June 6, 1989

Docket Nos.: 50-321
and 50-366

Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT NO. 163 TO FACILITY OPERATION LICENSE DPR-57 AND AMENDMENT NO.100 TO FACILITY OPERATING LICENSE NPF-5 - EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2 (TACS 61281/61282)

The Commission has issued the enclosed Amendment No. 163 to Facility Operating License DPR-57 and Amendment No. 100 to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated March 27, 1986, as supplemented April 22 and December 27, 1988.

The amendments change the Unit 1 reactor protection system (RPS) and control rod block surveillance requirements to provide for consistency with the Unit 2 TS and change the RPS functional test frequencies and equipment outage times for both units.

A copy of our Safety Evaluation is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Jon B. Hopkins, Project Manager

for Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 163 to DPR-57
2. Amendment No. 100 to NPF-5
3. Safety Evaluation
4. F.R. Notice

cc w/ enclosures:
See next page

PDII-3	PDII-3	PDII-3
MRood	LCrocker:ls	DMatthews
04/14/89	for 04/14/89	~~04/ /89~~ 05/31/89

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101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

DATED June 6, 1989

AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE NPF-5, EDWIN I. HATCH, UNIT 2
AMENDMENT NO. 100 TO FACILITY OPERATING LICENSE DPR-57, EDWIN I. HATCH, UNIT 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated March 27, 1986, as supplemented Arpil 22 and December 27, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Lawrence P. Crocker

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 6, 1989

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:
David B. Matthews
David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical Specifications

Date of Issuance: June 6, 1989

OFFICIAL RECORD COPY

LA:PDII-3	PM:PDII-3	DEST:SICB	OGC-WF	D:PDII-3
MRood	LCrocker:	SNewberry	NRomney	DBMatthews
04/14/89	04/14/89	04/27/89	05/30/89 Subject to revisions of SER	~~04/ /89~~ 05/31/89

ATTACHMENT TO LICENSE AMENDMENT NO. 163

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Page	Insert Page
3.1-7	3.1-7
3.1-8	3.1-8
3.1-9	3.1-9
3.2-40	3.2-40
3.2-41	3.2-41

Table 4.1-1

Reactor Protection System (RPS) Instrumentation Functional Test, Functional Test Minimum Frequency, and Calibration Minimum Frequency

Scram Number (a)	Source of Scram Trip Signal	Group (b)	Instrument Check Minimum Frequency	Instrument Functional Test Minimum Frequency (c)	Instrument Calibration Minimum Frequency
1	Mode Switch in SHUTDOWN	A	NA	Once/Operating Cycle	Not Applicable
2	Manual Scram	A	NA	Once/week	Not Applicable
3	IRM High High Flux	C	D	Once/Week (e)(l)(n)	Once/Operating Cycle
	Inoperative	C	NA	Once/week (e)	NA
4	Reactor Vessel Steam Dome Pressure - High	D	S	Every 3 months	Once/Operating Cycle
5	Drywell Pressure - High	D	S	Every 3 months	Once/Operating Cycle
6	Reactor Vessel Water Level - Low (Level 3)	D	S	Every 3 months (g)	Once/Operating Cycle
7	Scram Discharge Volume High High Level				
	a. Float Switches	A	NA	Every 3 months	(h)
	b. Thermal Level Sensors	B	NA	Every 3 months	Once/Operating Cycle
8	APRM Fixed High-High Flux	B	S	Every 3 months (e)(l)	Once/Week (p),SA
	Inoperable	B	NA	Every 3 months (e)	NA
	Downscale	B	NA	Once/Week(e)	NA
	Flow Reference Simulated Thermal Power Monitor	B	S	Every 3 months (l)	Once/Week (p)(q), SA
	15% Flux	C	S	Once/Week during refueling (l)(m)(n)	Once/Week during refueling (l)(m)
9	LPRM	B	D	NA	Every 1000 Effective Full Power Hours

Table 4.1-1 (Cont.)

9	Main Steam Line High Radiation	B	D	Every 3 months (e)	Every 3 months(i)
10	Main Steam Line Isolation Valve Closure	A	NA	Every 3 months	(h)
11	Turbine Control Valve Fast Closure	A	NA	Every 3 months (j)	Once/Operating Cycle (k)
12	Turbine Stop Valve Closure	A	NA	Every 3 months	(h)
	RPS Channel Switch	A	NA	Once/Operating Cycle	Not Applicable
	Turbine First Stage Pressure Permissive	A	NA	Every 3 months	Every 6 months
	Reactor Pressure Permissive	A	NA	Every 3 months	Every 6 months

a. The column entitled "Scram Number" is for convenience so that a one-to-one relationship can be established between items in Table 4.1-1 and items in Table 3.1-1.

b. The definition for each of the four groups is as follows:

Group A. On-off sensors that provide a scram trip signal.
Group B. Analog devices coupled with bi-stable trips that provide a scram trip signal.
Group C. Devices which only serve a useful function during some restricted mode of operation, such as startup or shutdown, or for which the only practical test is one that can be performed at shutdown.
Group D. Analog transmitters and trip units that provide a scram trip function.

c. Functional tests are not required when the systems are not required to be operable or are tripped. However, if functional tests are missed, they shall be performed prior to returning the systems to an operable status.

d. Calibrations are not required when the systems are not required to be operable or are tripped. However, if calibrations are missed, they shall be performed prior to returning the system to an operable status.

e. This instrumentation is exempted from the instrument functional test definition. This instrument functional test will consist of injecting a simulated electrical signal into the measurement channels.

f. Deleted

g. The water level in the reactor will be perturbed and the corresponding level indicator changes will be monitored. This perturbation test will be performed every 3 months after completion of the functional test program.

h. Physical inspection and actuation of these position switches will be performed once per operating cycle.

i. Standard current source used which provides an instrument channel alignment. Calibration using a radiation source shall be made once per operating cycle.

j. Measure time interval from EHC pressure switch actuation to RPS relay K14 de-energization.

Notes for Table 4.1-1 (Cont'd)

k. The electrohydraulic control oil pressure sensors shall be set to trip at >600 psig control oil pressure.

l. Perform within 24 hours of startup if not performed within the previous 7 days.

m. When changing from the Run Mode to the Start and Hot Standby Mode, perform the required surveillance within 12 hours after entering the Start and Hot Standby Mode unless performed within the previous 7 days.

n. The APRM, LRM and SRM channels shall be compared for overlap during each startup, if not performed within the previous 7 days.

p. This calibration shall consist of the adjustment of the APRM channel to conform to the power values calculated by a heat balance during the Run Mode when thermal power ≥25% of rated thermal power. Adjust the APRM channel if the absolute difference ≥2%.

q. This calibration shall consist of the adjustment of the APRM flow referenced simulated thermal power channel to conform to a calibrated flow signal.

Table 4.2-7

Check, Functional Test, and Calibration Minimum Frequency for Neutron Monitoring Instrumentation Which Initiates Control Rod Blocks

Ref. No. (a)	Instrument	Instrument Check Minimum Frequency (b)	Instrument Functional Test Minimum Frequency (c)	Instrument Calibration Minimum Frequency (d)
1	SOURCE RANGE MONITORS			
	a. Detector not full in	NA	S/U(f), W	NA
	b. Upscale	NA	S/U(f), W	R
	c. Inoperative	NA	S/U(f), W	NA
	d. Downscale	NA	S/U(f), W	R
2	INTERMEDIATE RANGE MONITORS			
	a. Detector not full in	NA	S/U(f), W(e)	NA
	b. Upscale	NA	S/U(f), W(e)	R
	c. Inoperative	NA	S/U(f), W(e)	NA
	d. Downscale	NA	S/U(f), W(e)	R
3	APRM			
	a. Flow Referenced Simulated Thermal Power-Upscale	NA	S/U(f), Q	R
	b. Inoperative	NA	S/U(f), Q	NA
	c. Downscale	NA	S/U(f), M	R
	d. Neutron Flux - High, 12%	NA	S/U(f), Q	R
4	ROD BLOCK MONITOR			
	a. Upscale	NA	S/U(f), Q	R
	b. Inoperative	NA	S/U(f), Q	NA
	c. Downscale	NA	S/U(f), Q	R
5	SCRAM DISCHARGE VOLUME			
	a. Water Level-High	NA	Q	R

Notes for Table 4.2-7

a. The column titled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in Table 4.2-7 and items in Table 3.2-7.

b. Deleted.

Notes for Table 4.2-7

c. This instrumentation is exempted from the functional test definition. The instrument functional test will consist of injecting a simulated electrical signal into the measurement channel. Functional tests are not required when these instruments are not required to be operable or are tripped. However, if functional tests are missed they shall be performed prior to returning the instrument to an operable status.

d. Calibrations are not required when the instruments are not required to be operable or are tripped. However, if calibrations are missed, they shall be performed prior to returning the instrument to an operable status.

e. When changing from the Run Mode to the Start and Hot Standby Mode, perform the required surveillance within 12 hours after entering the Start and Hot Standby Mode unless performed within the previous 7 days.

f. Within 24 hours of startup if not performed within the previous 7 days.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 100
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated March 27, 1986, as supplemented Arpil 22 and December 27, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 100, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Lawrence P. Crocker for

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 6, 1989

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.100 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:
David B. Matthews

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 6, 1989

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04/14/89	04/14/89	04/ /89 5/22/89	04/30/89	04/ /89 05/31/89
		Concurred on Unit 1	5 Subject to revisions in SER	

ATTACHMENT TO LICENSE AMENDMENT NO. 100

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Page	Insert Page
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3/4 3-8	3/4 3-8

3/4.3 INSTRUMENTATION

3/4.3.1 REACTOR PROTECTION SYSTEM INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.1 As a minimum, the reactor protection system instrumentation channels shown in Table 3.3.1-1 shall be OPERABLE with the REACTOR PROTECTION SYSTEM RESPONSE TIME as shown in Table 3.3.1-2. Set points and interlocks are given in Table 2.2.1-1.

APPLICABILITY: As shown in Table 3.3.1-1.

ACTION:

a. With the requirements for the minimum number of OPERABLE channels not satisfied for one trip system, place at least one inoperable channel in the tripped condition within 12 hours.

b. With the requirements for the minimum number of OPERABLE channels not satisfied for both trip systems, place at least one inoperable channel in at least one trip system* in the tripped condition within 1 hour and take the ACTION required by Table 3.3.1-1.

c. The provisions of Specification 3.0.3 are not applicable in OPERATIONAL CONDITION 5.

SURVEILLANCE REQUIREMENTS

4.3.1.1 Each reactor protection system instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTION TEST and CHANNEL CALIBRATION operations during the OPERATIONAL CONDITIONS and at the frequencies shown in Table 4.3.1-1.

4.3.1.2 LOGIC SYSTEM FUNCTIONAL TESTS and simulated automatic operation of all channels shall be performed at least once per 18 months and shall include calibration of time delay relays and timers necessary for proper functioning of the trip system.

4.3.1.3 The REACTOR PROTECTION SYSTEM RESPONSE TIME of each reactor trip function of Table 3.3.1-2 shall be demonstrated to be within its limit at least once per 18 months. Each test shall include at least one logic train such that both logic trains are tested at least once per 36 months and one channel per function such that all channels are tested at least once every N times 18 months were N is the total number of redundant channels in a specific reactor trip function.

*If both channels are inoperable in one trip system, select at least one inoperable channel in that trip system to place in the tripped conditions, except when this could cause the Trip Function to occur.

TABLE 3.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION

ACTION 9 - In OPERATIONAL CONDITION 1 or 2, be in at least HOT SHUTDOWN within 6 hours.

In OPERATIONAL CONDITION 3 or 4, lock the reactor mode switch in the Shutdown position within 1 hour.

In OPERATIONAL CONDITION 5, suspend all operations involving CORE ALTERATIONS or positive reactivity changes and fully insert all insertable control rods within 1 hour.

TABLE NOTATIONS

a. A channel may be placed in an inoperable status for up to 6 hours for required surveillance without placing the trip system in the tripped condition provided at least one OPERABLE channel in the same trip system is monitoring that parameter.

b. The "shorting links" shall be removed from the RPS circuitry during CORE ALTERATIONS and shutdown margin demonstrations performed in accordance with Specification 3.10.3.

c. The IRM scrams are automatically bypassed when the reactor vessel mode switch is in the Run position and all APRM channels are OPERABLE and on scale.

d. An APRM channel is inoperable if there are less than 2 LPRM inputs per level or less than 11 LPRM inputs to an APRM channel.

e. These functions are not required to be OPERABLE when the reactor pressure vessel head is unbolted or removed.

f. This function is automatically bypassed when the reactor mode switch is in other than the Run position.

g. This function is not required to be OPERABLE when PRIMARY CONTAINMENT INTEGRITY is not required.

h. With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.11.1 or 3.9.11.2.

i. These functions are bypassed when turbine first stage pressure is ≤250* psig, equivalent to THERMAL POWER less than 30% of RATED THERMAL POWER.

j. Within 24 hours prior to the planned start of the hydrogen injection test with the reactor power at greater than 20% rated power, the normal full-power radiation background level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip setpoints may be adjusted during the test based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and associated trip setpoints shall be set within 24 hours of re-establishing normal radiation levels after completion of hydrogen injection and prior to establishing reactor power levels below 20% rated power.

*Initial setpoint. Final setpoint to be determined during startup testing.

TABLE 4.3.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION[a]	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED
1. Intermediate Range Monitors:				
a. Neutron Flux - High	D	S/U[b][c]	R	2
	D	W	R	3, 4, 5
b. Inoperative	NA	W	NA	2, 3, 4, 5
2. Average Power Range Monitor:				
a. Neutron Flux - Upscale, 15%	S	S/U[b][c], W[d]	S/U[b], W[d]	2
	S	W	W	5
b. Flow Referenced Simulated Thermal Power - Upscale	S	S/U[b], Q	W[e][f], SA	1
c. Fixed Neutron Flux - Upscale, 118%	S	S/U[b], Q	W[e], SA	1
d. Inoperative	NA	Q	NA	1, 2, 5
e. Downscale	NA	W	NA	1
f. LPRM	D	NA	(g)	1, 2, 5
3. Reactor Vessel Steam Dome Pressure - High	S	Q	R	1, 2
4. Reactor Vessel Water Level - Low (Level 3)	S	Q	R	1, 2
5 Main Steam Line Isolation Valve - Closure	NA	Q	R[h]	1
6. Main Steam Line Radiation - High	D	Q[i]	R	1, 2
7. Drywell Pressure - High	S	Q	R	1, 2
8. Scram Discharge Volume Water Level - High	NA	Q	R[h]	1, 2, 5

TABLE 4.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED
9. Turbine Stop Valve - Closure	NA	Q	R(h)	1
10. Turbine Control Valve Fast Closure, Trip Oil Pressure - Low	NA	Q	R	1
11. Reactor Mode Switch in Shutdown Position	NA	R	NA	1, 2, 3, 4, 5
12. Manual Scram	NA	W	NA	1, 2, 3, 4, 5

a. Neutron detectors may be excluded from CHANNEL CALIBRATION.

b. Within 24 hours prior to startup, if not performed within the previous 7 days.

c. The APRM, IRM and SRM channels shall be compared for overlap during each startup, if not performed within the previous 7 days.

d. When changing from CONDITION 1 to CONDITION 2, perform the required surveillance within 12 hours after entering CONDITION 2.

e. This calibration shall consist of the adjustment of the APRM channel to conform to the power values calculated by a heat balance during CONDITION 1 when THERMAL POWER ≥ 25% of RATED THERMAL POWER. Adjust the APRM channel if the absolute difference ≥ 2%.

f. This calibration shall consist of the adjustment of the APRM flow referenced simulated thermal power channel to conform to a calibrated flow signal.

g. The LPRM's shall be calibrated at least once per 1000 effective full power hours (EFPH) using the TIP system.

h. Physical inspection and actuation of switches for instruments 2C11-N013A, B, C, D.

i. Instrument alignment using a standard current source.

j. Calibration using a standard radiation source.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 163 AND 100 TO

FACILITY OPERATING LICENSES DPR-57 AND NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-321 AND 50-366

1.0 INTRODUCTION

By letter dated March 27, 1986, Georgia Power Company (the licensee) requested changes to the Technical Specifications (TS) for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The proposed changes would: (1) change the Hatch Unit 1 Reactor Protection System (RPS) and Control Rod Block surveillance requirements to provide for consistency with the Hatch Unit 2 TS; and (2) change the RPS functional test frequencies and equipment outage times for both units based on General Electric Company Topical Report NEDC-30851 P which provides a probabilistic basis for RPS surveillance frequencies and equipment outage times. The proposed changes were noticed in the Federal Register (FR 23610) with an opportunity for prior hearing on June 30, 1986. No requests for a hearing on the application have been received.

By letters dated April 22 and December 27, 1988, the licensee furnished additional information in support of the original request for amendment. These letters did not alter the changes requested, but merely supplied amplifying and clarifying information in support of the request.

2.0 EVALUATION

On July 15, 1987, the Division of Engineering and Systems Technology, Office of Nuclear Reactor Regulation (NRR), issued a Safety Evaluation Report (SER) based on its review of GE Topical Report NEDC-30851P. That SER stated that NEDC-30851P provided an acceptable generic basis for supporting plant-specific TS changes related to the RPS for plants using a relay RPS, subject to the following plant-specific conditions as stated in Table 1 of the SER:

Condition 1. Confirm the applicability of the generic analyses for NEDC-30851P to its plant.

Condition 2. Demonstrate by use of current drift information provided by the equipment vendor or plant-specific data, that the drift characteristics for instrumentation used in the RPS channels in the plant are bounded by the assumptions used in NEDC-30851P when the functional test interval is extended from monthly to quarterly.

Condition 3. Confirm that the differences between the RPS in the plant and the RPS of the generic analysis plant were included in the plant-specific analysis using the procedures of Appendix K of NEDC-30851P, or provide plant-specific analyses to demonstrate that there is no appreciable change in RPS availability or public risk.

The staff has reviewed the plant-specific analyses reports for Hatch Units 1 and 2, MDE-75-0485 and MDE-76-0485, and has determined that they confirm: (1) the applicability of the generic analysis, NEDC-30851P and (2) the differences between the Hatch RPS and the RPS of the generic analysis plant. The staff also reviewed GPC's response to RPS instrumentation drift when the functional test interval is extended from monthly to quarterly and determined that response to be acceptable.

GE reports MDE-75-0485 and MDE-76-0485 were utilized to support a plant-specific evaluation of the differences between the RPS in the plant and the RPS of the generic analysis plant. A seven step process of evaluation is documented in Appendix A of these reports. These reports conclude by stating that the results of the evaluations indicated that the RPS configurations for Hatch Units 1 and 2 have several differences compared to the RPS configuration in the generic evaluation. The analysis reported in NEDC-30851P states that these differences would not significantly affect the improvement in plant safety due to the changes in the TS based on the generic analysis. The generic analysis in NEDC-30851P is, therefore, applicable to Hatch Units 1 and 2.

The staff has determined that GPC has met the requirements as set forth in the NRR Safety Evaluation Report dated July 15, 1987, and as clarified in the NRC Staff Guidance for Condition 2, dated April 27, 1988. Approval of the proposed amendments would provide for consistency between Unit 1 and Unit 2 RPS and Control Rod Block instrumentation surveillance requirements and would change the RPS surveillance frequencies and outage times for both units.

The staff has evaluated the licensee's proposed amendments and found them acceptable. The acceptance is based on the criterion that safety will be maintained as stated in the Edwin I. Hatch Nuclear Plant Final Safety Analysis Report Update.

3.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.32, the Commission has determined that the issuance of these amendments will have no significant impact on the environment (54 FR 24279).

4.0 CONCLUSION

The Commission issued a Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Opportunity for Prior Hearing which was published in the Federal Register on June 30, 1986 (51 FR 23610). No public comments were received, and the state of Georgia did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. D. Starkey, RII
Lawrence P. Crocker, PDII-3/DRP-I/II

Dated: June 6, 1989

UNITED STATES NUCLEAR REGULATORY COMMISSION

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NOS. 50-321 AND 50-366

NOTICE OF ISSUANCE OF AMENDMENTS TO

FACILITY OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 163 to Facility Operating License No. DPR-57 and Amendment No. 100 to Facility Operating License No. NPF-5 issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensee), which revised the Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Units 1 and 2, (the facility) located in Appling County, Georgia. The amendments were effective as of the date of issuance.

The amendments change the Unit 1 reactor protection system (RPS) and control rod block surveillance requirements to provide for consistency with the Unit 2 Technical Specifications and change the RPS functional test frequencies and equipment outage times for both units.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments.

Notice of Consideration of Issuance of Amendments and Opportunity for Prior Hearing in connection with this action was published in the FEDERAL REGISTER on June 30, 1986 (51 FR 23610). No request for a hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of the amendments will not have a significant effect on the quality of the human environment.

For further details with respect to the action see (1) the application for amendments dated March 27, 1986, as supplemented April 22 and December 27, 1988, (2) Amendment No.163 to License No. DPR-57 and Amendment No. 100 to License No. NPF-5 and (3) the Commission's related Safety Evaluation and Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, 2120 L Street, N.W., and at the Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Projects I/II.

Dated at Rockville, Maryland this 6th day of June 1989.

FOR THE NUCLEAR REGULATORY COMMISSION

Lawrence P. Crocker

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Dated at Rockville, Maryland this 6th day of June 1989.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

June 6, 1989

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DOCKET NOS. 50-321/366

MEMORANDUM FOR: Regulatory Publications Branch
Division of Freedom of Information and Publications Services
Office of Administration and Resources Management

FROM: Office of Nuclear Reactor Regulation

SUBJECT: E. I. Hatch Nuclear Plant, Units 1 and 2 (Georgia Power Company, et al.)

One signed original of the *Federal Register* Notice identified below is enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (5) of the Notice are enclosed for your use.

- ☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- ☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- ☐ Notice of Consideration of Issuance of Amendment to Facility Operating License. (Call with____-day insert date).
- ☐ Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- ☐ Notice of Availability of NRC Draft/Final Environmental Statement.
- ☐ Notice of Limited Work Authorization.
- ☐ Notice of Availability of Safety Evaluation Report.
- ☐ Notice of Issuance of Construction Permit(s).
- ☒ Notice of Issuance of Facility Operating License(s) or Amendment(s).
- ☐ Order.
- ☐ Exemption.
- ☐ Notice of Granting Exemption.
- ☐ Environmental Assessment.
- ☐ Notice of Preparation of Environmental Assessment.
- ☐ Receipt of Petition for Director's Decision Under 10 CFR 2.206.
- ☐ Issuance of Final Director's Decision Under 10 CFR 2.206.
- ☐ Other: _____

Enclosure:
As stated

Contact: M. Rood
Phone: 21487

OFFICE	PD II-3						
SURNAME	MRood						
DATE	6/6/89						

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