March 18, 1991

Docket Nos. 50-321
and 50-366

DISTRIBUTION
See Next Page

Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P.O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT NO. 171 TO FACILITY OPERATING LICENSE DPR-57 AND AMENDMENT NO. 109 TO FACILITY OPERATING LICENSE NPF-5 - EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2 (TACs 77544/77545)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 171 to Facility Operating License No. DPR-57 and Amendment No. 109 to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) and the Environmental Technical Specifications (ETSs) in response to your application dated August 20, 1990.

The amendments make a number of editorial changes to the TSs and ETSs for Units 1 and 2.

A copy of the related Safety Evaluation is also enclosed. Notice of issuance of the amendments will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 171to DPR-57
2. Amendment No. 109to NPF-5
3. Safety Evaluation

cc w/enclosures:
See next page

OFC	LA:PDII-3	PM:PDII-3	PM:PDII-3	BC: OTSB	OGC	D:PDII-3
NAME	RIngram	FRinaldi:sw	KJabbour	JCalvo	J. Hull	DMatthews
DATE	2/19/91	2/20/91	2/26/91	3/15/91	3/6/91	3/18/91

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DATED: March 18, 1991

AMENDMENT NO.171 TO FACILITY OPERATING LICENSE DPR-57 - Edwin I. Hatch Nuclear Plant, Unit 1

AMENDMENT NO.109 TO FACILITY OPERATING LICENSE NPF-5 - Edwin I. Hatch Nuclear Plant, Unit 2

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Mr. W. G. Hairston, III
Georgia Power Company

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Units Nos. 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 171
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility), Facility Operating License No. DPR-57 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated August 20, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 171, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Darl Hood

for David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: March 18, 1991

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 109
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility), Facility Operating License No. NPF-5 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated August 20, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 109, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Carl Hood for

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: March 18, 1991

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 171 TO FACILITY OPERATING LICENSE DPR-57

AND AMENDMENT NO. 109 TO FACILITY OPERATING LICENSE NPF-5

GEORGIA POWER COMPANY, ET AL.

EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-321 AND 50-366

1.0 INTRODUCTION

By letter dated August 20, 1990, Georgia Power Company, et al., the licensee for the Edwin I. Hatch Nuclear Plant, proposed changes to the Technical Specifications (TSs) and Environmental Technical Specifications (ETSs) for Units 1 and 2. The proposed changes would modify the TSs and ETSs for Units 1 and 2 as follows:

1. Change Unit 2 TS Table 3.2.2-1 to replace the master parts list (MPL) for Items 1.f and 4.g.

2. (a) Change Unit 1 TS Table 4.2-2 to correct the spelling of "Emergency" under Reference No. 8.

   (b) Change Unit 2 TS Table 3.3.6.1-1 to correct the measurement range for the off-gas post-treatment monitors, and TS 4.6.6.1.1.d.3 to incorporate a plus/minus sign for the range of heat dissipated by the standby gas treatment subsystem heaters.

   (c) Delete the list of Figures in the ETS Table of Contents for Units 1 and 2.

3. Change Unit 2 TS 3.8.2.5 and TS Table 3.8.2.6-1 to make editorial corrections.

2.0 EVALUATION

2.1 Proposed Change 1

This proposed change to Unit 2 TS Table 3.3.2-1, "Isolation Actuation Instrumentation," would make revisions that are editorial in nature. It would correct the MPL numbers under Item 1.f, "Turbine Building Area Temperature - High," and Item 4.g, "Supression Pool Area Temperature Timer Relays." The specific MPL changes for 1.f would replace designators 2U61-R001 through R004 to 2U61-P001 through P004 and for 4.g from 2E41 to 2E51. The proposed change to 4.g corrects a typographical error, while the proposed change to item 1.f reflects the change in indicator panels. These changes are acceptable because they do not affect plant operation, and the safety-related equipment is not changed.

2.2 Proposed Change 2

These proposed changes to the Unit 1 and Unit 2 TSs and ETSs would make revisions that are editorial in nature. The specific administrative changes proposed to provide clarity are as follows:

- Unit 1 Table 4.2-2 - Correct the spelling of "Emergency" under Reference No. 8.

- Unit 2 Table 3.3.6.1-1 - Correct the measurement range of the off-gas post-treatment monitors from 10^{-1} to 10^{-6} counts per second (cps) to 10^{-1} to 10^{6} cps.

- Unit 2 Surveillance Requirement 4.6.6.1.1.d.3 - Correct the range of heat dissipated by the standby gas treatment subsystem heaters from 18.5 + 1.5 kW to 18.5 ± 1.5 kW.

- Units 1 and 2 ETS Table of Contents - Delete the list of Figures in the ETS Table of Contents since Figure 5.2-1 is the only figure listed and it was deleted in Amendments 145 and 80 for Units 1 and 2, respectively.

These changes are found acceptable because they are considered editorial changes that do not affect plant operation or safety and involve no physical changes to the plant.

2.3 Proposed Change 3

This proposed change to the Unit 2 TSs would make revisions that are editorial in nature. The specific administrative changes proposed to provide clarity are as follows:

- TS 3.8.2.5 Items a, b, c and d - Correct the word "Circuit" to "Breaker," and for Item c, the breaker numbers 26 and 32 would change to 28 and 34, respectively. Also, for Item d, the term, "Compartment" would change to "Frame."

- Table 3.8.2.6-1 - Relocate Items c.3 and c.4 to the next page of the table and change the designation for the drywell cooling unit breakers from thermal magnetic breakers to magnetic only breakers. Further, the sequential lettering for Type 6 and Type 7 devices would be corrected, and throughout the table, changes would be made to reflect current equipment designation and to make the terminology consistent with other design documents.

These changes are found acceptable because they are considered editorial changes that do not affect plant operation or safety and involve no physical changes to the plant.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes in requirements with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

4.0 CONCLUSION

The Commission's proposed determination that the amendments involve no significant hazards consideration was published in the Federal Register on October 31, 1990 (55 FR 45881), and the State of Georgia was consulted. No public comments were received, and the State of Georgia did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Kahtan N. Jabbour, PDII-3/DRP-I/II
Frank Rinaldi, PDII-3, DRP-I/II, NRR

Dated: March 18, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 171

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of Appendix "A" with the enclosed page and remove one page from Appendix "B" Technical Specifications. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Appendix A

	Remove Page	Insert Page
Unit 1	3.2-27	3.2-27

Appendix B

	iii (ETS)	-----

Table 4.2-2

Check, Functional Test, and Calibration Minimum Frequency for Instrumentation Which Initiates or Controls HPCI

Ref. No. (a)	Instrument	Instrument Check Minimum Frequency	Instrument Functional Test Minimum Frequency (b)	Instrument Calibration Minimum Frequency (c)
1	Reactor Vessel Water Level (Level 2)	Once/shift	Once/month	Once/operating cycle
2	Drywell Pressure	Once/shift	Once/month	Once/operating cycle
3	HPCI Turbine Overspeed	None	N/A	Once/operating cycle
4	HPCI Turbine Exhaust Pressure	Once/shift	Once/month	Once/operating cycle
5	HPCI Pump Suction Pressure	Once/shift	Once/month	Once/operating cycle
6	Reactor Vessel Water Level (Level 8)	Once/shift	Once/month	Once/operating cycle
7	HPCI Pump Discharge Flow	Once/shift	Once/month	Once/operating cycle
8	HPCI Emergency Area Cooler Ambient Temperature	Once/shift	Once/month	Once/operating cycle
9	HPCI Steam Supply Pressure	Once/shift	Once/month	Once/operating cycle

ATTACHMENT TO LICENSE AMENDMENT NO. 109

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of Appendix "A" with the enclosed pages and remove one page from Appendix "B" Technical Specifications. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Appendix A

	Remove Page	Insert Page
Unit 2	3/4 3-11	3/4 3-11
	3-13	3-13
	3-44	3-44
	6-41	6-41
	8-17	8-17
	8-20	8-20
	8-21	8-21
	8-22	8-22
	8-23	8-23

Appendix B

	iii (ETS)	-----

TABLE 3.3.2-1

ISOLATION ACTUATION INSTRUMENTATION

TRIP FUNCTION	VALVE GROUPS OPERATED BY SIGNAL(a)	MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)	APPLICABLE OPERATIONAL CONDITION	ACTION
1. PRIMARY CONTAINMENT ISOLATION				
a. Reactor Vessel Water Level				
1. Low (Level 3) (2B21-N680 A, B, C, D)	2, 6, 10, 11, 12	2	1, 2, 3	20
2. Low-Low (Level 2) (2B21-N682 A, B, C, D)	5, *	2	1, 2, 3	20
3. Low-Low-Low (Level 1) (2B21-N681 A, B, C, D)	1	2	1, 2, 3	20
b. Drywell Pressure - High (2C71-N650 A, B, C, D)	2, 6, 7, 10, 12, *	2	1, 2, 3	20
c. Main Steam Line				
1. Radiation - High (2D11-K603 A, B, C, D)	1, 12, (d)	2	1, 2, 3, (d)	21
2. Pressure - Low (2B21-N015 A, B, C, D)	1	2	1	22
3. Flow - High (2B21-N686 A, B, C, D) (2B21-N687 A, B, C, D) (2B21-N688 A, B, C, D) (2B21-N689 A, B, C, D)	1,	2/line	1, 2, 3	21
d. Main Steam Line Tunnel Temperature - High (2B21-N623 A, B, C, D) (2B21-N624 A, B, C, D) (2B21-N625 A, B, C, D) (2B21-N626 A, B, C, D)	1	2/line"	1, 2, 3	21
e. Condenser Vacuum - Low (2B21-N056 A, B, C, D)	1	2	1, 2,', 3'	23
f. Turbine Building Area Temperature - High (2U61-P001, 2U61-P002, 2U61-P003, 2U61-P004)	1	2"	1, 2, 3	21
g. Drywell Radiation - High (2D11-K621 A, B)	(')	1	1, 2, 3	29

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

TRIP FUNCTION	VALVE GROUPS OPERATED BY SIGNAL(a)	MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)	APPLICABLE OPERATIONAL CONDITION	ACTION
4. HIGH PRESSURE COOLANT INJECTION SYSTEM ISOLATION				
a. HPCI Steam Line Flow - High (2E41-N657 A,B)	3	1	1, 2, 3	26
b. HPCI Steam Supply Pressure - Low (2E41-N658 A,B,C,D)	3, 8	2	1, 2, 3	26
c. HPCI Turbine Exhaust Diaphragm Pressure - High (2E41-N655 A,B,C,D)	3	2	1, 2, 3	26
d. HPCI Pipe Penetration Room Temperature - High (2E41-N671 A, B)	3	1	1, 2, 3	26
e. Suppression Pool Area Ambient Temperature-High (2E51-N666 C, D)	3	1	1, 2, 3	26
f. Suppression Pool Area W Temp.-High (2E51-N665 C, D; 2E51-N663 C, D; 2E51-N664 C, D)	3	1	1, 2, 3	26
g. Suppression Pool Area Temperature Timer Relays (2E51-M603 A, B)	3	1	1, 2, 3	26
h. Emergency Area Cooler Temperature-High (2E41-N670 A, B)	3(d)	1	1, 2, 3	26
i. Drywell Pressure-High (2E11-N694 C, D)	8	1	1, 2, 3	26
j. Logic Power Monitor (2E41-K1)	NA(e)	1	1, 2, 3	27

TABLE 3.3.6.1-1

RADIATION MONITORING INSTRUMENTATION

INSTRUMENTATION	MINIMUM CHANNELS OPERABLE	APPLICABLE OPERATIONAL CONDITIONS	ALARM/TRIP SETPOINT	MEASUREMENT RANGE	ACTION
1. Off-Gas Post-Treatment Monitors (2D11-K615 A, B)	2	1, 2	(a)	10^{-1} to 10^{6} cps	50
2. Control Room Intake Monitors (1Z41-R615 A, B)	2	1, 2, 3, 4, 5	1 mr/hr	0.01 to 100 mr/hr	51

(a) Value not to exceed the equivalent of the stack release limit indicated in the Environmental Technical Specifications.

SURVEILLANCE REQUIREMENTS (Continued)

3. Verifying a system flow rate of 4000 +0, -1000 cfm during system operation when tested in accordance with ANSI N510-1975.

c. After every 720 hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 1, July 1976, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 1, July 1976.

d. At least once per 18 months by:

1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is < 6 inches Water Gauge while operating the filter train at a flow rate of 4000 +0, -1000 cfm.

2. Verifying that the filter train starts and isolation dampers open on each of the following test signals:

a. Drywell pressure-high,

b. High radiation on the;

1) Refueling floor,

2) Reactor building.

c. Reactor Vessel Water Level-Low Low (Level 2).

3. Verifying that the heaters dissipate 18.5 ± 1.5 KW when tested in accordance with ANSI N510-1975.

ELECTRICAL POWER SYSTEMS

A.C. CIRCUITS INSIDE PRIMARY CONTAINMENT

LIMITING CONDITIONS FOR OPERATION

3.8.2.5 The following A.C. circuits inside primary containment shall be de-energized*:

a. Breaker Numbers 2, 4, 6, 8, 10, 12, 14, 40 and 42 in panel 2T51-S003,

b. Breaker Numbers 2, 4, 6, 8, 10, 12, 40 and 42 in panel 2T51-S004,

c. Breaker Numbers 28 and 34 in panel 2R25-S105, and

d. Frame 1EL on MCC 2R24-S014.

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

With any of the above required circuits energized, trip the associated circuit breaker(s) in the specified panel within 1 hour.

SURVEILLANCE REQUIREMENTS

4.8.2.5 Each of the above required A.C. circuits shall be determined to be de-energized at least once per 24 hours by verifying that the associated circuit breakers in the specified panels are in the tripped condition.

*Except during entry into the drywell.

TABLE 3.8.2.6-1

PRIMARY CONTAINMENT PENETRATION CONDUCTOR OVERCURRENT PROTECTIVE DEVICES

DEVICE NUMBER AND LOCATION*	SYSTEM/COMPONENT POWERED
a. Type 1:	
1. 4KV CKT BRKR 2R22-S001, FR 5	REACTOR RECIRC WTR PUMP MG SET 2A DRIVE MOTOR 2B31-S001A
2. 4KV CKT BRKR 2R22-S008, FR 2	RECIRC A PUMP MOTOR 2B31-C001A
3. 4KV CKT BRKR 2R22-S002, FR 5	REACTOR RECIRC WTR PUMP MG SET 2B DRIVE MOTOR 2B31-S001B
4. 4KV CKT BRKR 2R22-S009, FR 2	RECIRC B PUMP MOTOR 2B31-C001B
b. Type 2:	
1. 600 VAC, MCB, TM 2R24-S012, FR 2DL	DRYWELL AREA COOLING UNIT 2T47-B009B
2. 600 VAC, MCB, TM 2R24-S012, FR 2DR	DRYWELL AREA COOLING UNIT 2T47-B009B
3. 600 VAC, MCB, TM 2R24-S012, FR 3FL	DRYWELL AREA COOLING UNIT 2T47-B008B
4. 600 VAC, MCB, TM 2R24-S012, FR 3FR	DRYWELL AREA COOLING UNIT 2T47-B008B
5. 600 VAC, MCB, TM 2R24-S011, FR 1DL	DRYWELL AREA COOLING UNIT 2T47-B008A
6. 600 VAC, MCB, TM 2R24-S011, FR 1DR	DRYWELL AREA COOLING UNIT 2T47-B008A
7. 600 VAC, MCB, TM 2R24-S011, FR 20AR	DRYWELL AREA COOLING UNIT 2T47-B009A
8. 600 VAC, MCB, TM 2R24-S011, FR 20E	DRYWELL AREA COOLING UNIT 2T47-B009A

*MCB - molded case circuit breaker
MO - magnetic only
TM - thermal magnetic

TABLE 3.8.2.6-1 (Continued)

PRIMARY CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

DEVICE NUMBER AND LOCATION*	SYSTEM/COMPONENT POWERED
c. Type 3:	
1. 600 VAC, MCB, TM 2R24-S014, FR 5E	RECIRC PUMP MOTOR HEATER 2B31-C001B
2. 600 VAC, MCB, TM 2R24-S013, FR 5B	RECIRC PUMP MOTOR HEATER 2B31-C001A
d. Type 4:	
1. 120 VAC, MCB, TM 2R25-S102, BRKR 10	CABLES BHE808M01 AND BHE808M02
2. 120 VAC, MCB, TM 2R25-S101, BRKR 10	CABLES BGE708M01 AND BGE708M02
e. Type 5:	
1. 600 VAC, MCB, MO 2R24-S014, FR 2A	DRYWELL EQUIP DR SUMP PUMP DISCH MOV 2G11-F018
2. 600 VAC, MCB, MO 2R24-S014, FR 6C	DRYWELL EQUIP DR SUMP PUMP DISCH MOV 2G11-F015
3. 600 VAC, MCB, MO 2R24-S012B, FR 4A	RCIC STEAM SUPPLY ISOLATION MOV 2E51-F007
4. 600 VAC, MCB, MO 2R24-S011, FR 9A	REACTOR HEAD SPRAY VALVE MOV 2E11-F022
5. 600 VAC, MCB, MO 2R24-S011A, FR 4A	HPCI INBOARD STEAM ISOLATION MOV 2E41-F002
6. 600 VAC, MCB, MO 2R24-S011, FR 14C	RWCU INBOARD ISOLATION VALVE MOV 2G31-F001
7. 600 VAC, MCB, MO 2R24-S011, FR 15B	MAIN STEAM LINE DRAIN VALVE MOV 2B21-F016

*MCB - molded case circuit breaker
MO - magnetic only
TM - thermal magnetic

TABLE 3.8.2.6-1 (Continued)

PRIMARY CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

DEVICE NUMBER AND LOCATION*	SYSTEM/COMPONENT POWERED
f. Type 6:	
1. 600 VAC, MCB, MO 2R24-S018A, FR 2A	RECIRC PUMP SUCTION VALVE MOV 2B31-F023A
2. 600 VAC, MCB, MO 2R24-S018A, FR 2B	RECIRC PUMP DISCHARGE VALVE MOV 2B31-F031A
3. 600 VAC, MCB, MO 2R24-S018B, FR 3A	RECIRC PUMP SUCTION VALVE MOV 2B31-F023B
4. 600 VAC, MCB, MO 2R24-S018B, FR 3B	RECIRC PUMP DISCHARGE VALVE MOV 2B31-F031B
5 600 VAC, MCB, MO 2R24-S014, FR 1B	DRYWELL EQUIP DRAIN SUMP PUMP B 2G11-C006B
6. 600 VAC, MCB, MO 2R24-S014, FR 7D	DRYWELL FLOOR DRAIN SUMP PUMP B 2G11-C001B
7. 600 VAC, MCB, MO 2R24-S013, FR 4A	DRYWELL FLOOR DRAIN SUMP PUMP 1A 2G11-C001A
8. 600 VAC, MCB, MO 2R24-S013, FR 4B	DRYWELL EQUIP DRAIN SUMP PUMP A 2G11-C006A
9. 600 VAC, MCB, MO 2R24-S012, FR 18B	DRYWELL AREA COOLING UNIT 2T47-B007B
10. 600 VAC, MCB, MO 2R24-S012, FR 19A	DRYWELL RETURN AIR FAN 2T47-C001B
11. 600 VAC, MCB, MO 2R24-S011, FR 6C	RHR SHUTDOWN COOLING SUCTION VALVE MOV 2E11-F009
12. 600 VAC, MCB, MO 2R24-S011, FR 18A	DRYWELL AREA COOLING UNIT 2T47-B007A

*MCB - molded case circuit breaker
MO - magnetic only
TM - thermal magnetic

TABLE 3.8.2.6-1 (Continued)

PRIMARY CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

DEVICE NUMBER AND LOCATION*	SYSTEM/COMPONENT POWERED
13. 600 VAC, MCB, MO 2R24-S011, FR 18C	DRYWELL RETURN AIR FAN 2T47-C001A
14. 600 VAC, MCB, MO 2R24-S013, FR 3B	DRYWELL COOLING UNIT 2T47-B010A
15. 600 VAC, MCB, MO 2R24-S014, FR 8A	DRYWELL COOLING UNIT 2T47-B010B
16. 600 VAC, MCB, TM 2R24-S013, FR 3B	DRYWELL COOLING UNIT 2T47-B010A
17. 600 VAC, MCB TM 2R24-S014, FR 8A	DRYWELL COOLING UNIT 2T47-B010B
g. Type 7:	
1. 208 VAC, MCB, MO 2R24-S013, FR 11D	DRYWELL CHEMICAL DRAIN SUMP PUMP 2G11-C101
2. 208 VAC, MCB, MO 2R24-S012, FR 23C	DRYWELL RETURN AIR FAN 2T47-C002B
3. 208 VAC, MCB, MO 2R24-S011, FR 22C	DRYWELL RETURN AIR FAN 2T47-C002A

*MCB - molded case circuit breaker
MO - magnetic only
TM - thermal magnetic