

March 1, 1993

Docket Nos. 50-321

Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

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Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT - EDWIN I. HATCH NUCLEAR PLANT,
UNIT 1 (TAC NO. M85293)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 184 to Facility Operating License DPR-57 Edwin I. Hatch Nuclear Plant, Unit 1. The amendment consist of changes to the Technical Specifications (TS) in response to your application dated December 21, 1992.

The amendment deletes two primary containment isolation valves associated with the residual heat removal head spray mode from Hatch Unit 1 TS Tables 3.7-1, 4.2-1, and 3.7-4. In addition, Table 3.7-3 will be revised to identify Penetration X-17 as a spare.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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P PDR

Enclosures:

1. Amendment No. 184 to DPR-57
2. Safety Evaluation

cc w/enclosure:
See next page

OFFICE	PDII-3/IA	PDII-3/PM	SCSB/BC	SRSB/BC	OGC	PDII-3/D
NAME	L. BERRY	K. JABBOUR	R. BARRETT	R. JONES	E. HOLLER	D. MATTHEWS
DATE	2/4/93	2/4/93	2/4/93	2/8/93	2/11/93	2/25/93

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Senior Vice President -
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Georgia Power Company
P. O. Box 1295
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Sincerely,

Handwritten signature of Kahtan N. Jabbour in cursive.

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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cc w/enclosure:
See next page

Mr. W. G. Hairston, III
Georgia Power Company

Edwin I. Hatch Nuclear Plant

cc:

Mr. Ernest L. Blake, Jr.
Shaw, Pittman, Potts and Trowbridge
2300 N Street, NW.
Washington, DC 20037

Mr. R. P. McDonald
Executive Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Mr. J. T. Beckham
Vice President - Plant Hatch
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Mr. Alan R. Herdt, Chief
Project Branch #3
U. S. Nuclear Regulatory Commission
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

Mr. S. J. Bethay
Manager Licensing - Hatch
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Mr. Dan H. Smith, Vice President
Power Supply Operations
Oglethorpe Power Corporation
2100 East Exchange Place
Tucker, Georgia 30085-1349

Mr. L. Sumner
General Manager, Nuclear Plant
Georgia Power Company
Route 1, Box 439
Baxley, Georgia 31513

Charles A. Patrizia, Esquire
Paul, Hastings Janofsky & Walker
12th Floor
1050 Connecticut Avenue, NW.
Washington, DC 20036

Resident Inspector
U. S. Nuclear Regulatory Commission
Route 1, Box 725
Baxley, Georgia 31513

Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
101 Marietta Street, NW. Suite 2900
Atlanta, Georgia 30323

Mr. Charles H. Badger
Office of Planning and Budget
Room 610
270 Washington Street, SW.
Atlanta, Georgia 30334

Harold Reheis, Director
Department of Natural Resources
205 Butler Street, SE., Suite 1252
Atlanta, Georgia 30334

Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-321
EDWIN I. HATCH NUCLEAR PLANT, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 184
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated December 21, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 184, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: March 1, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 184

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages

3.2-26

3.7-18

3.7-22

3.7-23

Insert Pages

3.2-26

3.7-18

3.7-22

3.7-23

Notes for Table 4.2-1 (Cont'd)

from other BWR's for which the same design instrument operates in an environment similar to that of HNP-1. The failure rate data must be reviewed and approved by the AEC prior to any change in the once-a-month frequency.

- e. This instrumentation is exempted from the instrument functional test definition. This instrument functional test will consist of injecting a simulated electrical signal into the measurement channels.
 - f. Standard current source used which provides an instrument channel alignment. Calibration using a radiation source shall be made once per operating cycle.
-

Logic system functional tests and simulated automatic actuation shall be performed once each operating cycle

for the following:

- | | |
|---|------------------------------------|
| 1. Main Steam Line Isolation Valves | 8. Reactor Water Cleanup Isolation |
| 2. Main Steam Line Drain Valves | 9. Drywell Isolation Valves |
| 3. Reactor Water Sample Valves | 10. TIP Withdrawal |
| 4. RHR - Isolation Valve Control | 11. Atmospheric Control Valves |
| 5. Shutdown Cooling Valves | 12. Sump Drain Valves |
| 6. Deleted | 13. Standby Gas Treatment |
| 7. Drywell Equipment Sump Discharge to Radwaste | 14. Reactor Building Isolation |

The logic system functional tests shall include a calibration of time delay relays and timers necessary for proper functioning of the trip systems.

TABLE 3.7-1 (Cont'd)
 PRIMARY CONTAINMENT ISOLATION VALVES WHICH
 RECEIVE A PRIMARY CONTAINMENT ISOLATION SIGNAL

Isolation Group (b)	Valve Identification (d)	Number of Power Operated Valves		Maximum Operating Time (sec)	Normal Position (e)	Action on Initiating Signal (e)
		Inside	Outside			
6	RHR reactor shutdown cooling suction (supply) (E11-F008, E11-F009)	1	1	24	C	SC
3	HPCI - turbine steam (E41-F002, E41-F003)	1	1	50	0	GC
4	RCIC - turbine steam (E51-F007, E51-F008)	1	1	20	0	GC
5	Reactor water cleanup from recirculation loop (G31-F001, G31-F004)	1	1	30	0	GC
2	Post-accident sampling system supply (B21-F111, B21-F112)		2	5	C	SC
2	Post-accident sampling system return (E41-F122, E41-F121)		2	5	C	SC
2	Core spray test line to suppression pool (E21-F015A,B)		1 each line	57	C	SC

Table 3.7-3

Testable Penetrations with Testable Bellows

<u>Penetration Number</u>	<u>Penetration Description</u>	<u>Notes</u>
X-7A	Primary Steamline 'A'	(1) (2) (4) (6)
X-7B	Primary Steamline 'B'	(1) (2) (4) (6)
X-7C	Primary Steamline 'C'	(1) (2) (4) (6)
X-7D	Primary Steamline 'D'	(1) (2) (4) (6)
X-8	Steamline Condensate Drain	(1) (2) (4) (6)
X-9A	Feedwater Line 'A'	(1) (2) (4) (6)
X-9B	Feedwater Line 'B'	(1) (2) (4) (6)
X-10	Steam to RCIC Turbine	(1) (2) (4) (6)
X-11	Steam Line to HPCI Turbine	(1) (2) (4) (6)
X-12	RHRS Shutdown Cooling Suction	(1) (2) (4) (6)
X-13A	RHR LPCI to Reactor	(1) (2) (4) (6)
X-13B	RHR LPCI to Reactor	(1) (2) (4) (6)
X-14	Reactor Water Cleanup Line	(1) (2) (4) (6)
X-16A	Core Spray to Reactor	(1) (2) (4) (6)
X-16B	Core Spray to Reactor	(1) (2) (4) (6)
X-17	Spare	(1) (2) (4) (6)
X-201A through X-201H	Drywell Suppression Chamber to Vent Line	(1) (2) (4) (6)

Table 3.7-4

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>
X-7A	B21-F022A & F028A Main Steam Isolation Valves	(1) (2) (3) (5) (9)
X-7B	B21-F022B & F028B Main Steam Isolation Valves	(1) (2) (3) (5) (9)
X-7C	B21-F022C & F028C Main Steam Isolation Valves	(1) (2) (3) (5) (9)
X-7D	B21-F022D & F028D Main Steam Isolation Valves	(1) (2) (3) (5) (9)
X-8	B21-F016 & F019	(1) (2) (4) (5) (9)
X-9A	B21-F010B	(1) (2) (3) (5) (10)
X-9A	B21-F032B	(1) (2) (3) (5) (10)
X-9A	E41-F006	(1) (2) (4) (5) (9)
X-9A	G31-F203	(1) (2) (4) (5) (10)
X-9B	B21-F010A	(1) (2) (3) (5) (10)
X-9B	B21-F032A	(1) (2) (3) (5) (10)
X-9B	E51-F013	(1) (2) (4) (5) (9)
X-9B	G31-F039	(1) (2) (4) (5) (10)
X-10	E51-F007, F008	(1) (2) (4) (5) (9)
X-11	E41-F002 & F003	(1) (2) (4) (5) (9)
X-12	E11-F008 & F009	(1) (2) (4) (5) (9)
X-13A	E11-F015A	(1) (2) (4) (5) (9)
X-13B	E11-F015B	(1) (2) (4) (5) (9)
X-14	G31-F001 & F004	(1) (2) (4) (5) (9)
X-16A	E21-F005A	(1) (2) (4) (5) (9)
X-16B	E21-F005B	(1) (2) (4) (5) (9)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 184 TO FACILITY OPERATING LICENSE DPR-57

GEORGIA POWER COMPANY, ET AL.

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-321

1.0 INTRODUCTION

By letter dated December 21, 1992, Georgia Power Company, et al. (the licensee), proposed a revision to the Edwin I. Hatch Nuclear Plant, Unit 1, Technical Specifications (TS). The revision would delete two primary containment isolation valves (PCIVs), E11-F022 and E11-F023, associated with the residual heat removal (RHR) head spray mode from Hatch Unit 1 TS Table 3.7-1, "Primary Containment Isolation Valves Which Receive a Primary Containment Isolation Signal;" Table 4.2-1, "Check, Functional Test, and Calibration Minimum Frequency for Instrumentation Which Initiates Reactor Vessel and Primary Containment Isolation;" and Table 3.7-4, "Primary Containment Testable isolation Valves." In addition, Table 3.7-3, "Testable Penetrations with Testable Bellows," will be revised to identify Penetration X-17 as a spare. The testable bellows for Penetration X-17 will not be removed and surveillance testing of the bellows will continue.

2.0 EVALUATION

The purpose of this change is to support the physical removal of the above valves. The function of the head spray mode of RHR is to spray water in the reactor vessel head area, while in the process of reactor shutdown, to provide a rapid reactor head cooldown. It is a low flow, low pressure system designed to supply water to the vessel steam dome, through the head spray nozzle. Hatch Unit 1 was designed with this capability when it was anticipated that reactor vessel head conditions would be a critical path for beginning a refueling outage. However, operating experience has shown that the RHR head spray mode is unnecessary for cooldown of the vessel head, does not perform any safety-related functions, and is not addressed in the emergency operating procedures. In addition, use of the head spray mode of RHR is not practical given the restrictive cooldown rates established in TS 3.6. Additionally, removal of the RHR head spray piping spool piece delayed removal of the reactor head for refueling which resulted in unnecessary personnel exposure. Accordingly, the licensee deactivated the RHR head spray mode, and removed the spool piece (between the flanged connection on the reactor vessel head and the flanged connection just downstream of valve E11-F019) in 1986 in accordance with 10 CFR 50.59.

Although the RHR head spray mode of operation was deactivated and the spool piece removed, the associated PCIVs, E11-F022 and E11-F023, were not removed. Accordingly, testing in accordance with 10 CFR 50, Appendix J, has continued on the PCIVs, resulting in unnecessary personnel exposure. The licensee currently plans to physically remove these valves during the upcoming Unit 1, spring 1993, refueling outage. Following removal of the PCIVs, Penetration X-17 will be capped, thus, preserving containment integrity.

Based on its review, the NRC staff finds that the proposed revision has no adverse impact on safety and does not pose an undue risk to public health and safety. Therefore, it is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 6997 dated February 3, 1993). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Kahtan Jabbour

Date: March 1, 1993