

September 21, 1992

Docket No. 50-321

Distribution  
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Mr. W. G. Hairston, III  
Senior Vice President -  
Nuclear Operations  
Georgia Power Company  
P. O. Box 1295  
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT - EDWIN I. HATCH NUCLEAR PLANT,  
UNIT 1 (TAC NO. M84078)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 182 to Facility Operating License DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated July 17, 1992, as supplemented July 30, 1992.

The amendment would revise the TSs for Hatch Unit 1 to extend the allowed outage time for diesel generator 1B and correct errors in the diesel generator TSs.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,  
/s/

Kahtan N. Jabbour, Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No.182 to DPR-57
- 2. Safety Evaluation

*SELB/BC*  
*CBerlinger*

cc w/enclosures:  
See next page

*8/10/192*

OFC	PDII-3/PM	PDII-3/PM	OGC	PDII-3/D
NAME	LBerry	KJabbour/rst	CPW	DMatthews
DATE	<i>8/15/92</i>	<i>8/28/92</i>	<i>9/2/92</i>	<i>9/10/92</i>

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

September 21, 1992

Docket No. 50-321

Mr. W. G. Hairston, III  
Senior Vice President -  
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Georgia Power Company  
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Sincerely,

A handwritten signature in cursive script that reads "Kahtan N. Jabbour".

Kahtan N. Jabbour, Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 182 to DPR-57
2. Safety Evaluation

cc w/enclosures:  
See next page

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DATED: SEPTEMBER 21, 1992

AMENDMENT NO. 182 TO EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

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Hatch R/F

S. Varga	14-E-4
D. Matthews	14-H-25
L. Berry	14-H-25
K. Jabbour	14-H-25
OGC-WF	15-B-18
D. Hagan	MNBB 4702
G. Hill (8)	P1-37
W. Jones	MNBB 7103
C. Grimes	11-F-23
ACRS (10)	P-135
GPA/PA	17-F-2
OC/LFMB	MNBB 4702



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 182  
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated July 17, 1992, as supplemented July 30, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 182, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective no later than 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: September 21, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 182

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages

3.5-10

3.9-5

Insert Pages

3.5-10

3.9-5

**3.5.G. Minimum Core and Containment Cooling Systems Availability**

During any period when one of the standby diesel generators is inoperable, continued reactor operation is limited to 7\* days unless operability of the diesel generator is restored within this period. During such 7\* days all of the components in the RHR system LPCI mode and containment cooling mode shall be operable. If this requirement cannot be met, an orderly shutdown shall be initiated and the reactor shall be in the Cold Shutdown Condition within 24 hours. Specification 3.9. provides further guidance on electrical system availability.

Any combination of inoperable components in the core and containment cooling systems shall not defeat the capability of the remaining operable components to fulfill the core and containment cooling functions.

When irradiated fuel is in the reactor vessel and the reactor is in the Cold Shutdown Condition, both CS systems and the LPCI and containment cooling subsystems of the RHR system may be inoperable provided that the shutdown cooling subsystem of the RHR system is operable in accordance with Specification 3.5.B.1.b and that no work is being done which has the potential for draining the reactor vessel.

**4.5.G. Surveillance of Core and Containment Cooling Systems**

When it is determined that one of the standby diesel generators is inoperable, all of the components in the RHR system LPCI mode and containment cooling mode connected to the operable diesel generators shall be verified to be operable.

\*The 1B diesel generator may be inoperable for up to 14 days during the Hatch Unit 2 tenth refueling outage to perform preventative maintenance and modification work.

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**LIMITING CONDITIONS FOR OPERATION**

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**SURVEILLANCE REQUIREMENTS**

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3.9.B.1. One Startup Auxiliary Transformer (1C or 1D) Inoperable or Only One Offsite Power Source Available (230 kV Transmission Line)

Reactor operation is permissible for seven days from the date that one startup auxiliary transformer (1C or 1D) is inoperable or incoming power is available from only one 230 kV offsite transmission line provided the increased Surveillance Requirements as stated in Specification 4.9.B.1. are implemented.

2. One Diesel Generator (1A, 1B, or 1C) Inoperable

From and after the date that one of the diesel generators is made or found to be inoperable, continued reactor operation is permissible in accordance with Specification 3.5.G. for a period not to exceed seven days\* provided that two 230 kV offsite transmission lines are available, both remaining diesel generators and associated emergency buses are operable, and the increased Surveillance Requirements as stated in Specification 4.9.B.2. are implemented.

3. One 125/250 Volt DC Power System (Plant Battery 1A or 1B) Inoperable

From and after the date that one of the two 125/250 volt plant batteries is made or found to be inoperable, continued reactor operation is permissible during the succeeding seven (7) days within electrical safety considerations, provided repair work is initiated immediately to return the failed component to an operable state, Specification 3.5.G. is satisfied,

4.9.B.1. One Startup Auxiliary Transformer (1C or 1D) Inoperable or Only One Offsite Power Source Available (230 kV Transmission Line)

When it is established that one startup auxiliary transformer (1C or 1D) is inoperable or incoming power is available from only one 230 kV offsite transmission line, verify correct breaker alignments and indicated power availability within one hour and at least once per eight hours thereafter, and perform Surveillance Requirement 4.9.A.2.a.1 within 24 hours.

2. One Diesel Generator (1A, 1B, or 1C) Inoperable

When it is established that one diesel generator (1A, 1B, or 1C) is inoperable, verify correct breaker alignments and indicated power availability within one hour and at least once per eight hours thereafter, and perform Surveillance Requirement 4.9.A.2.a.1 within 24 hours, and every 72 hours thereafter.

3. One 125/250 Volt DC Power System (Plant Battery 1A or 1B) Inoperable

When it is established that one of the 125/250 volt DC power systems (plant battery 1A or 1B) is made or found to be inoperable, the pilot cell voltage and specific gravity and the overall battery voltage of the operable plant battery shall be tested daily and determined to be satisfactory.

\*The 1B diesel generator may be inoperable for up to 14 days during the Hatch Unit 2 tenth refueling outage to perform preventative maintenance and modification work.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO.182 TO FACILITY OPERATING LICENSE DPR-57

GEORGIA POWER COMPANY, ET AL.

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-321

1.0 INTRODUCTION

By letter dated July 17, 1992, as supplemented July 30, 1992, Georgia Power Company, et al. (the licensee), proposed license amendments to change the Technical Specifications (TS) for the Edwin I. Hatch Nuclear Plant, Unit 1. The proposed changes are as follows:

a. Proposed Change 1

The licensee requested that the allowed outage time (AOT) of 7 days for diesel generator (DG) 1B (swing diesel) be extended to 14 days. This extension would allow the licensee to perform preventative maintenance and modifications for the purpose of increasing its reliability and, in the future, upgrading the DG rated capacity. This extension will be used only once and only for this purpose.

b. Proposed Change 2

This proposed change will correct two errors in Hatch Unit 1 DG TSs. The original markup of Unit 1 TSs 4.9.B.1 and 4.9.B.2 required performance of "Surveillance Requirement 4.9.A.2.a.1." That change was typed and submitted by the licensee as "Surveillance Requirement 4.9.A.2.a" which included all parts of that TS. This proposed change will correct these errors and make both TSs read "Surveillance Requirement 4.9.A.2.a.1."

This proposed change supersedes the same proposed change (i.e., proposed Change 3) of the licensee's submittal dated July 17, 1992, in response to Generic Letter 91-11, "Resolution of Generic Issue 48, 'LCO's for Class 1E Vital Instrument Buses,' and 49, 'Interlocks and LCO's for Class 1E Tie Breakers' Pursuant to 10 CFR 50.54(f)."

2.0 EVALUATION

During the Fall 1992 refueling outage for Plant Hatch Unit 2, the licensee plans to perform preventative maintenance and modifications to emergency diesel generator (DG) 1B for the purpose of increasing its reliability and, in

the future, upgrading its rated capacity. DG 1B serves as the "swing" diesel in that it is capable of supplying emergency power to either Unit 1 or 2. During the planned work, Hatch Unit 1 is expected to be in power operation. Pursuant to Unit 1 TSs 3.5.G and 3.9.B.2, one DG may be inoperable for up to 7 days whenever the reactor is in the Start and Hot Standby or Run Mode and the reactor water temperature is greater than 212 degrees F. The planned work on DG 1B is expected to take approximately 12 days to complete. Therefore, in order to account for possible contingencies, the licensee is requesting the allowable outage time of 7 days be extended to 14 days for performance of this maintenance and modification work. This extension will be used only once and only for this purpose.

The licensee stated in its submittal that the Individual Plant Examination analysis performed for Plant Hatch, shows an extremely small probability of occurrence for any accident involving loss-of-offsite-power during the proposed seven day extended AOT for the 1B DG. In addition, the reliability of the Hatch DGs has been shown to be very high. The combination of these two factors results in the conclusion that increasing the AOT by seven days for the 1B DG as a one time change will not cause a significant increase in the consequences of an accident previously evaluated.

The NRC staff has reviewed the above change and finds it acceptable.

b. Proposed Change 2

The purpose of the DGs is to provide AC power to equipment necessary for mitigation of accidents involving a loss-of-offsite power (LOSP). The purpose of TSs 4.9.B.1 and 4.9.B.2 is to ensure availability of sufficient AC power sources when one of the normal or emergency AC power sources is determined to be inoperable. This is done by demonstrating operability of the remaining normal and emergency AC power sources at an increased frequency. Performance of Surveillance Requirement 4.9.A.2.a.1 adequately demonstrates DG operability. Therefore, this change will not significantly impact the capability of the DGs to perform their required function of mitigating accidents involving a loss-of-offsite power.

The NRC staff has reviewed the above change and finds it acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards

consideration, and there has been no public comment on such finding (57 FR 37566 dated August 19, 1992). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Jabbour, PDII-3/NRR

Date: September 21, 1992