anal. in service.	_____
SRO	Directs Drywell Spray initiated @ 12 psig, if required.	_____
DUO	Initiates Drywell Spray	_____
	Reports D/W temp. & Press lowering	_____
SRO	Directs Sprays secured before reaching "0" psi in sprayed area	_____
	Directs AOI-100-1	_____
SRO	Classifies Event as an SAE (1.2-S)	_____

SHIFT TURNOVER SHEET

Equipment Out of Service/LCOs 2C RHR Pump tagged out for minor maintenance - 6 hours into a 7 day  
LCO.

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Operations/Maintenance For the Shift Reduce power to 80% for performance of 2-SR-3.3.1.1.8(9),  
Turbine Control Valve Fast Closure

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Unusual Conditions/Problem Areas Severe thunder storm warning in effect.

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