

July 12, 1988

Docket Nos: 50-321
and 50-366

Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT NOS. 155 AND 94 TO FACILITY OPERATING LICENSES
DPR-57 AND NPF-5, EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2
(TACS 68222/68223)

The Commission has issued the enclosed Amendment Nos. 155 and 94 to Facility Operating Licenses DPR-57 and NPF-5 for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments are being issued in response to your letter dated May 6, 1988, supplemented June 7, 1988.

The amendments modify the Technical Specifications by replacing the organization charts with more general organization requirements. The amendments are effective as of date of issuance.

A copy of our safety evaluation supporting these amendments is enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original signed by:

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

Enclosures:

1. Amendment No. 155 to DPR-57
2. Amendment No. 94 to NPF-5
3. Safety Evaluation

cc w/ enclosures:
See next page

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LA:PDII-3
MRood
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LCrocker:pw
7/15/88

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for DMatthews
7/15/88

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Mr. W. G. Hairston, III
Georgia Power Company

Edwin I. Hatch Nuclear Plant,
Units Nos. 1 and 2

cc:

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Chairman
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County Courthouse
Baxley, Georgia 31513

DATED July 12, 1988

AMENDMENT NO. 155 TO FACILITY OPERATING LICENSE DPR-57, EDWIN I. HATCH, UNIT 1
AMENDMENT NO. 94 TO FACILITY OPERATING LICENSE NPF-5, EDWIN I. HATCH, UNIT 2

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W. Jones	P-130A
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 155
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated May 6, 1988, supplemented June 7, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 155, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 12, 1988

OFFICIAL RECORD COPY

LA:PDII-3
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PM:PDII-3
LCrocker:pw
7/15/88

OGC-WF
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7/12/88

ATTACHMENT TO LICENSE AMENDMENT NO. 155

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Page

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5-1 ETS
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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.0 The General Manager - Nuclear Plant shall provide direct executive oversight over all aspects of Plant Hatch.

6.1.1 The Plant Manager shall be responsible for overall unit operation, except for the Radiological Environmental Monitoring Program as described below and for delegation in writing of the succession of this responsibility during his absence. Certain plant support functions shall be the responsibility of the Plant Support Manager.

6.1.2 The Manager Radiological Safety shall be responsible for the Radiological Environmental Monitoring Program as described in the Specification 3/4.16 of Unit 1 and for the writing of the Annual Radiological Environmental Surveillance Report. The Manager Radiological Safety shall review the proposed changes to plant systems or equipment, provided that such changes are identified by the Plant Review Board (PRB) as having a potential radiological environmental impact.

6.1.3 Each of the above-mentioned individuals is responsible for the accuracy of the procedures needed to implement his responsibilities.

6.2 ORGANIZATION

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Plant Hatch Unit 2 updated FSAR.
- b. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President - Nuclear shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining and providing technical support to the plant to ensure nuclear safety.

- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 UNIT STAFF

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in table 6.2.2-1.
- b. At least one licensed Operator shall be in the control room for each reactor containing fuel.
- c. At least two licensed Operators shall be present in the control room for each reactor in the process of start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified to implement radiation protection procedures shall be on site when fuel is in either reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A Fire Team of at least five members shall be maintained onsite at all times. The Fire Team shall not include the minimum shift crew necessary for safe shutdown on Units 1 and 2 or any personnel required for other essential functions during a fire emergency.
- g. Administrative procedures shall be developed and implemented to limit the working hours of Unit staff who perform safety-related functions; e.g., senior reactor operators, reactor operators, auxiliary operators, health physicists, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a nominal 40-hour week while the plant is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used or during extended periods of shutdown for refueling, major maintenance, or major plant modifications, the following guidelines shall be followed on a temporary basis:

- (1) An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
- (2) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any

48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.

- (3) A break of at least 8 hours should be allowed between work periods, including shift turnover time.
- (4) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Plant Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

- h. The Operations Superintendent, Operations Supervisor, and Shift Supervisors shall hold a senior reactor operator license. The plant operators and assistant plant operators shall hold a reactor operator license.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and three voting members including alternates.

RESPONSIBILITIES

6.5.1.6 The PRB shall be responsible for:

- a. Review of all procedures required by Specification 6.8 and changes thereto, except those for the Radiological Environmental Monitoring Program, any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix A Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all reportable violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager - Nuclear Plant, the Vice President - Nuclear, and to the Safety Review Board (SRB).
- f. Review of all REPORTABLE EVENTS.
- g. Review of unit operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the SRB.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- i. Review of the Security Plan and implementing procedures and shall submit recommended changes to the SRB.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the SRB.
- k. Review of any unplanned onsite release of radioactive material to the environs when such release is in excess of 1 Ci, excluding dissolved and entrained gases and tritium for liquid effluents, and in excess of 150 Ci of noble gases or 0.02 Ci of radioiodines for gaseous effluents. Also included is the preparing and forwarding to the Plant Manager and the SRB reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL (ODCM), except for the section on the Radiological Environmental Monitoring Program in the ODCM.
- m. Review of proposed changes to plant systems and equipment to determine whether the proposed change has a potential radiological environmental impact. Such change(s) will be reported to the Manager-Radiological Safety.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Recommend in writing to the Plant Manager approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the General Manager - Nuclear Plant, the Vice President - Nuclear, and the SRB of disagreement between the PRB and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the General Manager - Nuclear Plant, the Vice President - Nuclear, and the SRB.

ADMINISTRATIVE CONTROLS

6.5.2 SAFETY REVIEW BOARD (SRB)

FUNCTION

6.5.2.1 The SRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations.
- b. Nuclear engineering.
- c. Chemistry and radiochemistry.
- d. Metallurgy.
- e. Instrumentation and control.
- f. Radiological safety.
- g. Mechanical and electrical engineering.
- h. Quality assurance practices.

COMPOSITION

6.5.2.2 The SRB shall be composed of a minimum of five persons who as a group provide the expertise to review and audit the operation of a nuclear power plant. The Chairman and other members shall be appointed by the Vice President - Nuclear or his designee. The composition of the SRB shall meet the requirements of ANSI N18.7-1976.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRB Chairman to serve on a temporary basis; however, no more than a minority of alternates shall participate as voting members in SRB activities at any one time. However in extenuating circumstances, the Vice President - Nuclear may designate the use of additional alternates with voting authority when regular members are not available within necessary time constraints.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRB Chairman to provide expert advice to the SRB.

MEETING FREQUENCY

6.5.2.5 The SRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.5.2.6. The minimum quorum of the SRB necessary for the performance of the SRB review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least a majority of the members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7. The SRB shall be responsible for the review of:

- a. The safety evaluations for (1) changes to procedures, equipment, or systems and (2) tests or experiments completed under the provision of section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. ALL REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the Plant Review Board.

ADMINISTRATIVE CONTROLS

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the SRB. Each inspection or audit shall be performed within the specified time interval with:

1. A maximum allowable extension not to exceed 25 percent of the inspection or audit interval.
2. A total maximum combined interval time for any three consecutive inspections or audit intervals not to exceed 3.25 times the specified inspection or audit interval.

These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR 50, at least once per 24 months.
- e. The Emergency Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.
- f. The Security Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.
- g. Any other area of unit operation considered appropriate by the SRB or the Vice President - Nuclear.
- h. The Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years. During the year in which the inspection or audit occurs, the requirements of 6.5.2.8.i can be affected concurrently.

ADMINISTRATIVE CONTROLS

- k. The Radiological Environmental Monitoring Program and the results thereof annually.
- 1. The ODCM, Process Control Program, and implementing procedures at least once per 24 months.

AUTHORITY

6.5.2.9 The SRB shall report to and advise the Vice President - Nuclear on those areas of responsibility specified in sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of SRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRB meeting shall be prepared, approved and forwarded to the Vice President - Nuclear within 14 days following each meeting.
- b. Reports of reviews encompassed by section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President - Nuclear within 14 days following completion of the review.
- c. Audit reports encompassed by section 6.5.2.8 above, shall be forwarded to the Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PRB and the results of this review shall be submitted to the SRB, the General Manager - Nuclear Plant, and the Vice President - Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT SHUTDOWN within 2 hours.
- b. The Safety Limit violation shall be reported to the Commission as soon as practical and in all cases within 1 hour of occurrence. The General Manager - Nuclear Plant, and Vice President - Nuclear, and the SRB shall be notified within 24 hours.

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- c. A Licensee Event Report shall be prepared pursuant to 10 CFR 50.73.
- d. The Licensee Event Report shall be submitted to the Commission in accordance with 10 CFR 50.73, and to the PRB, the SRB, the General Manager - Nuclear Plant, and the Vice President - Nuclear within 30 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. ODCM implementation.

6.8.2 Each procedure of 6.8.1 and other procedures which the Plant Manager or Plant Support Manager has determined to affect nuclear safety, and changes thereto, shall be reviewed by the PRB and approved by the appropriate member of plant management, designated by the Plant Manager or Plant Support Manager prior to implementation. The Plant Manager or Plant Support Manager will approve administrative procedures, security plan implementing procedures, and changes thereto. The Manager-Plant Training and Onsite Emergency Preparedness shall approve the emergency plan implementing procedures and changes thereto. All other procedures of this specification and changes thereto will be approved by the department head designated by the Plant Manager or Plant Support Manager. The procedures of this specification shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.

5.0 Administrative Controls

This section describes administrative and management controls established to implement the Environmental Technical Specifications (ETS). Measures specified in this section include assignments of responsibility, review and audit functions, procedures, and reporting requirements.

Corporate responsibility for implementing the ETS and for assuring that the station is operated in such a way as to provide protection for the environment rests with the Vice President - Nuclear.

Responsibilities for compliance with the ETS and for the environmental monitoring program required by the ETS are given below.

Independent audit shall be provided, as discussed in section 5.3.2, by the General Manager-Quality Assurance.

5.1 Responsibility

5.1.1 The Plant Manager is responsible for the environmental monitoring programs. The Plant Manager is also responsible for implementing the special surveillance activities described in section 4.

5.1.2 The General Manager-Quality Assurance is responsible for assuring that the periodic audits of plant operation and the environmental monitoring activities to ensure conformance with the ETS are conducted.

5.2 Organization

Company organization relative to environmental matters is presented in Chapter 13 of the Plant Hatch Unit 2 updated Final Safety Analysis Report.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 94
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated May 6, 1988, supplemented June 7, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 94, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 12, 1988

OFFICIAL RECORD COPY

LA:PDII-3
MRood
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7/4/88

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7/12/88

ATTACHMENT TO LICENSE AMENDMENT NO. 94

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove Page</u>	<u>Insert Page</u>
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5-1 ETS	5-1
5-10 ETS	--

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.0 The General Manager - Nuclear Plant shall provide direct executive oversight over all aspects of Plant Hatch.

6.1.1 The Plant Manager shall be responsible for overall unit operation, except for the Radiological Environmental Monitoring Program as described below and for delegation in writing of the succession of this responsibility during his absence. Certain plant support functions shall be the responsibility of the Plant Support Manager.

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6.1.3 Each of the above-mentioned individuals is responsible for the accuracy of the procedures needed to implement his responsibilities.

6.2 ORGANIZATION

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Plant Hatch Unit 2 updated FSAR.
- b. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President - Nuclear shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining and providing technical support to the plant to ensure nuclear safety.

6.0 ADMINISTRATIVE CONTROLS

- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 UNIT STAFF

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in table 6.2.2-1.
- b. At least one licensed Operator shall be in the control room for each reactor containing fuel.
- c. At least two licensed Operators shall be present in the control room for each reactor in the process of start-up, scheduled reactor shutdown and during recovery from reactor trips.
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- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A Fire Team of at least five members shall be maintained onsite at all times. The Fire Team shall not include the minimum shift crew necessary for safe shutdown on Units 1 and 2 or any personnel required for other essential functions during a fire emergency.
- g. Administrative procedures shall be developed and implemented to limit the working hours of Unit staff who perform safety-related functions; e.g., senior reactor operators, reactor operators, auxiliary operators, health physicists, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a nominal 40-hour week while the plant is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used or during extended periods of shutdown for refueling, major maintenance, or major plant modifications, the following guidelines shall be followed on a temporary basis:

- (1) An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.

ADMINISTRATIVE CONTROLS

- (2) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.
- (3) A break of at least 8 hours should be allowed between work periods, including shift turnover time.
- (4) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Plant Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

- h. The Operations Superintendent, Operations Supervisor, and Shift Supervisors shall hold a senior reactor operator license. The plant operators and assistant plant operators shall hold a reactor operator license.

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Health Physics Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager of Training and shall meet or exceed the requirements and recommendations of section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR part 55.

6.4.2 A training program for fire protection shall be maintained under the direction of the Senior Regulatory Specialist and shall meet or exceed the requirements of section 27 of the NFPA Code-1975, except for fire protection training sessions which shall be held at least once per 92 days.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1 The PRB shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PRB shall be composed of, as a minimum, a supervisor or higher level individual from each of the departments listed below:

- Operations
- Maintenance
- Quality Control (QC)
- Health Physics
- Nuclear Safety and Compliance
- Engineering Support

The Chairman, his alternate, and other members of the PRB shall be designated by the Plant Manager. The Chairman and his designated alternate shall both be managers of one of the six above listed departments or a higher level onsite manager.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and three voting members including alternates.

RESPONSIBILITIES

6.5.1.6 The PRB shall be responsible for:

- a. Review of all procedures required by Specification 6.8 and changes thereto, except those for the Radiological Environmental Monitoring Program, any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix A Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all reportable violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager - Nuclear Plant, the Vice President - Nuclear, and to the Safety Review Board (SRB).
- f. Review of all REPORTABLE EVENTS.
- g. Review of unit operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the SRB.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- i. Review of the Security Plan and implementing procedures and shall submit recommended changes to the SRB.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the SRB.
- k. Review of any unplanned onsite release of radioactive material to the environs when such release is in excess of 1 Ci, excluding dissolved and entrained gases and tritium for liquid effluents, and in excess of 150 Ci of noble gases or 0.02 Ci of radioiodines for gaseous effluents. Also included is the preparing and forwarding to the Plant Manager and the SRB reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL (ODCM), except for the section on the Radiological Environmental Monitoring Program in the ODCM.
- m. Review of proposed changes to plant systems and equipment to determine whether the proposed change has a potential radiological environmental impact. Such change(s) will be reported to the Manager-Radiological Safety.
- n. Review of the Fire Protection Program and implementing procedures and shall submit recommended changes to the SRB.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Recommend in writing to the Plant Manager approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the General Manager - Nuclear Plant, the Vice President - Nuclear, and the SRB of disagreement between the PRB and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the General Manager - Nuclear Plant, the Vice President - Nuclear, and the SRB.

ADMINISTRATIVE CONTROLS

6.5.2 SAFETY REVIEW BOARD (SRB)

FUNCTION

6.5.2.1 The SRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations.
- b. Nuclear engineering.
- c. Chemistry and radiochemistry.
- d. Metallurgy.
- e. Instrumentation and control.
- f. Radiological safety.
- g. Mechanical and electrical engineering.
- h. Quality assurance practices.

COMPOSITION

6.5.2.2 The SRB shall be composed of a minimum of five persons who as a group provide the expertise to review and audit the operation of a nuclear power plant. The Chairman and other members shall be appointed by the Vice President - Nuclear or his designee. The composition of the SRB shall meet the requirements of ANSI N18.7-1976.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRB Chairman to serve on a temporary basis; however, no more than a minority of alternates shall participate as voting members in SRB activities at any one time. However in extenuating circumstances, the Vice President - Nuclear may designate the use of additional alternates with voting authority when regular members are not available within necessary time constraints.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRB Chairman to provide expert advice to the SRB.

MEETING FREQUENCY

6.5.2.5 The SRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.2.6 The minimum quorum of the SRB necessary for the performance of the SRB review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least a majority of the members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7 The SRB shall be responsible for the review of:

- a. The safety evaluations for (1) changes to procedures, equipment, or systems and (2) tests or experiments completed under the provision of section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the PRB.

ADMINISTRATIVE CONTROLS

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the SRB. Each inspection or audit shall be performed within the specified time interval with:

1. A maximum allowable extension not to exceed 25 percent of the inspection or audit interval.
2. A total maximum combined interval time for any three consecutive inspections or audit intervals not to exceed 3.25 times the specified inspection or audit interval.

These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR 50, at least once per 24 months.
- e. The Emergency Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.
- f. The Security Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.
- g. Any other area of unit operation considered appropriate by the SRB or the Vice President - Nuclear.
- h. The Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years. During the year in which the inspection or audit occurs, the requirements of 6.5.2.8.i can be affected concurrently.

ADMINISTRATIVE CONTROLS

- k. The Radiological Environmental Monitoring Program and the results thereof annually.
- l. The ODCM, Process Control Program, and implementing procedures at least once per 24 months.

AUTHORITY

6.5.2.9 The SRB shall report to and advise the Vice President - Nuclear on those areas of responsibility specified in sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of SRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRB meeting shall be prepared, approved and forwarded to the Vice President - Nuclear within 14 days following each meeting.
- b. Reports of reviews encompassed by section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President - Nuclear within 14 days following completion of the review.
- c. Audit reports encompassed by section 6.5.2.8 above, shall be forwarded to the Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PRB and the results of this review shall be submitted to the SRB, the General Manager - Nuclear Plant, and the Vice President - Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT SHUTDOWN within 2 hours.
- b. The Safety Limit violation shall be reported to the Commission as soon as practical and in all cases within 1 hour of occurrence. The General Manager - Nuclear Plant, and Vice President - Nuclear, and the SRB shall be notified within 24 hours.

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- c. A Licensee Event Report shall be prepared pursuant to 10 CFR 50.73.
- d. The Licensee Event Report shall be submitted to the Commission in accordance with 10 CFR 50.73, and to the PRB, the SRB, the General Manager - Nuclear Plant, and the Vice President - Nuclear within 30 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. ODCM implementation.

6.8.2 Each procedure of 6.8.1 and other procedures which the Plant Manager or Plant Support Manager has determined to affect nuclear safety, and changes thereto, shall be reviewed by the PRB and approved by the appropriate member of plant management, designated by the Plant Manager or Plant Support Manager prior to implementation. The Plant Manager or Plant Support Manager will approve administrative procedures, security plan implementing procedures, and changes thereto. The Manager-Plant Training and Onsite Emergency Preparedness shall approve the emergency plan implementing procedures and changes thereto. All other procedures of this specification and changes thereto will be approved by the department head designated by the Plant Manager or Plant Support Manager. The procedures of this specification shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.

5.0 Administrative Controls

This section describes administrative and management controls established to implement the Environmental Technical Specifications (ETS). Measures specified in this section include assignments of responsibility, review and audit functions, procedures, and reporting requirements.

Corporate responsibility for implementing the ETS and for assuring that the station is operated in such a way as to provide protection for the environment rests with the Vice President - Nuclear.

Responsibilities for compliance with the ETS and for the environmental monitoring program required by the ETS are given below.

Independent audit shall be provided, as discussed in section 5.3.2, by the General Manager-Quality Assurance.

5.1 Responsibility

5.1.1 The Plant Manager is responsible for the environmental monitoring programs. The Plant Manager is also responsible for implementing the special surveillance activities described in section 4.

5.1.2 The General Manager-Quality Assurance is responsible for assuring that the periodic audits of plant operation and the environmental monitoring activities to ensure conformance with the ETS are conducted.

5.2 Organization

Company organization relative to environmental matters is presented in Chapter 13 of the Plant Hatch Unit 2 updated Final Safety Analysis Report.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 155 AND 94 TO

FACILITY OPERATING LICENSES DPR-57 AND NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-321 AND 50-366

1.0 INTRODUCTION

By letter dated May 6, 1988, supplemented June 7, 1988, Georgia Power Company (GPC) proposed to replace Technical Specification (TS) Figure 6.2.1-1, "Offsite Organization," and TS Figure 6.2.2-1, "Unit Organization," with a narrative description of the offsite and onsite organizations' functional requirements. Guidance for these proposed changes to the TS was provided to licensees and applicants by Generic Letter (GL) 88-06, "Removal of Organization Charts from Technical Specification Administrative Control Requirements" dated March 22, 1988.

2.0 BACKGROUND

Consistent with the guidance provided in the Standard Technical Specifications, Sections 6.2.1 and 6.2.2 of the administrative control requirements have referenced offsite and unit (onsite) organization charts that are provided as figures to these sections. On a plant specific basis, these organization charts have been provided by applicants and included in the TS issued with the operating license. Subsequent restructuring of either the offsite or unit organizations, following the issuance of an operating license, has required licensees to submit a license amendment for NRC approval to reflect the desired changes in these organizations. As a consequence, organizational changes have necessitated the need to request an amendment of the operating license.

Because of these limitations on organization structure, the nuclear industry has highlighted this as an area for improvement in the TS. The Shearon Harris licensee proposed changes to remove organization charts from its TS under the lead-plant concept that included the endorsement of the proposed changes by the Westinghouse Owners Group. In its review of the Shearon Harris (a lead-plant) proposal, the staff concluded that most of the essential elements of offsite and onsite organization charts are captured by other regulatory requirements, notably Appendix B to 10 CFR Part 50.

However, there were aspects of the organizational structure that are important to ensure that the administrative control requirements of 10 CFR 50.36 would be met and that would not be retained with the removal of the organization charts. The applicable regulatory requirements are those administrative controls that are necessary to ensure safe operation of the facility. Therefore, those aspects of organization charts for Shearon Harris that were essential for conformance with regulatory requirements were added (1) to TS 6.2.1 to define functional requirements for the offsite and onsite organizations and (2) to TS 6.2.2 to define qualification requirements of their unit staff.

By letter dated January 27, 1988, the staff issued Amendment No. 3 to Facility Operating License NPF-63 for the Shearon Harris Nuclear Power Plant that incorporated these changes to their TS. Subsequently, the staff developed guidance on an acceptable format for license amendment requests to remove the organization charts from TS. GL 88-06 provided this guidance to all power reactors.

3.0 EVALUATION

By letter dated May 6, 1988, supplemented June 7, 1988, GPC states that the proposed changes to the TS are in accordance with the guidance provided by GL 88-06. The NRC staff evaluation follows:

1. GPC requested the deletion of Figures 6.2.1-1 (Offsite Organization) and 6.2.2-1 (Unit Organization) from the Technical Specifications. In conjunction with this change, Section 6.2-1 will be revised to delete references to the figures and to add general requirements that capture the essential elements of the organizational structure. The licensee has stated that equivalent information to that contained in the organization charts will be documented in the Final Safety Analysis Report.

We find this requested change acceptable as it conforms to the guidance of Generic Letter 88-06, "Removal of Organization Charts from Technical Specification Administrative Control Requirements."

2. GPC requested the revision of Sections 6.5.1.6.e, 6.5.1.7.c, 6.5.1.8., 6.5.2.2, 6.5.2.3, 6.5.2.8.g, 6.5.2.9, 6.5.2.10.a, 6.5.2.10.b, 6.5.2.10.c, 6.6.1.b, 6.7.1.b, and 6.7.1.d to change the reference in each of these sections from Senior Vice President-Nuclear Operations to the new title of Vice President-Nuclear.

We find these requested changes acceptable as the position of Vice President-Nuclear is a sufficiently high level corporate position to implement the responsibilities designated within each of these sections.

3. GPC requested the revision of Sections 6.1.0, 6.5.1.6.e, 6.5.1.7.c, 6.5.1.8, 6.6.1.b, 6.7.1.b, and 6.7.1.d, to change the reference in each of these sections from Vice President-Plant Hatch to General Manager-Nuclear Plant.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 12, 1988

Docket Nos. 50-321
and 50-366

MEMORANDUM FOR: Sholly Coordinator

FROM: Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

SUBJECT: REQUEST FOR PUBLICATION IN BIWEEKLY FR NOTICE - NOTICE
OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE
(TACS 68222/68223)

Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric
Authority of Georgia, City of Dalton, Georgia, Docket Nos. 50-321 and 50-366,
Edwin I. Hatch Nuclear Plant, Units 1 and 2, Appling County, Georgia

Date of application for amendments: May 6, 1988, as supplemented June 7, 1988

Brief description of amendments: The amendments modified the Technical
Specifications by replacing the organization charts with more general
organization requirements.

Date of issuance: July 12, 1988

Effective date: July 12, 1988

Amendment Nos.: 155 and 94

Facility Operating License Nos. DPR-57 and NPF-5. Amendments revised the
Technical Specifications.

Date of initial notice in FEDERAL REGISTER: June 3, 1988 (53 FR 20397). The
supplemental information submitted by the licensee on June 7, 1988 was
editorial in nature and did not affect the substance of the initial notice
or the staff's proposed no significant hazards consideration determination.
The Commission's related evaluation of the amendments is contained in a Safety
Evaluation dated July 12, 1988.

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Sholly Coordinator

- 2 -

No significant hazards consideration comments received: No.

Local Public Document Room Location: Appling County Public Library,
301 City Hall Drive, Baxley, Georgia 31513

Original signed by:

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

DISTRIBUTION:

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We find the requested changes acceptable as the new position of General Manager-Nuclear Plant is the position that provides direct executive oversight over all aspects of the Hatch Plant.

4. GPC requested the revision of Section 5 to delete Figure 5.2-1 and change the reference in this section from Senior Executive Vice President to Vice President-Nuclear.

We find these requested changes acceptable as they conform to Generic Letter 88-06 and the new designation of responsibilities.

On the basis of its review as discussed above, the NRC staff concludes that the licensee has provided an acceptable response to these items as addressed in the NRC guidance on removing organization charts from the administrative control requirements of the TS. Furthermore, the NRC staff finds that these changes are consistent with the staff's generic finding on the acceptability of such changes as noted in GL 88-06. Accordingly, the NRC staff finds the TS changes to be acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration as published on June 3, 1988 in the Federal Register (53 FR 20397), and consulted with the State of Georgia. No public comments were received, and the State of Georgia did not have any comments.

On the basis of the considerations discussed above, the staff concludes that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: F. Allenspach, LPEB/DLPQ
L. Crocker, PDII-3/DRP-I/II

Dated: July 12, 1988