

June 26, 1986

Docket No. 50-321

Mr. J. T. Beckham, Jr.
Vice President - Nuclear Generation
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued Amendment No. 128 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated July 26, 1985.

The amendment revises the TSs to add a specification and table addressing component cyclic and transient limits.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/S/

George W. Rivenbark, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:

1. Amendment No.128
2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-321
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 128
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated July 26, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

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PDR ADDCK 05000321
P PDR

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 128, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "DMuller for", is written over the typed name.

Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 26, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 128

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment numbers and contain vertical lines indicating the area of change.

Remove

Insert

5.0-2

5.0-2

-

5.0-3

6-20

6-20

3. Fuel Storage

Fuel in the Spent Fuel Pool shall have a maximum fuel loading of 15.2 grams of Uranium-235 per axial centimeter.

5.0.F. Seismic Design

The reactor building and all engineered safeguard systems are designed for the design basis earthquake with a horizontal ground acceleration of 0.15 g. The operating basis earthquake has a horizontal ground acceleration of 0.08 g.

G. Component Cyclic or Transient Limit

The Reactor Pressure Vessel is designed for and shall be maintained within the cyclic or transient limits of Table 5.0.G-1.

H. References

1. FSAR Section 4.2, Reactor Vessel and Appurtenances Mechanical Design
2. FSAR Section 5.2, Primary Containment System
3. FSAR Section 5.3, Secondary Containment System
4. FSAR Section 12.4.4, Governing Codes and Regulations
5. FSAR Section 10.3, Spent Fuel Storage
6. FSAR Section 10.2, New Fuel Storage

TABLE 5.0.G-1COMPONENT CYCLIC OR TRANSIENT LIMITS

<u>COMPONENT</u>	<u>CYCLIC OR TRANSIENT LIMIT</u>	<u>DESIGN CYCLE OR TRANSIENT</u>
A. Reactor Pressure Vessel (Normal, Upset and Testing Conditions)	120 heatup and cooldown 10,000 2,000 80 195 2	Cold Shutdown to 100% Rated Power Reduction to 75% Rated Power Reduction to 50% Rated Power Loss of feedwater heater Scram from 100% Rated Power Improper start of cold recirculation loop

ADMINISTRATIVE CONTROL

RECORD RETENTION (Continued)

- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient or operational cycles for those unit components identified in Table 5.0.G-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the unit staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PRB and SRB.
- l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.15.
- m. Records of analyses required by the Radiological Environmental Monitoring Program.
- n. Records of the service lives of all safety-related hydraulic and mechanical snubbers including the date at which the service life commences and associated installation and maintenance records.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit*. Any individual or group of individuals permitted

*Health Physics personnel, or personnel escorted by Health Physics personnel in accordance with approved emergency procedures, shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 128 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-321

1.0 Introduction and Evaluation

By letter dated July 26, 1985, the licensee, Georgia Power Company (GPC), has proposed to add a specification and a table to address component cyclic and transient limits in Section 5.0 of the Hatch Unit 1 Technical Specifications (TSs). GPC also proposed to correct Section 6.10.2.e in the TSs to reference the new table being added to Section 5.0.

As stated in Attachment 1 to GPC's July 26, 1985 submittal, Hatch Unit 1 TSs currently refer to a non-existent table to identify component cyclic and transient limits. In Attachment 4 to the July 26, 1985 submittal, GPC has provided the new specifications, and a table that lists selected cyclic and transient limits for which the reactor pressure vessel is designed and within which it shall be maintained. The proposed limits conservatively bound (lower bound) the design cyclic and transient limits given in the Hatch Unit 1 updated FSAR (Appendix I), and are consistent with corresponding limits in the Hatch Unit 2 TSs. Furthermore, the proposed limits are consistent with those given in Part 2 of Standard TSs for GE Boiling Water Reactors (BWR/5 - NUREG-0123, Revision 3, issued in Fall 1980).

Based on our review, as discussed above, we have concluded that the proposed changes to the Hatch Unit 1 TSs are acceptable.

2.0 Environmental Considerations

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in the cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

3.0 Conclusion

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
R. Pichumani

Dated: June 26, 1986