February 16, 1988

Docket No.: 50-321

Mr. James P. O'Reilly
Senior Vice President - Nuclear Operations
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. O'Reilly:

Subject: Issuance of Amendment No. 152 to Facility Operating License DPR-57 - Edwin I. Hatch Nuclear Plant, Unit 1 (TAC 65997)

The Commission has issued the enclosed Amendment No. 152 to Facility Operating License DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit 1. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated July 13, 1987.

The amendment modifies the definition of surveillance frequency in Section 1.II of the TS to provide for an 18-month operating cycle instead of the 15 months specified previously.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

Enclosures:
1. Amendment No. 152 to DPR-57
2. Safety Evaluation

cc w/enclosures:
See next page

PD#II-3/DRP-I/II	PD#II-3/DRP-I/II	PD#II-3/DRP-I/II
MRood/mac	LCrocker	Acting PD
~~12/ /87~~	~~12/ /87~~	~~12/ /87~~
1/4/88	1/4/88	2/1/88

DATED February 16, 1988

AMENDMENT NO. 152 TO FACILITY OPERATING LICENSE DPR-57, EDWIN I. HATCH, UNIT 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 152
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated July 13, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 152, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Lawrence P. Crocker , Acting Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 16, 1988

Concurrence subject to resolution of Matters set forth in note of 1/29/88.

PD#II-3/DRP-I/II	PD#II-3/DRP-I/II	NRR/OTSB	OGC-Bethesda
MRood/mac	LCrocker	REmch	SH Lewis
~~12/ /87~~ 1/4/88	~~12/ /87~~ 1/4/88	~~12/ /87~~ 1/6/88	~~12/ /87~~ 1/28/88
PD#II-3/DRP-I/II	AD/DRPI/II		
Acting PD	GLainas		
~~12/ /87~~ 2/1/88	~~12/ /87~~ 1/ /87		

ATTACHMENT TO LICENSE AMENDMENT NO. 152

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

Remove Page	Insert Page
1.0-6	1.0-6

GG. Simulated Automatic Actuation - Simulated automatic actuation means applying a simulated signal to the sensor to actuate the circuit in question.

HH. Start & Hot Standby Mode - The reactor is in the Start & Hot Standby Mode when the Mode Switch is in the START & HOT STANDBY position. In this mode the reactor protection system is energized with IRM and APRM (Start & Hot Standby Mode) neutron monitoring system trips and control rod withdrawal inter-locks in service.

II. Surveillance Frequency - Periodic surveillance tests, checks, calibrations, and examinations shall be performed within the specified surveillance intervals. These intervals may be adjusted plus or minus 25%. The operating cycle interval is defined as 18 months. In the case where the elapsed interval has exceeded 100% of the specified interval, the next surveillance interval shall commence at the end of the original specified interval.

JJ. Surveillance Requirements - The surveillance requirements are requirements established to ensure that the Limiting Conditions for Operation as stated in Section 3 of these Technical Specifications are met. Surveillance requirements are not required on systems or parts of systems that are not required to be operable or are tripped. If tests are missed on parts not required to be operable or are tripped, then they shall be performed prior to returning the system to an operable status.

KK. Total Peaking Factor (TPF) - The total peaking factor is the highest product of radial, axial, and local peaking factors simultaneously operative at any segment of fuel rod.

LL. Transition Boiling - Transition boiling is the boiling that occurs between nucleate and film boiling. Transition boiling is manifested by an unstable fuel cladding surface temperature, rising suddenly as steam blanketing of the heat transfer surface occurs, then dropping as the steam blanket is swept away by the coolant flow, then rising again.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 152TO

FACILITY OPERATING LICENSE DPR-57

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-321

INTRODUCTION

By letter dated July 13, 1987, (Reference 1), Georgia Power Company (the licensee) requested a change to the Technical Specifications (TS) for the Edwin I. Hatch Nuclear Plant, Unit 1. The requested change would modify the definition of Surveillance Frequency in Section 1.II of the TS to provide for an operating cycle length of 18 months instead of the 15 months specified in the existing TS.

EVALUATION

Section 1.II of the Hatch Unit 1 TS states in part, "The operating cycle interval as pertaining to instrument and electrical surveillance shall never exceed 15 months." The licensee proposes to change this wording to state, "The operating cycle interval is defined as 18 months." Hatch Unit 1 has "custom" TS, that were issued at a time (1974) when the normal fuel cycles were on the order of 12 months. Since that time, improvements in fuel design have allowed longer operating cycles between refueling outages. Hatch Unit 1 is now operating in cycle 11, which is expected to last on the order of 18 months. The existing TS would require the licensee to shut down the reactor after 15 months of operation to perform required surveillances, even though the original intent was to perform the surveillances at the end of the operating cycle.

Later plants, including Hatch Unit 2, have TS based upon the Standard Technical Specifications which contain a "Surveillance Frequency Notations" table defining a refueling cycle as being equal to 18 months. The terms operating cycle and refueling cycle have the same meaning and are used interchangeably. The licensee thus is in the position where the two Hatch units have different definitions of operating cycle (refueling cycle) length, even though the two units are essentially identical.

Compounding the problem, the Hatch Unit 1 TS are internally inconsistent. In conjunction with a license change regarding the radiological environmental technical specifications, Amendment No. 110 added a Table 1.1,

"Frequency Notations", to the Hatch Unit 1 TS. This table defines a refueling cycle as 18 months, but a corresponding change to Section 1.II to redefine the operating cycle as being equal to 18 months was not made. The proposed change would correct this inconsistency.

Actual plant trip setpoints for instruments and electrical equipment are set conservative to the TS allowable values, such that the allowable values are not compromised during an operating cycle by instrument drift. Extending the allowable time between refuelings from 15 months to 18 months would require an adjustment to the actual trip setpoints, but would not affect the TS allowable values. Thus, the design functions of the electrical and instrument systems are unaffected and the change would have no adverse effect on the safety analyses for the plant.

The licensee already is experienced in adjusting setpoints to compensate for instrument drift over an 18-month refueling cycle for Unit 2. Comparable adjustments for the Unit 1 instruments should pose no problems.

We conclude that the change requested by the licensee will resolve the internal inconsistency in the Hatch Unit 1 TS and will result in both Hatch Units being on an 18-month operating cycle. Both of these are desirable. Since no changes are made to the allowable trip setpoints, the change would have no adverse effect on plant safety. We, therefore, conclude that the proposed change is acceptable.

ENVIRONMENTAL CONSIDERATIONS

This amendment changes surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register on January 13, 1988 (53 FR 827), and consulted with the state of Georgia. No public comments were received, and the state of Georgia did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

REFERENCE

1. Letter from J. P. O'Reilly, GPC, to U. S. Nuclear Regulatory Commission, dated July 13, 1987.

Principal Contributor: Lawrence P. Crocker, PDII-3/DRPI/II

Dated: February 16, 1988

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