

May 19, 1992

Docket Nos. 50-321  
and 50-366

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Mr. W. G. Hairston, III  
Senior Vice President -  
Nuclear Operations  
Georgia Power Company  
P. O. Box 1295  
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENTS - EDWIN I. HATCH NUCLEAR PLANT,  
UNITS 1 AND 2 (TACS M77820 AND M77821)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 179 to Facility Operating License DPR-57 and Amendment No. 120 to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated September 21, 1990, as supplemented February 19, 1992.

The amendments would modify the TS relating to protective instrumentation for Hatch Units 1 and 2 by adding notes to (1) Unit 1 TS Tables 3.1-1 and 3.2-1 to allow for placing an inoperable channel in a tripped condition, and (2) Unit 1 TS Table 3.2-1 and Unit 2 TS Table 3.3.2-1 to allow the temporary bypassing of the reactor water clean-up (RWCU) system differential flow isolation instrumentation during periods of system restoration, maintenance, or testing.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,  
*/s/*

Kahtan N. Jabbour, Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 179 to DPR-57
2. Amendment No. 120 to NPF-5
3. Safety Evaluation

cc w/enclosures:  
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PDII-3/LA LBerry 4/19/92	PDII-3/PM KJabbour/rst 4/22/92	BC:SICB <del>S. Newberry</del> 4/29/92	OGC R. Bachmann 5/1/92	PDII-3/D DMatthews 5/19/92
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

May 19, 1992

Docket Nos. 50-321  
and 50-366

Mr. W. G. Hairston, III  
Senior Vice President -  
Nuclear Operations  
Georgia Power Company  
P. O. Box 1295  
Birmingham, Alabama 35201

Dear Mr. Hairston:

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UNITS 1 AND 2 (TACS M77820 AND M77821)

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The amendments would modify the TS relating to protective instrumentation for Hatch Units 1 and 2 by adding notes to (1) Unit 1 TS Tables 3.1-1 and 3.2-1 to allow for placing an inoperable channel in a tripped condition, and (2) Unit 1 TS Table 3.2-1 and Unit 2 TS Table 3.3.2-1 to allow the temporary bypassing of the reactor water clean-up (RWCU) system differential flow isolation instrumentation during periods of system restoration, maintenance, or testing.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Kahtan N. Jabbour".

Kahtan N. Jabbour, Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 179 to DPR-57
2. Amendment No. 120 to NPF-5
3. Safety Evaluation

cc w/enclosures:  
See next page

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DATED:           MAY 19, 1992          

AMENDMENT NO. 179 TO EDWIN I. HATCH NUCLEAR PLANT, UNIT 1  
AMENDMENT NO. 120 TO EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 179  
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated September 21, 1990, as supplemented February 19, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 179, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: May 19, 1992



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKFT NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 120  
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated September 21, 1990, as supplemented February 19, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 120, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: May 19, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 179

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

AND

TO LICENSE AMENDMENT NO. 120

FACILITY OPERATING LICENSE NO. NPI-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

	<u>Remove Pages</u>	<u>Insert Pages</u>
Unit 1	3.1-6 3.2-3 3.2-4	3.1-6 3.2-3 3.2-4
Unit 2	3/4 3-12 3/4 3-15 ----	3/4 3-12 3/4 3-15 3/4 3-15a

Table 3.1-1 (Cont'd)

Scram Number (a)	Source of Scram Trip Signal	Operable Channels Required Per Trip System (b)	Scram Trip Setting	Source of Scram Signal is Required to be Operable Except as Indicated Below
12	Turbine Stop Valve Closure	4	≤10% valve closure from full open Tech Spec 2.1.A.3.	Automatically bypassed when turbine steam flow is below that corresponding to 30% of rated thermal power as measured by turbine first stage pressure.

Notes for Table 3.1-1

- a. The column entitled "Scram Number" is for convenience so that a one-to-one relationship can be established between items in Table 3.1-1 and items in Table 4.1-1.
- b. There shall be two operable or tripped trip systems for each potential scram signal. If the number of operable channels cannot be met for one of the trip systems, the inoperable channel(s) or the associated trip system shall be tripped. However, one trip signal channel of a trip system may be inoperable for up to two (2) hours during periods of required surveillance testing without tripping the channel or associated trip system, provided that the other remaining channel(s) monitoring that parameter within that trip system is (are) operable.
- c. Within 24 hours prior to the planned start of the hydrogen injection test with the reactor power at greater than 20% rated power, the normal full power radiation background level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip setpoints may be adjusted during the test based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and associated trip setpoints shall be set within 24 hours of re-establishing normal radiation levels after completion of hydrogen injection and prior to establishing reactor power levels below 20% rated power.

Table 3.2-1 (Cont.)

HATCH - UNIT 1	Rel. No. (a)	Instrument	Trip Condition Nomenclature	Required Operable Channels per Trip System (b)	Trip Setting	Action to be taken if number of channels is not met for both trip systems (c)	Remarks (d)
	4	Main Steam Line Radiation	High	2	$\leq 3$ times normal full power back-ground <sup>(e)</sup>	Initiate an orderly load reduction and close MSIVs within 8 hours.	Initiates Group 1 isolation.
	5	Main Steam Line Pressure	Low	2	$\geq 825$ psig	Initiate an orderly load reduction and close MSIVs within 8 hours.	Initiates Group 1 isolation. Only required in RUN mode, therefore activated when Mode Switch is in RUN position.
	6	Main Steam Line Flow	High	2	$\leq 138\%$ rated flow ( $\leq 5$ psid)	Initiate an orderly load reduction and close MSIVs within 8 hours.	Initiates Group 1 isolation.
	7	Main Steam Line Tunnel Temperature	High	2	$\leq 194^\circ\text{F}$	Initiate an orderly load reduction and close MSIVs within 8 hours.	Initiates Group 1 isolation.
3.2-3	8	Reactor Water Cleanup System Differential Flow	High	1	20-80 gpm	Isolate reactor water cleanup system.	(f)
	9	Reactor Water Cleanup Area Temperature	High	2	$\leq 150^\circ\text{F}$	Isolate reactor water cleanup system.	
Amendment No. 179	10	Reactor Water Cleanup Area Ventilation Differential Temperature	High	2	$\leq 67^\circ\text{F}$	Isolate reactor water cleanup system.	
	11	Condenser Vacuum	Low	2	$\geq 7"$ Hg. vacuum	Initiate an orderly load reduction and close MSIVs within 8 hrs.	Initiate Group 1 Isolation
	12	Drywell Radiation	High	1	$\leq 138$ R/HR.	Close the affected isolation valves within 24 hours or be in Hot Shutdown within the next 6 hours and in Cold Shutdown within the next 30 hours.	Isolates containment purge and vent valves.

Notes for Table 3.2-1

- a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between lines in Table 3.2-1 and items in Table 4.2-1.
- b. Primary containment integrity shall be maintained at all times prior to withdrawing control rods for the purpose of going critical, when the reactor is critical, or when the reactor water temperature is above 212°F and fuel is in the reactor vessel except while performing low-power physics tests at atmospheric pressure at power levels not to exceed 5 MWt, or performing an inservice vessel hydrostatic or leakage test.

When primary containment integrity is required, there shall be two operable or tripped trip systems for each function.

When performing inservice hydrostatic or leakage testing on the reactor vessel with the reactor coolant temperature above 212°F, reactor vessel water level instrumentation associated with the low low (Level 2) trip requires two operable or tripped channels. The drywell pressure trip is not required because primary containment integrity is not required.

- c. If the number of operable channels cannot be met for one of the trip systems, the inoperable channel(s) or the associated trip system shall be tripped. However, one trip signal channel of a trip system may be inoperable for up to 2 hours during periods of required surveillance testing without tripping the channel or associated trip system, provided that the other remaining channel(s) monitoring that same parameter within that trip system is (are) operable.
- d. The valves associated with each Group isolation are given in Table 3.7-1.
- e. Within 24 hours prior to the planned start of the hydrogen injection test with the reactor power at greater than 20% rated power, the normal full-power radiation background level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip setpoints may be adjusted during the test based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and associated trip setpoints shall be set within 24 hours of re-establishing normal radiation levels after completion of hydrogen injection and prior to establishing reactor power levels below 20% rated power.
- f. The high differential flow signal to the RWCU isolation valves may be bypassed for up to 2 hours during periods of system restoration, maintenance, or testing.

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

HATCH - UNIT 2

3/4 3-12

Amendment No. 120

TRIP FUNCTION	VALVE GROUPS OPERATED BY SIGNAL(a)	MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)	APPLICABLE OPERATIONAL CONDITION	ACTION
<b>2. SECONDARY CONTAINMENT ISOLATION</b>				
a. Reactor Building Exhaust Radiation - High (2D11-K609 A, B, C, D)	6, 10, 12, *	2	1,2,3,5 and**	24
b. Drywell Pressure - High (2C71-N650 A, B, C, D)	2, 6, 7, 10, 12, *	2	1, 2, 3	24
c. Reactor Vessel Water Level - low low (level 2) (2B21-N682 A, B, C, D)	5, *	2	1, 2, 3	24
d. Refueling Floor Exhaust Radiation - High (2D11-K611 A, B, C, D)	6, 10, 12, *	2	1,2,3,5 and**	24
<b>3. REACTOR WATER CLEANUP SYSTEM ISOLATION</b>				
a. Δ Flow - High (2G31-N603 A, B)	5	1	1, 2, 3	25(1)
b. Area Temperature - High (2G31-N662 A, D, E, H, J, M)	5	1	1, 2, 3	25
c. Area Ventilation Δ Temp. - High (2G31-N663 A, D, E, H, J, M; 2G31-N661 A, D, E, H, J, M; 2G31-N662 A, D, E, H, J, M)	5	1	1, 2, 3	25
d. SLCS Initiation (NA)	5 <sup>(g)</sup>	NA	1, 2, 3	25
e. Reactor Vessel Water Level - Low Low (Level 2) (2B21-N682 A, B, C, D)	5, *	2	1, 2, 3	25

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

ACTION

- ACTION 20 - Be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.
- ACTION 21 - Be in at least STARTUP with the main steam line isolation valves closed within 2 hours or be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.
- ACTION 22 - Be in at least STARTUP within 2 hours.
- ACTION 23 - Be in at least STARTUP with the Group 1 isolation valves closed within 2 hours or in at least HOT SHUTDOWN within 6 hours.
- ACTION 24 - Establish SECONDARY CONTAINMENT INTEGRITY with the standby gas treatment system operating within one hour.
- ACTION 25 - Isolate the reactor water cleanup system.
- ACTION 26 - Close the affected system isolation valves and declare the affected system inoperable.
- ACTION 27 - Verify power availability to the bus at least once per 12 hours or close the affected system isolation valves and declare the affected system inoperable.
- ACTION 28 - Close the shutdown cooling supply and reactor vessel head spray isolation valves unless reactor steam dome pressure  $\geq$  145 psig.
- ACTION 29 - Either close the affected isolation valves within 24 hours or be in HOT SHUTDOWN within the next 6 hours and in COLD SHUTDOWN within the next 30 hours.

NOTES

- \* Actuates the standby gas treatment system.
- \*\* When handling irradiated fuel in the secondary containment.
- \*\*\* When performing inservice hydrostatic or leak testing with the reactor coolant temperature above 212° F.
- a. See Specification 3.6.3, Table 3.6.3-1 for valves in each valve group.
- b. A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.

- c. With a design providing only one channel per trip system, an inoperable channel need not be placed in the tripped condition where this would cause the Trip Function to occur. In these cases, the inoperable channel shall be restored to OPERABLE status within 2 hours or the ACTION required by Table 3.3.2-1 for that Trip Function shall be taken.
- d. Trips the mechanical vacuum pumps.
- e. A channel is OPERABLE if 2 of 4 instruments in that channel are OPERABLE.
- f. May be bypassed with all turbine stop valves closed.
- g. Closes only RWCU outlet isolation valve 2G3i-F004.
- h. Alarm only.
- i. Adjustable up to 60 minutes.
- j. Isolates containment purge and vent valves.
- k. Within 24 hours prior to the planned start of the hydrogen injection test with the reactor power at greater than 20% rated power, the normal full-power radiation background level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip setpoints may be adjusted during the test based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and associated trip setpoints shall be set within 24 hours of re-establishing normal radiation levels after completion of hydrogen injection and prior to establishing reactor power levels below 20% rated power.
- l. The high differential flow isolation signal to the RWCU isolation valves may be bypassed for up to 2 hours during periods of system restoration, maintenance, or testing.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 179 TO FACILITY OPERATING LICENSE DPR-57  
AND AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE NPF-5

GEORGIA POWER COMPANY, ET AL.

EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-321 AND 50-366

1.0 INTRODUCTION

By letter dated September 21, 1990, as supplemented February 19, 1992, Georgia Power Company, et al. (the licensee), proposed changes to the Technical Specifications (TS) for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The proposed changes are:

- (1) Revise Unit 1 TS Tables 3.1-1 and 3.2-1 to allow the placing of an inoperable channel in the tripped condition.
- (2) Revise Unit 1 TS Table 3.2-1 and Unit 2 TS Table 3.3.2-1 to allow the temporary bypassing (i.e., up to two hours) of the reactor water cleanup (RWCU) system differential flow isolation instrumentation during periods of system restoration, maintenance, or testing.

2.0 EVALUATION

2.1 Proposed Change 1

The proposed change allows the placing of an inoperable channel in the tripped condition.

For Table 3.1-1, "Reactor Protection system (RPS) Instrumentation Requirements," the addition to footnote 6 will not change the results of the actions required if the number of operable channels is not met for each trip system. However, for Table 3.2-1, "Instrumentation Which Initiates Reactor Vessel and Primary Containment Isolation," allowing the placing of an inoperable channel in the tripped condition in lieu of the entire trip system, will result in avoiding unnecessary isolations while still maintaining protection in accordance with the single failure criterion as defined in 10 CFR 50, Appendix A. These changes will also make Unit 1 TS similar to Unit 2 and the BWR/4 Standard TS.

Based on its review, the NRC staff finds that this change has no adverse impact on safety and does not pose an undue risk to public health and safety. Therefore, it is acceptable.

## 2.2 Proposed Change 2

The proposed change allows the temporary bypassing (i.e., up to two hours) of the RWCU system differential flow isolation instrumentation during periods of system restoration, maintenance, or testing.

Although isolation of the RWCU system is necessary to mitigate design basis events, General Electric (GE) report EASD-24-0489 (enclosed with the licensee's submittal) concludes the differential flow instrumentation is not required to accomplish this function, because other protective instrumentation is available. The report investigated the failure of both time delay relays, and demonstrated the safety-grade instrumentation provided for accident prevention (i.e., temperature instrumentation) would isolate the RWCU system during a loss-of-coolant accident or a high energy line break (HELB) event. Therefore, bypassing the isolation signals from the RWCU differential flow instrumentation will have no impact on the ability of the primary containment isolation system to mitigate design basis events.

Additionally, the GE report shows the differential flow sensors and their logic should not be considered as an engineered safety feature (ESF), because under any postulated accident scenario, containment isolation would be accomplished via the safety-related temperature and differential temperature sensors.

Furthermore, the licensee stated that, although the design basis HELB is unaffected, smaller line breaks and breaks in colder sections of the system could be mitigated more quickly by the high differential flow signal. Therefore, the licensee proposes to leave the instrumentation installed in order to protect plant equipment and retain the Unit 1 and Unit 2 TS requirements on differential flow instrumentation.

Based on its review, the staff finds that this change has no adverse impact on safety and does not pose an undue risk to public health and safety. Therefore, it is acceptable.

## 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendments. The State official had no comments.

## 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no

significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (57 FR 11110). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: R. Musser, Region II  
K. Jabbour, NRR

Date: May 19, 1992