September 25, 1986

Docket No. 50-321

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Mr. J. T. Beckham, Jr.
Vice President - Nuclear Generation
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendment No. 129 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated April 15, 1986.

The amendment updates TS Tables 3.7-1 and 3.7-4 to reflect the current plant design with respect to primary containment isolation valves (PCIVs). It also revises Section 3.7.D.1 to require that all PCIVs be operable.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original signed by

George W. Rivenbark, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:
1. Amendment No. 129 to DPR-57
2. Safety Evaluation

cc w/enclosures:
See next page

BWR:PD#2	BWR:PD#2	BWR:PD#2	OELD
SNorris:rs	GRivenbark	DMuller	
9/ 4/86	9/ 9 /86	9/16/86	9/10 /86

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PDR ADOCK 05000321
P PDR

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Georgia Power Company

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Units Nos. 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 129
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al., (the licensee) dated April 15, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

8610010003 860925
PDR ADOCK 05000321
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 129, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 25, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 129

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove	Insert
ix	ix
3.7-16	3.7-16
-----	3.7-16a
3.7-17	3.7-17
3.7-18	3.7-18
-----	3.7-18a
-----	3.7-18b
3.7-19	3.7-19
-----	3.7-19a
3.7-23	3.7-23
3.7-24	3.7-24
3.7-25	3.7-25
3.7-26	3.7-26
-----	3.7-26a
-----	3.7-26b
3.7-27	3.7-27

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TABLE 3.7-1

PRIMARY CONTAINMENT ISOLATION VALVES WHICH
RECEIVE A PRIMARY CONTAINMENT ISOLATION SIGNAL

Isolation Group (b)	Valve Identification (d)	Number of Power Operated Valves Inside	Number of Power Operated Valves Outside	Maximum Operating Time (sec)	Normal Position (a)	Action on Initiating Signal (a)
1	Main steam line (B21-F022 A,B,C,D; B21-F028 A,B,C,D)	4	4	3<T<5	O	GC
1	Main steam line drain (B21-F016, B21-F019)	1	1	15	C	SC
1	Reactor water sample line (B31-F019, B31-F020)	1	1	5	O	GC
2	Drywell purge inlet (T48-F307, T48-F308)		2	5	C	SC
2	Drywell main exhaust (T48-F319, T48-F320)		2	5	C	SC
2	Drywell exhaust valve bypass to standby gas treatment (T48-F341, T48-F340)		2	5	C	SC
2	Drywell nitrogen make-up line (normal operation) (T48-F118A)		1	5	C	SC
2	Suppression chamber purge inlet (T48-F309, T48-F324)		2	5	C	SC
2	Suppression chamber main exhaust (T48-F318, T48-F326)		2	5	C	SC

TABLE 3.7-1 (Cont'd)

PRIMARY CONTAINMENT ISOLATION VALVES WHICH
RECEIVE A PRIMARY CONTAINMENT ISOLATION SIGNAL

Isolation Group (b)	Valve Identification (d)	Number of Power Operated Valves		Maximum Operating Time (sec)	Normal Position (a)	Action on Initiating Signal (a)
		Inside	Outside			
2	H_2-O_2 Analyzer A Torus Sample Line (P33-F006, P33-F014)	0	2	5	C	SC
2	H_2-O_2 Analyzer A Drywell Sample Line (P33-F002, P33-F010)	0	2	5	O	GC
2	H_2-O_2 Analyzer A Return Line (P33-F004, P33-F012)	0	2	5	O	GC
2	H_2-O_2 Analyzer B Torus Sample Line (P33-F007, P33-F015)	0	2	5	O	GC
2	H_2-O_2 Analyzer B Drywell Sample Line (P33-F003, P33-F011)	0	2	5	C	SC
2	H_2-O_2 Analyzer B Return Line (P33-F005, P33-F013)	0	2	5	O	GC
2	Fission Products Monitor Sample Line (D11-F051, D11-F053)	0	2	5	O	GC
2	Fission Products Monitor Return Line (D11-F050, D11-F052)	0	2	5	O	GC

TABLE 3.7-1 (Cont'd)

PRIMARY CONTAINMENT ISOLATION VALVES WHICH
RECEIVE A PRIMARY CONTAINMENT ISOLATION SIGNAL

Isolation Group (b)	Valve Identification (d)	Number of Power Operated Valves Inside	Number of Power Operated Valves Outside	Maximum Operating Time (sec)	Normal Position (a)	Action on Initiating Signal (a)
2	Suppression chamber exhaust valve bypass to standby gas treatment (T48-F339, T48-F338)		2	5	C	SC
2	Suppression chamber nitrogen make-up line (normal operation) (T48-F118B)		1	5	C	SC
2	Drywell and suppression chamber nitrogen supply line (inerting) (T48-F103)		1	5	C	SC
2	Drywell and suppression chamber nitrogen make-up line (normal operation) (T48-F104)		1	5	C	SC
2	Drywell equipment drain sump discharge (G11-F019, G11-F020)		2	15	O	GC
2	Drywell floor drain sump discharge (G11-F003, G11-F004)		2	15	O	GC
2	TIP Guide Tube (C51-J004)		1 each line	NA	C	SC
(c)	Drywell pneumatic system (P70-F002, P70-F003)		2	5	O	GC

TABLE 3.7-1 (Cont'd)

PRIMARY CONTAINMENT ISOLATION VALVES WHICH
RECEIVE A PRIMARY CONTAINMENT ISOLATION SIGNAL

Isolation Group (b)	Valve Identification (d)	Number of Power Operated Valves Inside	Number of Power Operated Valves Outside	Maximum Operating Time (sec)	Normal Position (a)	Action on Initiating Signal (a)
6	RHR reactor shutdown cooling suction (supply) (E11-F008, E11-F009)	1	1	24	C	SC
6	RHR reactor head spray (E11-F022, E11-F023)	1	1	20/12	C	SC
3	HPCI - turbine steam (E41-F002, E41-F003)	1	1	50	O	GC
4	RCIC - turbine steam (E51-F007, E51-F008)	1	1	20	O	GC
5	Reactor water cleanup from recirculation loop (G31-F001, G31-F004)	1	1	30	O	GC
2	Post-accident sampling system supply (B21-F111, B21-F112)		2	5	C	SC
2	Post-accident sampling system return (E41-F122, E41-F121)		2	5	C	SC
2	Core spray test line to suppression pool (E21-F015A,B)		1 each line	50	C	SC

TABLE 3.7-1 (Cont'd)

PRIMARY CONTAINMENT ISOLATION VALVES WHICH RECEIVE A PRIMARY CONTAINMENT ISOLATION SIGNAL

Isolation Group (b)	Valve Identification (d)	Number of Power Operated Valves Inside	Number of Power Operated Valves Outside	Maximum Operating Time (sec)	Normal Position (a)	Action on Initiating Signal (a)
2	HPCI turbine exhaust vacuum breaker (E41-F111, E41-F104)		2	16	O	GC
2	RCIC turbine exhaust vacuum breaker (E51-F105, E51-F104)		2	16	O	GC
2	Torus drainage and purification suction (G51-F011, G51-F012)		2	12	C	SC
2	RHR drywell spray (E11-F016A,B)		1 each line	11	C	SC
2	RHR test line to the suppression pool (E11-F024A,B; E11-F028A,B)		2 each line	110/26	C	SC
2	RHR to torus spray header (E11-F027A,B; E11-F028A,B)		2 each line	10/26	C	SC
2	RHR heat exchanger to the suppression pool (E11-F011A,B; E11-F026A,B)		2 each line	22	C	SC

TABLE 3.7-1 (Cont'd)

PRIMARY CONTAINMENT ISOLATION VALVES WHICH
RECEIVE A PRIMARY CONTAINMENT ISOLATION SIGNAL

Isolation Group (b)	Valve Identification (d)	Number of Power Operated Valves Inside	Number of Power Operated Valves Outside	Maximum Operating Time (sec)	Normal Position (a)	Action on Initiating Signal (a)
2	RHR discharge to radwaste (E11-F049, E11-F040)		2	20/32	C	SC
2	Torus ventilation exhaust (T48-F332A,B; T48-F333A,B)	2	2	5	C	SC
2	Drywell ventilation exhaust (T48-F334A,B; T48-F335A,B)	2	2	5	C	SC
3	HPCI pump minimum flow (E41-F012)		1	11	C	SC
3	HPCI pump suction (E41-F042)		1	84	C	SC
4	RCIC pump minimum flow (E51-F019)		1	11	C	SC
4	RCIC pump suction (E51-F031)		1	33	C	SC

Table 3.7-1

Primary Containment Isolation Valves Which
Receive a Primary Containment Isolation Valve Signal

These notes refer to the lower case letters in parentheses on the previous page.

NOTES:

a. Key: O = Open SC = Stays closed
C = Closed GC = Goes closed

b. Isolation Groupings are as follows:

GROUP 1: The valves in Group 1 are actuated by any one of the following conditions:

1. Reactor vessel water level Low Low Low (Level 1)
2. Main steam line radiation high
3. Main steam line flow high
4. Main steam line tunnel temperature high
5. Main steam line pressure low
6. Condenser vacuum low
7. Turbine building temperature at the steam lines high

GROUP 2: The valves in group 2 are actuated by any one of the following conditions:

1. Reactor vessel water level low (Level 3)
2. Drywell pressure high

GROUP 3: Isolation valves in the high pressure coolant injection (HPCI) system are actuated by any one of the following conditions:

1. HPCI steam line flow high
2. High temperature in the vicinity of the HPCI steam line
3. HPCI steam supply pressure low
4. HPCI turbine exhaust diaphragm pressure
5. Torus room differential temperature high

GROUP 4: Primary Containment Isolation valves in the reactor core isolation cooling (RCIC) system are actuated by any one of the following conditions:

1. RCIC steam line flow high
2. High temperature in the vicinity of the RCIC steam line
3. RCIC steam line pressure low
4. RCIC turbine exhaust diaphragm pressure high
5. Torus room differential temperature high

Table 3.7-1
(Concluded)

Primary Containment Isolation Valves Which
Receive a Primary Containment Isolation Valve Signal

GROUP 5: The valves in Group 5 are actuated by any one of the following conditions:

1. Reactor vessel water level Low Low (Level 2)
2. Reactor water cleanup equipment room temperature high
3. Reactor water cleanup equipment room ventilation differential temperature high
4. Reactor water cleanup system differential flow high
5. Actuation of Standby Liquid Control System - closes outside valve only
6. High temperature following non-regenerative heat exchanger - closes outside valve only

GROUP 6: The valves in Group 6 are actuated by any one of the following conditions:

1. Reactor vessel water level low (Level 3)
2. Reactor vessel steam dome pressure low permissive

c. Requires a Group 2 signal or a Reactor Building ventilation high radiation isolation signal.

d. For redundant lines, only one set of valves is listed. Other sets are identical except for valve numbers, which are included. Valve numbers are listed in order from within primary containment outward for each line.

Table 3.7-4

Primary Containment Testable Isolation Valves

Penetration Number	Valve Designation	Notes
X-7A	B21-F022A & F028A Main Steam Isolation Valves	(1) (2) (3) (5) (9)
X-7B	B21-F022B & F028B Main Steam Isolation Valves	(1) (2) (3) (5) (9)
X-7C	B21-F022C & F028C Main Steam Isolation Valves	(1) (2) (3) (5) (9)
X-7D	B21-F022D & F028D Main Steam Isolation Valves	(1) (2) (3) (5) (9)
X-8	B21-F016 & F019	(1) (2) (4) (5) (9)
X-9A	B21-F010B	(1) (2) (3) (5) (10)
X-9A	B21-F032B	(1) (2) (3) (5) (10)
X-9A	E41-F006	(1) (2) (4) (5) (9)
X-9A	G31-F203	(1) (2) (4) (5) (10)
X-9B	B21-F010A	(1) (2) (3) (5) (10)
X-9B	B21-F032A	(1) (2) (3) (5) (10)
X-9B	E51-F013	(1) (2) (4) (5) (9)
X-9B	G31-F039	(1) (2) (4) (5) (10)
X-10	E51-F007, F008	(1) (2) (4) (5) (9)
X-11	E41-F002 & F003	(1) (2) (4) (5) (9)
X-12	E11-F008 & F009	(1) (2) (4) (5) (9)
X-13A	E11-F015A	(1) (2) (4) (5) (9)
X-13B	E11-F015B	(1) (2) (4) (5) (9)
X-14	G31-F001 & F004	(1) (2) (4) (5) (9)
X-16A	E21-F005A	(1) (2) (4) (5) (9)
X-16B	E21-F005B	(1) (2) (4) (5) (9)
X-17	E11-F022 & F023	(1) (2) (4) (5) (9)

Table 3.7-4
(Continued)

Primary Containment Testable Isolation Valves

Penetration Number	Valve Designation	Notes				
X-18	G11-F019 & F020	(1)	(2)	(4)	(5)	(9)
X-19	G11-F003 & F004	(1)	(2)	(4)	(5)	(9)
X-20	P41-F049 & F050	(1)	(2)	(4)	(5)	(9)
X-21	P51-F513 & F514	(1)	(2)	(4)	(5)	(9)
X-22	P70-F004, F005	(1)	(2)	(4)	(5)	(10)
X-23	P42-F051	(1)	(2)	(4)	(5)	(10)
X-24	P42-F052	(1)	(2)	(4)	(5)	(10)
X-25	T48-F307, F308, F309, F103 & F324	(1)	(2)	(4)	(5)	(9)
X-25	T48-F113	(1)	(2)	(4)	(5)	(14)
X-25	T48-F321	(1)	(2)	(4)	(5)	(14)
X-25	T48-F104	(1)	(2)	(4)	(5)	(14)
X-25	T48-F114, F118A, F118B & F322	(1)	(2)	(4)	(5)	(10)
X-26	T48-F319 & F320	(1)	(2)	(4)	(5)	(9)
X-26	T48-F334A	(1)	(2)	(4)	(5)	(14)
X-26	T48-F334B	(1)	(2)	(4)	(5)	(14)
X-26	T48-F335A & F335B	(1)	(2)	(4)	(5)	(10)
X-26	T48-F340 & F341	(1)	(2)	(4)	(5)	(9)
X-26	P33-F002	(1)	(2)	(4)	(5)	(10)
X-26	P33-F010	(1)	(2)	(4)	(5)	(14)
X-27A	D11-F051 & F053	(1)	(2)	(4)	(5)	(10)
X-27F	P70-F066 & F067	(1)	(2)	(4)	(5)	(10)

Table 3.7-4
(Continued)

Primary Containment Testable Isolation Valves

Penetration Number	Valve Designation	Notes
X-28F	P33-F003 & F011	(1) (2) (4) (5) (9)
X-28A	B31-F019 & F020	(1) (2) (4) (5) (9)
X-31D	P33-F004 & F012	(1) (2) (4) (5) (9)
X-31F	B31-F013A & F017A	(1) (2) (4) (5) (10)
X-33C	D11-F050 & F052	(1) (2) (4) (5) (10)
X-34E	P33-F005 & F013	(1) (2) (4) (5) (9)
X-35A	C51-Ball Valve	(1) (2) (4) (5) (10)
X-35B	C51-Ball Valve	(1) (2) (4) (5) (10)
X-35C	C51-Ball Valve	(1) (2) (4) (5) (10)
X-35D	C51-Ball Valve	(1) (2) (4) (5) (10)
X-35E	C51-Nitrogen Inerting	(1) (2) (4) (5) (10)
X-39A	E11-F016A	(1) (2) (4) (5) (9)
X-39B	E11-F016B	(1) (2) (4) (5) (9)
X-40C	P70-F002 & F003	(1) (2) (4) (5) (9)
X-42	C41-F006	(1) (2) (4) (5) (10)
X-42	C41-F007	(1) (2) (4) (5) (10)
X-45F	T23-F004 & F005	(1) (2) (4) (5) (9)
X-46	P21-F353 & F420	(1) (2) (4) (5) (9)
X-52B	B21-F111 & F112	(1) (2) (4) (5) (10) (11)
X-59A	B31-F013B & F017B	(1) (2) (4) (5) (10)
X-203	E51-F003 & F031	(1) (2) (4) (5) (9)

Table 3.7-4
(Continued)

Primary Containment Testable Isolation Valves

Penetration Number	Valve Designation	Notes
X-204A	E11-F004A	(1) (2) (5) (9) (12)
X-204A	E11-F030A	(1) (2) (4) (5) (11)
X-204B	E11-F004B	(1) (2) (5) (9) (12)
X-204B	E11-F030B	(1) (2) (4) (5) (11)
X-204C	E11-F004C	(1) (2) (5) (9) (12)
X-204C	E11-F030C	(1) (2) (4) (5) (11)
X-204D	E11-F004D	(1) (2) (5) (9) (12)
X-204D	E11-F030D	(1) (2) (4) (5) (11)
X-205	T48-F310 & F328A	(1) (2) (4) (5) (9)
X-205	T48-F115 & F116	(1) (2) (4) (5) (9)
X-205	T48-F311 & F328B	(1) (2) (4) (5) (9)
X-205	T48-F325 & F327	(1) (2) (4) (5) (9)
X-206A	E41-F121 & F122	(1) (2) (4) (5) (9)
X-207	E41-F042 & F051	(1) (2) (4) (5) (9)
X-208A	E21-F001A	(1) (2) (4) (5) (9)
X-208B	E21-F001B	(1) (2) (4) (5) (9)
X-210A	E11-F007A	(1) (2) (4) (5) (11)
X-210A	E21-F044A	(1) (2) (4) (5) (11)
X-210A	E51-F019 & F021	(1) (2) (4) (5) (9)
X-210A	E21-F031A	(1) (2) (4) (5) (11)
X-210A	E11-F024A, F027A & F028A	(1) (2) (4) (5) (9)

Amendment No. ~~33~~, ~~83~~, ~~89~~, 129

Table 3.7-4
(Continued)

Primary Containment Testable Isolation Valves

Penetration Number	Valve Designation	Notes
X-210A	E11-F011A & F026A	(1) (2) (4) (5) (9)
X-210A	E11-F025A, F029, F055A, F103A	(1) (2) (4) (5) (11)
X-210A	E21-F015A	(1) (2) (4) (5) (11)
X-210A	E11-Thermal Relief Valve	(1) (2) (4) (5) (11)
X-210B	E11-F007B	(1) (2) (4) (5) (11)
X-210B	E21-F044B	(1) (2) (4) (5) (11)
X-210B	E11-F097	(1) (2) (4) (5) (11)
X-210B	E41-F012 & F046	(1) (2) (4) (5) (9)
X-210B	E21-F031B	(1) (2) (4) (5) (11)
X-210B	E11-F024B, F027B & F028B	(1) (2) (4) (5) (9)
X-210B	E11-F011B & F026B	(1) (2) (4) (5) (9)
X-210B	E11-F025B, F055B, F103B	(1) (2) (4) (5) (11)
X-210B	E21-F015B	(1) (2) (4) (5) (11)
X-210B	E11-Thermal Relief Valve	(1) (2) (4) (5) (11)
X-212	E51-F001 & F040	(1) (2) (4) (5) (9)
X-213	E51-F002 & F028	(1) (2) (4) (5) (9)
X-214	E41-F021 & F049	(1) (2) (4) (5) (9)
X-215	E41-F022 & F040	(1) (2) (4) (5) (9)
X-217	P33-F007 & F015	(1) (2) (4) (5) (9)
X-218A	G51-F002	(1) (2) (4) (5) (10)
X-218A	G51-D001	(1) (2) (4) (5) (13)

Table 3.7-4
(Continued)

Primary Containment Testable Isolation Valves

Penetration Number	Valve Designation	Notes
X-218A	G51-F011	(1) (2) (4) (5) (10)
X-218A	G51-F012	(1) (2) (4) (5) (10)
X-220	P33-F006	(1) (2) (4) (5) (14)
X-220	P33-F014	(1) (2) (4) (5) (10)
X-220	T48-F318 & F326	(1) (2) (4) (5) (9)
X-220	T48-F332A	(1) (2) (4) (5) (14)
X-220	T48-F332B	(1) (2) (4) (5) (14)
X-220	T48-F333A & F333B	(1) (2) (4) (5) (10)
X-220	T48-F338 & F339	(1) (2) (4) (5) (9)
X-221C	E51-F104 & F105	(1) (2) (4) (5) (9)
X-222A	E41-F111 & F104	(1) (2) (4) (5) (9)
X-223A	T48-Air Cylinder	(1) (2) (4) (5) (11)
X-223A	T48-F342G-L	(1) (2) (4) (5) (11)
X-223B	T48-Air Cylinder	(1) (2) (4) (5) (11)
X-223B	T48-F342A-F	(1) (2) (4) (5) (11)

Notes for Tables 3.7-2 through 3.7-4

(1) Test duration for all valves and penetrations listed will generally exceed one hour.

(2) Test pressures are at least 59 psig for all valves and penetrations except MSIV's which are tested at 28 psig.

(3) MSIV acceptable leakage limit is 11.5 scfh/valve of air.

(4) The total acceptable leakage for all valves and penetrations other than the MSIV's is 0.6 L_a.

(5) Local leak tests on all testable isolation valves shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.

(6) Local leak tests on all testable penetrations shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.

(7) The personnel air lock shall be tested at intervals not to exceed 6 months.

(8) The personnel air lock door seals are tested at 10 psig after each opening.

(9) Identifies isolation valves that are tested by applying pressure between the inboard and outboard isolation valves. Inboard valve is not tested in the direction required for isolation but will have equivalent or more conservative leakage results.

(10) Identifies isolation valves that are tested by applying pressure between the isolation valve and a manually operated valve such that the isolation valve is tested in the direction required for isolation.

(11) Identifies isolation valves that are tested by applying pressure between the isolation valves and other system valves. Isolation valves not tested in the direction required for isolation will have equivalent or more conservative results.

(12) The RHR system remains water filled post-LOCA. Isolation valves are tested with water.

(13) Identified blind flange that is tested by applying pressure between the flange and a manually operated valve such that the flange is tested in the direction required for isolation.

(14) Identifies isolation valves that are tested by applying pressure between the inboard and outboard isolation valves.

Amendment No. ~~53~~, ~~85~~, ~~89~~, 129

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 129 TO

FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-321

1.0 INTRODUCTION

By letter dated March 3, 1986, Georgia Power Company, et al. (the licensee) proposed deletion of Table 3.7-1, "Primary Containment Isolation Valves, " from the Hatch Unit 1 Technical Specifications. The table would be replaced with references to valve listings in applicable plant procedures. Removal of component lists from Technical Specifications has been proposed as a generic change to the Standard Technical Specifications (STS) by the Atomic Industrial Forum and Owners Groups. Due to the significance of this proposal, the staft is in the process of developing generic guidelines for implementing this recommendation and updating the STS. Since this effort is not completed, the review of the proposed deletion of Table 3.7-1 will be delayed until completion of the generic effort. Based on the staff's position, the licensee, in a letter dated April 15, 1986 for Hatch Unit 1, proposed changes which would update rather than delete valve listings in Table 3.7-1 and Table 3.7-4, "Primary Containment Testable Isolation Valves." The following is the staff evaluation of the proposed TS updates.

2.0 EVALUATION

Proposed Change 1:

To clarify its intent, the licensee proposes to change the title of Table 3.7-1 in Technical Specification for Hatch Unit 1, page 3.7-16, from Primary Containment Isolation Valves to Primary Containment Isolation Valves Which Receive A Primary Containment Isolation Signal. This table is needed for the implementation of Technical Specification 4.7.D.1.a which requires the surveillance testing of operable isolation valves that are power operated and automatically initiated to be tested for simulated automatic initiation and the closure times specified in Table 3.7-1. The proposed Table 3.7-1 contains only power operated Primary Containment Isolation Valves (PCIVs) which receive a containment isolation signal. This proposed change provides consistency within the Technical Specification and is, therefore, acceptable.

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Proposed Change 2:

This proposed change would delete reference to Table 3.7-1 from the Hatch Unit 1 Technical Specification 3.7.D.1. The present Technical Specification requires operability of all PCIVs listed in Table 3.7-1. Deletion of the reference to Table 3.7-1, as proposed, requires that all PCIVs shall be operable, including automatic PCIVs in Table 3.7-1 and non-automatic PCIVs. Since this proposed change is conservative, it is acceptable.

Proposed Change 3:

This proposed change is to delete check valves from Table 3.7-1 of the Hatch Unit 1 Technical Specification and add nine of them to Table 3.7-4. The remaining two check valves are not Appendix J tested and are not included in Table 3.7-4. Since the proposed Table 3.7-1 is for surveillance of automatic PCIVs which receive a primary containment isolation signal and the check valves are not in this category, it is acceptable to delete the check valves from Table 3.7-1.

Proposed Change 4:

The proposed change is to include additional automatic PCIVs in Table 3.7-1. Some of the additional valves have been included due to plant design changes, while others were not previously listed in Table 3.7-1 although they met the criteria for inclusion. Since this proposed change is conservative and will result in additional Technical Specification requirements, the proposed inclusion of additional valves in Table 3.7-1 is acceptable.

Proposed Change 5:

The proposed change is editorial in nature. It corrects the Technical Specification for Hatch Unit 1 to accurately reflect actual plant conditions and does not represent a change in PCIV operation or configuration. Since this change upgrades the accuracy of the Technical Specification, it is acceptable.

Proposed Change 6:

This proposed change provides a listing in Table 3.7-4 of the Hatch Unit 1 Technical Specification of PCIVs subject to leak rate testing under the provisions of 10 CFR 50, Appendix J. This proposed table differs from the present table by including corrections to erroneous listings, additions and deletions to valve listings, as well as changes to descriptive information and notes relevant to this table. This proposed change is acceptable since it corrects Table 3.7-4 to be consistent with a listing of PCIVs that require testing under 10 CFR 50 Appendix J.

Proposed Change 7:

This change provides a note to Table 3.7-4 to indicate the method of testing. Since the method of testing meets applicable criteria per 10 CFR 50, Appendix J, the proposed change is acceptable.

Proposed Change 8:

The proposed change adds 72 valves to Table 3.7-4 which are currently listed in plant leak rate test procedures. This proposed change is acceptable since it is conservative and provides consistency between the technical Specifications and plant leak rate test procedures.

Proposed Change 9:

This proposed change deletes 26 valves presently listed in Table 3.7-4 which do not require leak rate testing because the valves do not serve as the primary containment isolation boundary. This proposed change is acceptable since the deleted valves are not PCIVs and are not part of the primary isolation boundary.

On the basis of our evaluation, we conclude that the proposed changes which clarify and correct the Hatch Unit 1 Technical Specifications with respect to containment isolation valves are acceptable. While acceptable, the staff notes that the relevant portions of the Hatch Unit 1 Technical Specifications differ from Unit 2. Since the licensee has proposed that Hatch be the lead plant for technical specifications improvement, these differences should be resolved during this review.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite; and that there should be no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

On the basis of the considerations discussed above, the staff has concluded that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations.

Principal Contributor: F. Witt

Dated: September 25, 1986