

Dockets Nos.: 50-321
and 50-366

August 10, 1987

Mr. James P. O'Reilly
Senior Vice President - Nuclear Operations
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. O'Reilly:

Subject: Issuance of Amendment Nos. 145 and 80 to Facility Operating Licenses DPR-57 and NPF-5 - Edwin I. Hatch Nuclear Plant, Units 1 and 2 (TACS 61637/61638/60982/60983)

The Commission has issued the enclosed Amendments Nos. 145 and 80 to Facility Operating Licenses DPR-57 and NPF-5, for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your applications dated February 17, May 16, August 27, October 27, and November 24, 1986.

The amendments modify the Technical Specifications to reflect organization changes.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

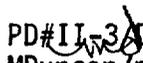


Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

Enclosures:

1. Amendment No. 145 to DPR-57
2. Amendment No. 80 to NPF-5
3. Safety Evaluation

cc w/enclosures:
See next page

 PD#II-3/DRP-I/II
MDuncan/mac
07/27/87

 PD#II-3/DRP-I/II
LCrocker
07/23/87

 PD#II-3/DRP-I/II
for BJYoungblood
08/10/87

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PDR
P

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 145, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

15)
B. J. Youngblood, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 10, 1987

only 5/8/87

~~*no file 7/29/87*~~

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OGC-Bethesda
in K. G. L. man for
07/23/87

me
PD#II-3/DRP-I/II
BJYoungblood
08/10/87

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. ⁸⁰, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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B. J. Youngblood, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 10, 1987

Handwritten signature and date: 2/29/87

Handwritten notes: no note 1/29/87

PD#II-3/DRP-I/II
MDuncan/mac
07/23/87

Handwritten initials: jrb
PD#II-3/DRP-I/II
LCrocker
07/23/87

OGC-Bethesda
Handwritten signature: J. Korman
07/27/87

PD#II-3/DRP-I/II
BJYoungblood
08/10/87

DATED August 10, 1987

AMENDMENT NO. 145 TO FACILITY OPERATING LICENSE DPR-57, EDWIN I. HATCH, UNITS 1 & 2
AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NPF-05, EDWIN I. HATCH, UNITS 1 & 2

DISTRIBUTION:

Docket File

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 10, 1987

Dockets Nos.: 50-321
and 50-366

Mr. James P. O'Reilly
Senior Vice President - Nuclear Operations
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. O'Reilly:

Subject: Issuance of Amendment Nos. 145 and 80 to Facility Operating
Licenses DPR-57 and NPF-5 - Edwin I. Hatch Nuclear Plant,
Units 1 and 2 (TACS 61637/61638/60982/60983)

The Commission has issued the enclosed Amendments Nos. 145 and 80 to Facility Operating Licenses DPR-57 and NPF-5, for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your applications dated February 17, May 16, August 27, October 27, and November 24, 1986.

The amendments modify the Technical Specifications to reflect organization changes.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

A handwritten signature in cursive script that reads "Lawrence P. Crocker".

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

Enclosures:

1. Amendment No. 145 to DPR-57
2. Amendment No. 80 to NPF-5
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. James P. O'Reilly
Georgia Power Company

Edwin I. Hatch Nuclear Plant,
Units Nos. 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-321
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 145
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated February 17, May 16, August 27, October 27, and November 24, 1986, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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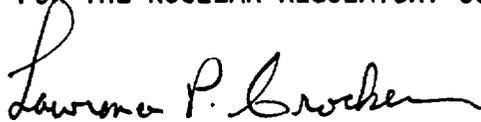
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 145, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for B. J. Youngblood, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 10, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 145

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove Page</u>	<u>Insert Page</u>
6-1	6-1
6-1a	6-1a
6-6	6-6
6-7	6-7
6-8	6-8
6-9	6-9
6-10	6-10
6-11	6-11
6-12	6-12
6-13	6-13
6-13a	6-13a

Replace the following pages of the Appendix B Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove Page</u>	<u>Insert Page</u>
1	i
5-1	5-1
5-2	5-2
5-3	5-3
5-4	5-4
5-5	5-5
5-7	5-7
5-6	5-6
5-9	5-9

6.1 RESPONSIBILITY

6.1.0 The Vice President-Plant Hatch shall provide direct executive oversight over all aspects of Plant Hatch.

6.1.1 The Plant Manager shall be responsible for overall unit operation, except for the Radiological Environmental Monitoring Program as described below and for delegation in writing of the succession of this responsibility during his absence. Certain plant support functions shall be the responsibility of the Plant Support Manager.

6.1.2 The Manager Radiological Safety shall be responsible for the Radiological Environmental Monitoring Program as described in the Specification 3/4.16 of Unit 1 and for the writing of the Annual Radiological Environmental Surveillance Report. The Manager Radiological Safety shall review the proposed changes to plant systems or equipment, provided that such changes are identified by the Plant Review Board (PRB) as having a potential radiological environmental impact.

6.1.3 Each of the above-mentioned individuals is responsible for the accuracy of the procedures needed to implement his responsibilities.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for unit management and technical support shall be as shown in Chapter 13 of the Plant Hatch Unit 2 updated Final Safety Analysis Report.

UNIT STAFF

6.2.2 The unit organization shall be as shown in Chapter 13 of the Plant Hatch Unit 2 updated Final Safety Analysis Report.

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.2-1.
- b. At least one licensed Operators shall be in the control room for each reactor containing fuel.
- c. At least two licensed Operators shall be present in the control room for each reactor in the process of start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified to implement radiation protection procedures shall be on site when fuel is in either reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

6.0 ADMINISTRATIVE CONTROLS

- f. A Fire Team of at least five members shall be maintained onsite at all times. The Fire Team shall not include the minimum shift crew necessary for safe shutdown on Units 1 and 2 or any personnel required for other essential functions during a fire emergency.
- g. Administrative procedures shall be developed and implemented to limit the working hours of Unit staff who perform safety-related functions; e.g., senior reactor operators, reactor operators, auxiliary operators, health physicists, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the plant is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used or during extended periods of shutdown for refueling, major maintenance, or major plant modifications, the following guidelines shall be followed on a temporary basis:

- (1) An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
- (2) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven day period, all excluding shift turnover time.
- (3) A break of at least eight hours should be allowed between work periods, including shift turnover time.
- (4) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Plant Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1. Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Health Physics Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1. A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager of Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2. A training program for fire protection shall be maintained under the direction of the Senior Regulatory Specialist and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for fire protection training sessions which shall be held at least once per 92 days.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1. The PRB shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2. The PRB shall be composed of, as a minimum, a supervisor or higher level individual from each of the departments listed below:

Operations
Maintenance
Quality Control (QC)
Health Physics
Nuclear Safety and Compliance
Engineering Support

The Chairman, his alternate, and other members of the PRB shall be designated by the Plant Manager. The Chairman and his designated alternate shall both be managers of one of the six above listed departments or a higher level onsite manager.

ALTERNATES

6.5.1.3. All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and three voting members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Review Board shall be responsible for:

- a. Review of all procedures required by Specification 6.8 and changes thereto, except those for the Radiological Environmental Monitoring Program, any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all reportable violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President-Plant Hatch, the Senior Vice President-Nuclear Operations, and to the Safety Review Board (SRB).
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of unit operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the SRB.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- i. Review of the Security Plan and implementing procedures and shall submit recommended changes to the SRB.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the SRB.
- k. Review of any unplanned onsite release of radioactive material to the environs when such release is in excess of 1 Ci, excluding dissolved and entrained gases and tritium for liquid effluents, and in excess of 150 Ci of noble gases or 0.02 Ci of radiiodines for gaseous effluents. Also included is the preparing and forwarding to the Plant Manager and the SRB reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL (ODCM), except for the section on the Radiological Environmental Monitoring Program in the ODCM.
- m. Review of proposed change(s) to plant systems and equipment to determine whether the proposed change has a potential radiological environmental impact. Such change(s) will be reported to the Manager-Radiological Safety.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Recommend in writing to the Plant Manager approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Plant Hatch, the Senior Vice President-Nuclear Operations, and the Safety Review Board of disagreement between the PRB and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Review Board shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Vice President-Plant Hatch, the Senior Vice President-Nuclear Operations, and the Safety Review Board.

ADMINISTRATIVE CONTROLS

6.5.2. SAFETY REVIEW BOARD (SRB)

FUNCTION

6.5.2.1. The SRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations.
- b. Nuclear engineering.
- c. Chemistry and radiochemistry.
- d. Metallurgy.
- e. Instrumentation and control.
- f. Radiological safety.
- g. Mechanical and electrical engineering.
- h. Quality assurance practices.

COMPOSITION

6.5.2.2. The SRB shall be composed of a minimum of five persons who as a group provide the expertise to review and audit the operation of a nuclear power plant. The Chairman and other members shall be appointed by the Senior Vice President - Nuclear Operations or such other person as he shall designate. The composition of the SRB shall meet the requirements of ANSI N18.7-1976.

ALTERNATES

6.5.2.3. All alternate members shall be appointed in writing by the SRB Chairman to serve on a temporary basis; however, no more than a minority of alternates shall participate as voting members in SRB activities at any one time. However, in extenuating circumstances, the Senior Vice President - Nuclear Operations may designate the use of additional alternates with voting authority when regular members are not available within necessary time constraints.

CONSULTANTS

6.5.2.4. Consultants shall be utilized as determined by the SRB Chairman to provide expert advice to the SRB.

MEETING FREQUENCY

6.5.2.5. The SRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.2.6. The minimum quorum of the SRB necessary for the performance of the SRB review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least a majority of the members. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7. The SRB shall be responsible for the review of:

- a. The safety evaluations for (1) changes to procedures, equipment, or systems and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meeting minutes of the Plant Review Board.

ADMINISTRATIVE CONTROLS

AUDITS

6.5.2.8. Audits of unit activities shall be performed under the cognizance of the SRB. Each inspection or audit shall be performed within the specified time interval with:

1. A maximum allowable extension not to exceed 25% of the inspection or audit interval.
2. A total maximum combined interval time for any 3 consecutive inspection or audit intervals not to exceed 3.25 times the specified inspection or audit interval.

These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once 24 months.
- e. The Emergency Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.
- f. The Security Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.
- g. Any other area of unit operation considered appropriate by the SRB or the Senior Vice President - Nuclear Operations.
- h. The Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite license personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years. During the year in which the inspection or audit occurs, the requirements of 6.5.2.8.i. can be affected concurrently.

- k. The Radiological Environmental Monitoring Program and the results thereof annually.
- l. The Offsite Dose Calculation Manual, Process Control Program, and implementing procedures at least once per 24 months.

AUTHORITY

6.5.2.9 The SRB shall report to and advise the Senior Vice President - Nuclear Operations on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of SRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRB meeting shall be prepared, approved and forwarded to the Senior Vice President-Nuclear Operations within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Senior Vice President-Nuclear Operations within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Executive Vice President, the Senior Vice President-Nuclear Operations and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring notification to the Commission shall be reviewed by the PRB and submitted to the SRB, the Vice President-Plant Hatch, and the Senior Vice President-Nuclear Operations.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT SHUTDOWN within two hours.
- b. The Safety Limit violation shall be reported to the Commission, the Vice President-Plant Hatch, the Senior Vice President-Nuclear Operations, and the SRB within 24 hours.

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PRB. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the SRB, the Vice President-Plant Hatch, and the Senior Vice President-Nuclear Operations within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.

6.8.2 Each procedure of 6.8.1 and other procedures which the Plant Manager or Plant Support Manager has determined to affect nuclear safety, and changes thereto, shall be reviewed by the PRB and approved by the appropriate member of plant management, designated by the Plant Manager or Plant Support Manager, prior to implementation. The Plant Manager or Plant Support Manager will approve administrative procedures, security plan implementing procedures, and changes thereto. The Manager Plant Training and Onsite Emergency Preparedness shall approve the emergency plan implementing procedures and changes hereto. All other procedures of this specification and changes thereto will be approved by the department head designated by the Plant Manager or Plant Support Manager. The procedures of this specification shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.

ADMINISTRATIVE CONTROLS

- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PRB and approved in writing by the Plant Manager or Plant Support Manager, or his designee as assigned by 6.8.2, within 14 days of implementation.

6.8.4 Proposed changes to procedures for implementing the responsibilities specified in section 6.1.2 shall be reviewed and approved by the Manager Radiological Safety or his designee. When deemed appropriate by the Manager Radiological Safety, such proposed changes shall also be reviewed by the Safety Review Board prior to implementation.

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5.0 Administrative Controls

This section describes administrative and management controls established to implement the Environmental Technical Specifications (ETS). Measures specified in this section include assignments of responsibility, review and audit functions, procedures, and reporting requirements.

Corporate responsibility for implementing the ETS and for assuring that the station is operated in such a way as to provide protection for the environment rests with the Senior Executive Vice President.

Responsibilities for compliance with the ETS and for the environmental monitoring program required by the ETS are given below.

Independent audit shall be provided, as discussed in section 5.3.2, by the General Manager-Quality Assurance.

5.1 Responsibility

5.1.1 The Plant Manager is responsible for the environmental monitoring programs. The Plant Manager is also responsible for implementing the special surveillance activities described in section 4.

5.1.2 The General Manager-Quality Assurance is responsible for assuring that the periodic audits of plant operations and the environmental monitoring activities to ensure conformance with the ETS are conducted.

5.2 Organization

Company organization relative to environmental matters is presented in Chapter 13 of the Plant Hatch Unit 2 updated Final Safety Analysis Report.

5.3 Review and Audit

5.3.1 Independent Review

5.3.1.1 The Manager of Environmental Affairs shall review the following:

- a. Proposed changes to plant systems or equipment, provided such changes are identified by the Plant Review Board (PRB) as having a potential adverse environmental impact.
- b. Proposed changes to the Environmental Technical Specifications (ETS).

5.3.1.2 The Safety Review Board (SRB) shall review the following:

- a. Proposed changes to the ETS.
- b. Violations of ETS to determine whether adequate corrective action is being taken to prevent recurrence.
- c. Procedures or changes hereto, which could affect the monitoring of station operation, that may be considered by the Manager of Environmental Affairs or the PRB to be appropriate for SRB review.

5.3.1.4 The PRB shall review the following:

- a. Procedures for implementing the responsibilities specified in section 5.1.1 and proposed changes thereto.
- b. Proposed changes to the ETS.

5.3.2 Audit Responsibility

5.3.2.1 The General Manager-Quality Assurance is responsible for an audit, conducted annually, of the activities of the Plant Manager and the Manager-Environmental Affairs, related to compliance with ETS.

5.3.2.2 Audits of facility activities shall be performed annually under the cognizance of the SRB to ensure conformance of facility operation to provisions of the ETS.

5.4 State and Federal Permit and Certificates

Section 401 of PL 92-500, the Federal Water Pollution Control Act Amendments of 1972 (FWPCA), requires any applicant for a Federal license or permit to conduct any activity that may result in any discharge into provisions of Sections 301, 302, 306, and 307 of the FWPCA. Section 401 of PL 92-500 further requires that any certification provided under this section shall set any effluent limitations and other limitations and monitoring requirements necessary to assure that any applicant for a Federal license or permit will comply with the applicable limitations. Certifications provided in accordance with Section 401 set forth conditions on the Federal license or permit for which the certification is provided. Accordingly, the licensee shall comply with the requirements set forth in the currently applicable 401 certification and amendments thereto issued to the licensee by the Georgia Environmental Protection Division. In accordance with the provisions of the Georgia Water Quality Control Act, the FWPCA and the rules and regulations promulgated pursuant to each of these acts, the Georgia Environmental Protection Division, under authority delegated by the U.S. EPA, issued NPDES permit No. GA 0004120 to the licensee. The NPDES permit authorizes the licensee to discharge from HNP Units 1 and 2 to the Altamaha River in accordance with effluent limitations, monitoring requirements, and other conditions stipulated in the permit, effective August 1, 1983, through December 5, 1987.

Subsequent revisions to the certifications will be accommodated in accordance with the provisions of section 5.6.3.

5.5 Procedures

Detailed written procedures, including applicable checklists and instructions, shall be prepared and followed for all activities involved in implementing the ETS. All procedures shall be maintained in a manner convenient for review and inspection. Procedures that are the responsibility of the Plant Manager shall be kept at the plant. Procedures that are the responsibility of the Manager-Environmental Affairs shall be kept at the Georgia Power Company General Office.

5.5.1 Quality Assurance of Program Results

Procedures shall be established to assure the quality of ETS program results, including analytical measurements. These procedures shall document the program in policy directives, designate responsible organizations or individuals, describe purchased services (e.g., contractual laboratory or other contract services), and provide for audits of results and procedures by licensee personnel. In addition, these quality assurance procedures shall provide for systems to identify and correct deficiencies in technical monitoring programs or related administrative activities, to investigate anomalous or suspect results, and to review and evaluate program results.

5.5.2 Compliance with Procedures

In addition to the procedures specified in Section 5.5, the station operating procedures shall include provisions to ensure that each Unit and all its systems and components are operated in compliance with the conditions established in the Environmental Technical Specifications (ETS).

5.5.3 Changes in Procedures and Station Design or Operation

Changes in the procedures and station design or operation may be made in accordance with Section 5.3 and subject to the conditions described below:

- a. The licensee may make changes in the stations design and operation and conduct tests and experiments without prior NRC approval, unless the proposed change, test or experiment involves either a change in the objectives of the ETS, an unreviewed environmental question of substantive impact.

- b. A proposed change, test, or experiment shall be deemed to involve an unreviewed environmental question if it concerns:
 - 1. A matter which may result in a significant increase in any adverse environmental impact previously evaluated in the final environmental statement, as modified by staff's testimony at the hearing, supplements thereto, environmental impact appraisals, or in initial or final adjudicatory decisions.
 - 2. A significant change in effluents or power level.
 - 3. A matter not previously reviewed and evaluated in the documents specified above which may have a significant adverse environmental impact.
- c. The licensee shall maintain records of changes to facility design or operation made pursuant to this section. The licensee also shall maintain records of tests and experiments carried out pursuant to paragraph (a) of this section. These records shall include a written evaluation which provides the bases for the determination that the change, test, or experiment does not involve an unreviewed environmental question of substantive impact, or does not constitute a change in the objectives of the ETS. The licensee shall furnish to the NRC, annually or at such shorter intervals as may be specified in the license, a report containing descriptions, analyses, interpretations, and evaluations of such changes, tests, and experiments.
- d. Proposed changes or modifications to plant systems or equipment shall be reviewed in accordance with Section 5.3.
- e. Proposed changes to procedures for implementing the responsibilities specified in Section 5.1.1 shall be reviewed and approved by the Plant Review Board (PRB). Temporary changes to the procedures that do not change the intent of the original procedure may be made with the concurrence of two members of the plant management staff, at least one of whom holds a Senior Reactor Operators license on the unit affected. Such changes shall be documented and subsequently reviewed by the PRB and approved by the Plant Manager within 14 days of implementation.

5.5.4 NRC Authority to Require Revisions

The NRC may require modification or revision of changes made by the licensee in accordance with section 5.5.3, as a result of NRC reviews of the results of these programs, if such modifications or revisions are judged necessary to maintain consistency with the initially approved program descriptions or with the intent of the ETS. The NRC also may require modifications or revisions because of changes in plant operation or changes in environmental conditions or concerns associated with plant operation.

5.6 Plant Reporting Requirements

5.6.1 Routine Reports

Annual Environmental Surveillance Report

A report on the environmental surveillance program for the previous calendar year shall be submitted to the NRC within 90 days after January 1 of each year. The report shall include

summaries, analyses, and interpretations or statistical evaluations, where appropriate, of the results of the environmental monitoring activities for the report period.

The Annual Environmental Surveillance Report also will include the following:

- a. Comparison with preoperational studies, with operational controls (as appropriate), and with previous environmental monitoring reports.
- b. An assessment of the observed impacts of plant operation on the environment.
- c. A summary of:
 1. All instances of Environmental Technical Specifications (ETS) noncompliance and corrective actions taken to remedy them.
 2. Changes to Federal and State permits and certificates made in accordance with 5.6.3.
 3. Changes in station design or operation that could involve an environmental impact or change in the findings of the final environmental statement.
 4. Changes in the ETS.
 5. Copies of all reports regarding station discharges made in accordance with NPDES permit No GA 0004120 (and subsequent revisions); these shall include reports made in accordance with Parts 1B and III of the NPDES permit.

If harmful effects or evidence of irreversible damage are detected by monitoring, the licensee shall provide a further analysis of the problem and a proposed course of action to alleviate the problem.

Results of analysis of all nonradiological environmental data collected shall be summarized and tabulated on an annual basis. In the event that some results are not available within 90 days after January 1, the report shall be submitted, noting and explaining the missing results. The missing data shall be submitted as soon thereafter as possible in a supplementary report.

Proposed changes to the ETS shall be reviewed and approved by the Manager-Environmental Affairs, the Plant Review Board, and the Safety Review Board. Prior to approval, the possible impact of the proposed changes will be evaluated.

5.6.3.2 Changes in Permits and Certificates

Changes or additions to required Federal, State, local, and regional authority permits and certificates for the protection of the environment that pertain to the requirements of the ETS shall be reported to the NRC within 30 days. In the event that the licensee initiates or becomes aware of a request for changes to any water quality requirements, limits, or values stipulated in any certificate or permit issued pursuant to Section 401 or 402 of PL 92-500, which are also the subject of an ETS reporting requirement, the NRC shall be notified concurrently with the authorizing agency. The notification to the NRC shall include an evaluation of the environmental impact of the revised requirement, limit, or value being sought.

If, during the NRC's review of the proposed change, it is determined that a potentially severe environmental impact could result from the change, the NRC will consult with the authorizing agency to determine the appropriate action to be taken.

5.7 Records Retention

5.7.1 Records and logs relative to the following areas shall be made and retained for the life of the plant in a manner convenient for review and inspection. These logs shall be made available to the NRC on request.

- a. Records and drawings detailing plant design changes and modifications made to systems and equipment as described in section 5.5.3.
- b. Records of all data from environmental monitoring and surveillance programs required by the ETS.

5.7.2 All other records and logs relating to the ETS shall be retained, in a manner convenient for review and inspection, for 5 years following logging or recording.

5.7.3 These records shall be stored at the plant or at the Georgia Power Company General Office, as appropriate, under the control of the responsible organization.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-366
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated February 17, May 16, August 27, October 27, and November 24, 1986, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 80, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for B. J. Youngblood, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 10, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 80

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove Page</u>	<u>Insert Page</u>
5-1	5-1
5-2	5-2
5-3	5-3
6-1	6-1
6-1a	6-1a
6-5	6-5
6-6	6-6
6-7	6-7
6-8	6-8
6-9	6-9
6-10	6-10
6-11	6-11
6-12	6-12
6-12a	6-12a

Replace the following pages of the Appendix B Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove Page</u>	<u>Insert Page</u>
i	i
5-1	5-1
5-2	5-2
5-3	5-3
5-4	5-4
5-5	5-5
5-6	5-6
5-7	5-7
5-9	5-9

5.0 DESIGN FEATURES

5.1 SITE

EXCLUSION AREA

5.1.1 The exclusion area shall be as shown in Figure 3.11-1.

LOW POPULATION ZONE

5.1.2 The low population zone coincides with the exclusion area and is also shown in Figure 3.11-1.

5.2 CONTAINMENT

CONFIGURATION

5.2.1 The primary containment is a steel structure composed of a series of vertical right cylinders and truncated cones which form a drywell. This drywell is attached to a suppression chamber through a series of vents. The suppression chamber is a steel pressure vessel in the shape of a torus. The primary containment has a total minimum free air volume of 255,978 cubic feet.

DESIGN TEMPERATURE AND PRESSURE

5.2.2 The primary containment is designed and shall be maintained for:

- a. Maximum design internal pressure 56 psig.
- b. Maximum allowable internal pressure 62 psig.
- c. Maximum internal temperature 340°F.
- d. Maximum external pressure 2 psig.

5.3 REACTOR CORE

FUEL ASSEMBLIES

5.3.1 The core shall consist of not more than 560 fuel assemblies and shall be limited to those fuel assemblies which have been analyzed with NRC approved codes and methods and have been shown to comply with all Safety Design Bases in the Final Safety Analysis Report (FSAR).

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DESIGN FEATURES

CONTROL ROD ASSEMBLIES

5.3.2 The reactor core shall contain 137 cruciform-shaped control rod assemblies.

5.4 REACTOR COOLANT SYSTEM

DESIGN PRESSURE AND TEMPERATURE

5.4.1 The reactor coolant system is designed and shall be maintained:

- a. In accordance with the code requirements specified in Section 5.2 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
- b. For a pressure of 1250 psig, and
- c. For a temperature of 575°F.

VOLUME

5.4.2 The total water and steam volume of the reactor vessel and recirculation system is approximately 17,050 cubic feet at a nominal Tave of 540°F.

5.5 METEOROLOGICAL TOWER LOCATION

5.5.1 The primary and backup meteorological towers shall be located as shown on Figure 3.11-1.

5.6 FUEL STORAGE

CRITICALITY

5.6.1 The new and spent fuel storage racks are designed and shall be maintained with sufficient center-to-center distance between fuel assemblies placed in the storage racks to ensure a k_{eff} equivalent to ≤ 0.95 when flooded with unborated water. The k_{eff} of ≤ 0.95 includes conservative allowances for uncertainties.

6.1 RESPONSIBILITY

6.1.0 The Vice President-Plant Hatch shall provide direct executive oversight over all aspects of Plant Hatch.

6.1.1 The Plant Manager shall be responsible for overall unit operation, except for the Radiological Environmental Monitoring Program as described below and for delegation in writing of the succession of this responsibility during his absence. Certain plant support functions shall be the responsibility of the Plant Support Manager.

6.1.2 The Manager Radiological Safety shall be responsible for the Radiological Environmental Monitoring Program as described in the Specification 3/4.16 of Unit 1 and for the writing of the Annual Radiological Environmental Surveillance Report. The Manager Radiological Safety shall review the proposed changes to plant systems or equipment, provided that such changes are identified by the Plant Review Board (PRB) as having a potential radiological environmental impact.

6.1.3 Each of the above-mentioned individuals is responsible for the accuracy of the procedures needed to implement his responsibilities.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for unit management and technical support shall be as shown in Chapter 13 of the Plant Hatch Unit 2 updated Final Safety Analysis Report.

UNIT STAFF

6.2.2 The unit organization shall be as shown in Chapter 13 of the Plant Hatch Unit 2 updated Final Safety Analysis Report.

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.2-1.
- b. At least one licensed Operators shall be in the control room for each reactor containing fuel.
- c. At least two licensed Operators shall be present in the control room for each reactor in the process of start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified to implement radiation protection procedures shall be on site when fuel is in either reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

ADMINISTRATIVE CONTROLS

- f. A Fire Team of at least five members shall be maintained onsite at all times. The Fire Team shall not include the minimum shift crew necessary for safe shutdown on Units 1 and 2 or any personnel required for other essential functions during a fire emergency.
- g. Administrative procedures shall be developed and implemented to limit the working hours of Unit staff who perform safety-related functions; e.g., senior reactor operators, reactor operators, auxiliary operators, health physicists, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the plant is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used or during extended periods of shutdown for refueling, major maintenance, or major plant modifications, the following guidelines shall be followed on a temporary basis:

- (1) An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
- (2) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven day period, all excluding shift turnover time.
- (3) A break of at least eight hours should be allowed between work periods, including shift turnover time.
- (4) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Plant Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1. Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Health Physics Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1. A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager of Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2. A training program for fire protection shall be maintained under the direction of the Senior Regulatory Specialist and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for fire protection training sessions which shall be held at least once per 92 days.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1. The PRB shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2. The PRB shall be composed of, as a minimum, a supervisor or higher level individual from each of the departments listed below:

- Operations
- Maintenance
- Quality Control (QC)
- Health Physics
- Nuclear Safety and Compliance
- Engineering Support

The Chairman, his alternate, and other members of the PRB shall be designated by the Plant Manager. The Chairman and his designated alternate shall both be managers of one of the six above listed departments or a higher level onsite manager.

ALTERNATES

6.5.1.3. All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and three voting members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Review Board shall be responsible for:

- a. Review of all procedures required by Specification 6.8 and changes thereto, except those for the Radiological Environmental Monitoring Program, any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all reportable violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President-Plant Hatch, the Senior Vice President-Nuclear Operations, and to the Safety Review Board (SRB).
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of unit operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the SRB.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- i. Review of the Security Plan and implementing procedures and shall submit recommended changes to the SRB.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the SRB.
- k. Review of any unplanned onsite release of radioactive material to the environs when such release is in excess of 1 Ci, excluding dissolved and entrained gases and tritium for liquid effluents, and in excess of 150 Ci of noble gases or 0.02 Ci of radioiodines for gaseous effluents. Also included is the preparing and forwarding to the Plant Manager and the SRB reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL (ODCM), except for the section on the Radiological Environmental Monitoring Program in the ODCM.
- m. Review of proposed change(s) to plant systems and equipment to determine whether the proposed change has a potential radiological environmental impact. Such change(s) will be reported to the Manager-Radiological Safety.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Recommend in writing to the Plant Manager approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Plant Hatch, the Senior Vice President-Nuclear Operations, and the Safety Review Board of disagreement between the PRB and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Review Board shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Vice President-Plant Hatch, the Senior Vice President-Nuclear Operations, and the Safety Review Board.

ADMINISTRATIVE CONTROLS

6.5.2. SAFETY REVIEW BOARD (SRB)

FUNCTION

6.5.2.1. The SRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations.
- b. Nuclear engineering.
- c. Chemistry and radiochemistry.
- d. Metallurgy.
- e. Instrumentation and control.
- f. Radiological safety.
- g. Mechanical and electrical engineering.
- h. Quality assurance practices.

COMPOSITION

6.5.2.2. The SRB shall be composed of a minimum of five persons who as a group provide the expertise to review and audit the operation of a nuclear power plant. The Chairman and other members shall be appointed by the Senior Vice President - Nuclear Operations or such other person as he shall designate. The composition of the SRB shall meet the requirements of ANSI N18.7-1976.

ALTERNATES

6.5.2.3. All alternate members shall be appointed in writing by the SRB Chairman to serve on a temporary basis; however, no more than a minority of alternates shall participate as voting members in SRB activities at any one time. However, in extenuating circumstances, the Senior Vice President - Nuclear Operations may designate the use of additional alternates with voting authority when regular members are not available within necessary time constraints.

CONSULTANTS

6.5.2.4. Consultants shall be utilized as determined by the SRB Chairman to provide expert advice to the SRB.

MEETING FREQUENCY

6.5.2.5. The SRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.2.6. The minimum quorum of the SRB necessary for the performance of the SRB review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least a majority of the members. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7. The SRB shall be responsible for the review of:

- a. The safety evaluations for (1) changes to procedures, equipment, or systems and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meeting minutes of the Plant Review Board.

ADMINISTRATIVE CONTROLS

AUDITS

6.5.2.8. Audits of unit activities shall be performed under the cognizance of the SRB. Each inspection or audit shall be performed within the specified time interval with:

1. A maximum allowable extension not to exceed 25% of the inspection or audit interval.
2. A total maximum combined interval time for any 3 consecutive inspection or audit intervals not to exceed 3.25 times the specified inspection or audit interval.

These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once 24 months.
- e. The Emergency Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.
- f. The Security Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.
- g. Any other area of unit operation considered appropriate by the SRB or the Senior Vice President - Nuclear Operations.
- h. The Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite license personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years. During the year in which the inspection or audit occurs, the requirements of 6.5.2.8.i. can be affected concurrently.

- k. The Radiological Environmental Monitoring Program and the results thereof annually.
- l. The Offsite Dose Calculation Manual, Process Control Program, and implementing procedures at least once per 24 months.

AUTHORITY

6.5.2.9 The SRB shall report to and advise the Senior Vice President - Nuclear Operations on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of SRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRB meeting shall be prepared, approved and forwarded to the Senior Vice President-Nuclear Operations within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Senior Vice President-Nuclear Operations within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Executive Vice President, the Senior Vice President-Nuclear Operations and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring notification to the Commission shall be reviewed by the PRB and submitted to the SRB, the Vice President-Plant Hatch, and the Senior Vice President-Nuclear Operations.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT SHUTDOWN within two hours.
- b. The Safety Limit violation shall be reported to the Commission, the Senior Vice President-Nuclear Operations, and the SRB within 24 hours.

SAFETY LIMIT VIOLATION (Continued)

- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PRB. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the SRB, the Vice President-Plant Hatch, and the Senior Vice President-Nuclear Operations within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.

6.8.2 Each procedure of 6.8.1 and other procedures which the Plant Manager or Plant Support Manager has determined to affect nuclear safety, and changes thereto, shall be reviewed by the PRB and approved by the appropriate member of plant management, designated by the Plant Manager or Plant Support Manager, prior to implementation. The Plant Manager or Plant Support Manager will approve administrative procedures, security plan implementing procedures, and changes thereto. The Manager Plant Training and Onsite Emergency Preparedness shall approve the emergency plan implementing procedures and changes thereto. All other procedures of this specification and changes thereto will be approved by the department head designated by the Plant Manager or Plant Support Manager. The procedures of this specification shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.

ADMINISTRATIVE CONTROLS

- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PRB and approved in writing by the Plant Manager or Plant Support Manager, or his designee as assigned by 6.8.2, within 14 days of implementation.

6.8.4 Proposed changes to procedures for implementing the responsibilities specified in section 6.1.2 shall be reviewed and approved by the Manager Radiological Safety or his designee. When deemed appropriate by the Manager Radiological Safety, such proposed changes shall also be reviewed by the Safety Review Board prior to implementation.

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5.0 Administrative Controls

This section describes administrative and management controls established to implement the Environmental Technical Specifications (ETS). Measures specified in this section include assignments of responsibility, review and audit functions, procedures, and reporting requirements.

Corporate responsibility for implementing the ETS and for assuring that the station is operated in such a way as to provide protection for the environment rests with the Senior Executive Vice President.

Responsibilities for compliance with the ETS and for the environmental monitoring program required by the ETS are given below.

Independent audit shall be provided, as discussed in section 5.3.2, by the General Manager-Quality Assurance.

5.1 Responsibility

5.1.1 The Plant Manager is responsible for the environmental monitoring programs. The Plant Manager is also responsible for implementing the special surveillance activities described in section 4.

5.1.2 The General Manager-Quality Assurance is responsible for assuring that the periodic audits of plant operations and the environmental monitoring activities to ensure conformance with the ETS are conducted.

5.2 Organization

Company organization relative to environmental matters is presented in Chapter 13 of the Plant Hatch Unit 2 updated Final Safety Analysis Report.

5.3 Review and Audit

5.3.1 Independent Review

5.3.1.1 The Manager of Environmental Affairs shall review the following:

- a. Proposed changes to plant systems or equipment, provided such changes are identified by the Plant Review Board (PRB) as having a potential adverse environmental impact.
- b. Proposed changes to the Environmental Technical Specifications (ETS).

5.3.1.2 The Safety Review Board (SRB) shall review the following:

- a. Proposed changes to the ETS.
- b. Violations of ETS to determine whether adequate corrective action is being taken to prevent recurrence.
- c. Procedures or changes hereto, which could affect the monitoring of station operation, that may be considered by the Manager of Environmental Affairs or the PRB to be appropriate for SRB review.

5.3.1.4 The PRB shall review the following:

- a. Procedures for implementing the responsibilities specified in section 5.1.1 and proposed changes thereto.
- b. Proposed changes to the ETS.

5.3.2 Audit Responsibility

5.3.2.1 The General Manager-Quality Assurance is responsible for an audit, conducted annually, of the activities of the Plant Manager and the Manager-Environmental Affairs, related to compliance with ETS.

5.3.2.2 Audits of facility activities shall be performed annually under the cognizance of the SRB to ensure conformance of facility operation to provisions of the ETS.

5.4 State and Federal Permit and Certificates

Section 401 of PL 92-500, the Federal Water Pollution Control Act Amendments of 1972 (FWPCA), requires any applicant for a Federal license or permit to conduct any activity that may result in any discharge into provisions of Sections 301, 302, 306, and 307 of the FWPCA. Section 401 of PL 92-500 further requires that any certification provided under this section shall set any effluent limitations and other limitations and monitoring requirements necessary to assure that any applicant for a Federal license or permit will comply with the applicable limitations. Certifications provided in accordance with Section 401 set forth conditions on the Federal license or permit for which the certification is provided. Accordingly, the licensee shall comply with the requirements set forth in the currently applicable 401 certification and amendments thereto issued to the licensee by the Georgia Environmental Protection Division. In accordance with the provisions of the Georgia Water Quality Control Act, the FWPCA and the rules and regulations promulgated pursuant to each of these acts, the Georgia Environmental Protection Division, under authority delegated by the U.S. EPA, issued NPDES permit No. GA 0004120 to the licensee. The NPDES permit authorizes the licensee to discharge from HNP Units 1 and 2 to the Altamaha River in accordance with effluent limitations, monitoring requirements, and other conditions stipulated in the permit, effective August 1, 1983, through December 5, 1987.

Subsequent revisions to the certifications will be accommodated in accordance with the provisions of section 5.6.3.

5.5 Procedures

Detailed written procedures, including applicable checklists and instructions, shall be prepared and followed for all activities involved in implementing the ETS. All procedures shall be maintained in a manner convenient for review and inspection. Procedures that are the responsibility of the Plant Manager shall be kept at the plant. Procedures that are the responsibility of the Manager-Environmental Affairs shall be kept at the Georgia Power Company General Office.

5.5.1 Quality Assurance of Program Results

Procedures shall be established to assure the quality of ETS program results, including analytical measurements. These procedures shall document the program in policy directives, designate responsible organizations or individuals, describe purchased services (e.g., contractual laboratory or other contract services), and provide for audits of results and procedures by licensee and personnel. In addition, these quality assurance procedures shall provide for systems to identify and correct deficiencies in technical monitoring programs or related administrative activities, to investigate anomalous or suspect results, and to review and evaluate program results.

5.5.2 Compliance with Procedures

In addition to the procedures specified in Section 5.5, the station operating procedures shall include provisions to ensure that each Unit and all its systems and components are operated in compliance with the conditions established in the Environmental Technical Specifications (ETS).

5.5.3 Changes in Procedures and Station Design or Operation

Changes in the procedures and station design or operation may be made in accordance with Section 5.3 and subject to the conditions described below:

- a. The licensee may make changes in the stations design and operation and conduct tests and experiments without prior NRC approval, unless the proposed change, test or experiment involves either a change in the objectives of the ETS, an unreviewed environmental question of substantive impact.

- b. A proposed change, test, or experiment shall be deemed to involve an unreviewed environmental question if it concerns:
1. A matter which may result in a significant increase in any adverse environmental impact previously evaluated in the final environmental statement, as modified by staff's testimony at the hearing, supplements thereto, environmental impact appraisals, or in initial or final adjudicatory decisions.
 2. A significant change in effluents or power level.
 3. A matter not previously reviewed and evaluated in the documents specified above which may have a significant adverse environmental impact.
- c. The licensee shall maintain records of changes to facility design or operation made pursuant to this section. The licensee also shall maintain records of tests and experiments carried out pursuant to paragraph (a) of this section. These records shall include a written evaluation which provides the bases for the determination that the change, test, or experiment does not involve an unreviewed environmental question of substantive impact, or does not constitute a change in the objectives of the ETS. The licensee shall furnish to the NRC, annually or at such shorter intervals as may be specified in the license, a report containing descriptions, analyses, interpretations, and evaluations of such changes, tests, and experiments.
- d. Proposed changes or modifications to plant systems or equipment shall be reviewed in accordance with Section 5.3.
- e. Proposed changes to procedures for implementing the responsibilities specified in Section 5.1.1 shall be reviewed and approved by the Plant Review Board (PRB). Temporary changes to the procedures that do not change the intent of the original procedure may be made with the concurrence of two members of the plant management staff, at least one of whom holds a Senior Reactor Operators license on the unit affected. Such changes shall be documented and subsequently reviewed by the PRB and approved by the Plant Manager within 14 days of implementation.

5.5.4 NRC Authority to Require Revisions

The NRC may require modification or revision of changes made by the licensee in accordance with section 5.5.3, as a result of NRC reviews of the results of these programs, if such modifications or revisions are judged necessary to maintain consistency with the initially approved program descriptions or with the intent of the ETS. The NRC also may require modifications or revisions because of changes in plant operation or changes in environmental conditions or concerns associated with plant operation.

5.6 Plant Reporting Requirements

5.6.1 Routine Reports

Annual Environmental Surveillance Report

A report on the environmental surveillance program for the previous calendar year shall be submitted to the NRC within 90 days after January 1 of each year. The report shall include

summaries, analyses, and interpretations or statistical evaluations, where appropriate, of the results of the environmental monitoring activities for the report period.

The Annual Environmental Surveillance Report also will include the following:

- a. Comparison with preoperational studies, with operational controls (as appropriate), and with previous environmental monitoring reports.
- b. An assessment of the observed impacts of plant operation on the environment.
- c. A summary of:
 1. All instances of Environmental Technical Specifications (ETS) noncompliance and corrective actions taken to remedy them.
 2. Changes to Federal and State permits and certificates made in accordance with 5.6.3.
 3. Changes in station design or operation that could involve an environmental impact or change in the findings of the final environmental statement.
 4. Changes in the ETS.
 5. Copies of all reports regarding station discharges made in accordance with NPDES permit No GA 0004120 (and subsequent revisions); these shall include reports made in accordance with Parts 1B and III of the NPDES permit.

If harmful effects or evidence of irreversible damage are detected by monitoring, the licensee shall provide a further analysis of the problem and a proposed course of action to alleviate the problem.

Results of analysis of all nonradiological environmental data collected shall be summarized and tabulated on an annual basis. In the event that some results are not available within 90 days after January 1, the report shall be submitted, noting and explaining the missing results. The missing data shall be submitted as soon thereafter as possible in a supplementary report.

Proposed changes to the ETS shall be reviewed and approved by the Manager-Environmental Affairs, the Plant Review Board, and the Safety Review Board. Prior to approval, the possible impact of the proposed changes will be evaluated.

5.6.3.2 Changes in Permits and Certificates

Changes or additions to required Federal, State, local, and regional authority permits and certificates for the protection of the environment that pertain to the requirements of the ETS shall be reported to the NRC within 30 days. In the event that the licensee initiates or becomes aware of a request for changes to any water quality requirements, limits, or values stipulated in any certificate or permit issued pursuant to Section 401 or 402 of PL 92-500, which are also the subject of an ETS reporting requirement, the NRC shall be notified concurrently with the authorizing agency. The notification to the NRC shall include an evaluation of the environmental impact of the revised requirement, limit, or value being sought.

If, during the NRC's review of the proposed change, it is determined that a potentially severe environmental impact could result from the change, the NRC will consult with the authorizing agency to determine the appropriate action to be taken.

5.7 Records Retention

5.7.1 Records and logs relative to the following areas shall be made and retained for the life of the plant in a manner convenient for review and inspection. These logs shall be made available to the NRC on request.

- a. Records and drawings detailing plant design changes and modifications made to systems and equipment as described in section 5.5.3.
- b. Records of all data from environmental monitoring and surveillance programs required by the ETS.

5.7.2 All other records and logs relating to the ETS shall be retained, in a manner convenient for review and inspection, for 5 years following logging or recording.

5.7.3 These records shall be stored at the plant or at the Georgia Power Company General Office, as appropriate, under the control of the responsible organization.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 145 AND 80 TO
FACILITY OPERATING LICENSES DPR-57 AND NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-321 AND 50-366

INTRODUCTION

By letters dated February 17, 1986 (Reference 1), May 16, 1986 (Reference 2), August 27, 1986 (Reference 3), October 27, 1986 (Reference 4), and November 24, 1986 (Reference 5), the Georgia Power Company (GPC, the licensee) requested changes to the Technical Specifications (TS) for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The requested changes would amend: (1) Section 5.0, Design Features, of the Hatch Unit 2 Appendix A Technical Specification; (2) Section 6.0, Administrative Controls, of the Hatch Units 1 and 2 Appendix A Technical Specification; and (3) Section 5.0, Administrative controls, of the Hatch Units 1 and 2 Appendix B Technical Specifications. The organizational changes requested in References 1-4 reflected a continuing evolution of the licensee's organization. Reference 5 provided a clarification of the qualifications and minimum plant department representation on the revised Plant Review Board that had been requested by Reference 2. However, it did not change the request for the Technical Specification amendment as described in Reference 2.

Specifically, the proposed changes would:

1. Revise Section 5, Design Features, of the Hatch Unit 2 Appendix A TS to delete Figure 5.1.1-1 and change the references in Specifications 5.1.2 and 5.5.1 from Figure 5.1.1-1 to Figure 3.11-1.
2. Revise Section 6.1, Responsibility, of the Appendix A TS for Hatch Units 1 and 2 to reflect the addition of an onsite executive position and several title and responsibility changes.
3. Revise Section 6.2, Organization, of the Appendix A TS for Hatch Units 1 and 2 to amend Figure 6.2.1-1 to reflect changes to the offsite and onsite organizations. The October 27, 1986 letter (Reference 4) also proposed to delete Figures 6.2.1-1 and 6.2.2-1 from the Appendix A TS.
4. Revise Section 6.5, Review and Audit, of the Appendix A TS for Hatch Units 1 and 2 to change the membership, quorum requirements, and responsibilities of the Plant Review Board and the Safety Review Board.

5. Revise Section 6.6, Reportable Occurrence Action, of the Appendix A TS for Hatch Units 1 and 2 to make the reporting requirements consistent with the organization and title changes proposed in Sections 6.1 and 6.2 (items 2 and 3 above).
6. Revise Section 6.7, Safety Limit Violation, of the Appendix A TS for Hatch Units 1 and 2 to make the reporting requirements consistent with the organization and title changes proposed in Sections 6.1 and 6.2 (items 2 and 3 above).
7. Revise Section 6.8, Procedures, of the Appendix A TS for Hatch Units 1 and 2 to change approval authorities for procedures.
8. Revise Section 5, Administrative Controls, of the Appendix B TS for Hatch Units 1 and 2 to be consistent with the organization and title changes proposed in Sections 6.1 and 6.2 of the Appendix A TS (items 1 and 2 above). The October 27, 1986 letter (Reference 4) also proposed to delete Figure 5.2-1 from the Appendix B TS.

EVALUATION

1. Section 5.0, Design Features, Hatch Unit 2 Appendix A TS

- ° Figure 5.1.1-1 shows the exclusion area (area inside the site boundary), the low population zone (coincides with the exclusion area) and the location of the old meteorological tower. This same information is shown on Figure 3.11-1 in Section 3/4.11, Radioactive Effluents, of the TS. In addition, Figure 3.11-1 shows the location of the new meteorological tower and a recreation area that has been added adjacent to the site boundary for the use of Hatch employees and their families. The licensee proposes to delete Figure 5.1.1-1.
- ° Specifications 5.1.1 and 5.1.2 would be revised to reference Figure 3.11-1 instead of Figure 5.1.1-1.
- ° Specification 5.5.1 would be revised to state that the primary and backup meteorological tower are located as shown on Figure 3.11-1.

Elimination of Figure 5.1.1-1 removes the redundancy that presently exists in the TS between Figure 5.1.1-1 and Figure 3.11-1 and thus eliminates a potential source of conflict and confusion. There is no change to the physical boundary and thus no change to the exclusion area or low population zone. We find this deletion of the redundant Figure 5.1.1-1 and the change in references to Figure 3.11-1 to be acceptable. The new meteorological tower which is now the primary source of meteorological information provides an added capability. Retention of the old meteorological tower as an installed, operating backup assures that data from the new primary tower can be correlated with the historical meteorological data at the site. The new tower thus provides an enhanced capability and is acceptable.

2. Section 6.1, Responsibility, Units 1 and 2 Appendix A TS

- ° Specification 6.1.0 has been added to indicate the addition of the onsite executive position, the Vice President-Plant Hatch who shall provide direct executive oversight for all aspects of Plant Hatch.
- ° Specification 6.1.1 has been revised to reflect that the Plant Support Manager has responsibility for plant support functions.
- ° The former 6.1.2 and 6.1.3 specifications (new 6.1.2) have been combined to reflect the change in title and responsibilities of the Manager Radiological Safety as described below in Item 3.a. In addition, specification 6.1.4 has become the new 6.1.3 specification.

These changes should promote improved corporate control and support of plant operations and, since they meet the criteria of Sections 13.1.1 and 13.1.2 of NUREG-0800, the Standard Review Plan (SRP), are acceptable.

3. Section 6.2, Organization, Units 1 and 2 Appendix A TS

3.a Revisions to the Offsite Organization Chart, Figure 6.2.1-1

The offsite organization Figure 6.2.1-1, has been revised as follows:

- ° Addition of a new position, Senior Executive Vice President and deletion of the Executive Vice President Power Supply position. The position of Senior Executive Vice President who reports directly to the President GPC and is responsible for all activities related to power supply, is similar in function to the former position of Executive Vice President Power Supply. The new positions, Senior Vice President Nuclear Operations and the General Manager Quality Assurance, report to this position.
- ° Addition of a new position, Senior Vice President Nuclear Operations who has assumed all the functions for Plant Hatch formerly assigned to the Senior Vice President Nuclear Power position, which has been deleted. The new positions, Manager Nuclear Support and Manager Nuclear Performance, report to this position.
- ° Addition of a new position, Vice President Plant Hatch who is onsite and who reports directly to the Senior Vice President Nuclear Operations. This position maintains the line authority and responsibility previously held by the corporate position of Vice President and General Manager Nuclear Operations which has been deleted. In addition, the following new positions report to the Vice President Plant Hatch: Plant Manager; Plant Support Manager; Manager Plant Training and Onsite Emergency Preparedness; and Manager Nuclear Safety and Compliance.
- ° Addition of a new position, Manager Radiological Safety (formerly titled Manager of Nuclear Chemistry and Health Physics) who:

1. Has assumed responsibility for the Radiological Monitoring Program. The responsibility for this program was formerly divided between the General Manager-Plant Hatch, the Manager of Nuclear Engineering and Chief Nuclear Engineer, and the Manager of Environmental Affairs.
 2. Continues to have responsibility for the management of the Health Physics and Chemistry Program, and
 3. Has assumed the responsibilities of the Radiological Health and Safety Representative position which has been deleted.
- Addition of a new position, Plant Manager (formerly titled General Manager-Plant Hatch), who has overall and direct responsibility for activities strictly related to the safety and effectiveness of day to day operation of the plant, while support type functions are the responsibility of the new position, Plant Support Manager.
 - Addition of a new position, General Manager Quality Assurance (formerly titled General Manager Quality Assurance and Radiological Health and Safety) whose responsibilities for radiological health and safety have been transferred to the Manager Radiological Safety as described above.
 - The new position, Manager Nuclear Safety and Licensing (formerly titled Manager Nuclear Engineering and Chief Nuclear Engineer) and the new position Manager Nuclear Training report to the Senior Vice President Nuclear Operations which is a higher reporting level than the former position had. The following four new positions report to the Manager, Nuclear Safety and Licensing: Nuclear Licensing Manager, Hatch; Nuclear Licensing Manager, Vogtle; Nuclear Safety Manager; and Nuclear Safety Reviews Manager.
 - The Hatch Quality Assurance Site Manager's primary onsite line of communication is with the Vice President Plant Hatch (a higher reporting level) instead of with the General Manager-Plant Hatch (now Plant Manager).
 - Deletion of the following positions from the Engineering and Construction Services Department: Vice President and Chief Engineer Power Supply Engineering and Services, Manager Engineering Services, and Hatch Project Manager. The title, Hatch Project Manager has been changed to Manager Nuclear Projects and this position now reports to the Nuclear Operations Department.
 - Deletion of the position of Deputy General Manager-Plant Hatch. This position reported to the former General Manager-Plant Hatch. The responsibilities of the General Manager-Plant Hatch and the Deputy General Manager-Plant Hatch have been assumed by the new positions, Plant Manager and Plant Support Manager, who have replaced the General Manager-Plant Hatch position.
 - Deletion of the position of Manager Nuclear Generation who had corporate responsibility for fire protection. This position reported to the former Vice President and General Manager Nuclear Operations. The latter position

was replaced by the Vice President Plant Hatch and moved onsite. The responsibilities of the Manager Nuclear Generation including fire protection have been assumed by the new position, Manager Engineering Liaison. The designation for corporate responsibility for fire protection has been removed from the Offsite Organization Chart but continues to be addressed in the Fire Protection Program, as required by Appendix R of 10 CFR 50. The responsibility of the former Manager Nuclear Generation for receiving reports, written notifications, and minutes of the PRB; reportable occurrence notifications; and safety limit violation notifications is now assumed by the Vice President Plant Hatch.

- o Titles of positions mentioned above have been changed throughout Section 6.0 to reflect the proposed changes.

We find that the organization and staffing, as revised (1) should promote a higher level of management control, (2) should promote management responsibility for distinct functional areas, thereby increasing accountability and (3) meet the acceptance criteria of Sections 13.1.1 and 13.1.2 of the SRP and are, therefore, acceptable.

3.b Deletion of Organization Charts, Figures 6.2.1-1 and 6.2.2-1

The licensee's submittal of October 27, 1986 (Reference 4) proposed to delete Figures 6.2.1-1 and 6.2.2-1 from the Appendix A TS.

The staff has been examining the requirement for organization charts in the TS to see if an alternate means might be used to maintain an awareness of and appropriate controls over the organizations of licensees. This is being handled generically and, at such time as a decision is reached, any relief will be offered to all licensees. Pending a generic decision however, no action is being taken on this request.

4. Section 6.5, Review and Audit, Units 1 and 2 Appendix A TS

4.a Revisions to the Plant Review Board Composition and Quorum, Specifications 6.5.1.2 and 6.5.1.5

The proposed changes to the Plant Review Board (PRB) membership requirements would allow for supervisory level personnel from six onsite departments to serve as permanent members. The PRB, which had eight voting members would be revised to have six permanent members. The minimum quorum requirement for the PRB would be reduced from five to four members to coincide with the proposed reduction in the minimum membership requirement. These changes would also delete the requirement for the Plant Manager (formerly General Manager-Plant Hatch) and the Deputy General Manager-Plant Hatch (whose position, has been deleted) to serve as permanent members on the PRB. The PRB chairman and his designated alternate would be required to be a manager of one of the six onsite departments listed in the proposed technical specification or higher. Therefore, they will be of sufficiently high level of management to deal effectively with the other PRB members. The remainder of the members

will be at the level of supervisor or higher and, therefore, will be of sufficiently high level to deal with issues that the PRB has responsibility for.

The requested revision does not change the minimum qualification requirements for membership on the PRB (per Regulatory Guide 1.8). The proposed changes to the PRB would allow the Plant Manager to devote more time to line duties while maintaining adequate management representation on the PRB. The Plant Manager would, therefore, have more flexibility. In addition, plant management control of the PRB would be maintained since the Plant Manager will be responsible for PRB membership appointments.

The new minimum quorum requirement is acceptable since four members is a majority of the revised total PRB membership of six.

The proposed changes to the PRB are acceptable since they provide for interdisciplinary reviews of the subject matter, meet the relevant requirements of Regulatory Guide 1.33 and meet the acceptance criteria of Section 13.4 of the SRP.

4.b Plant Review Board Responsibilities, Specification 6.5.1.6

- In specification 6.5.1.6a, concerning PRB review of all procedures required by specification 6.8, the licensee added the words "except those for the Radiological Environmental Monitoring Program" which will be the responsibility of the Manager Radiological Safety.
- In specification 6.5.1.6e, concerning PRB responsibility for investigation of all violations of the TS, the licensee added the word "reportable" before the word "violations" and added the position of "Senior Vice President Nuclear Operations" to receive reports.
- Specification 6.5.1.6f has been changed as follows:

The words "24-hour" have been deleted and new words have been added so that it now reads "Review of events requiring written notification to the Commission per the Technical Specifications and/or reporting requirements of 10 CFR 50.73."
- In specification 6.5.1.6l, concerning PRB responsibility for review of changes to the Process Control Program and the Offsite Dose Calculation Manual (ODCM), the licensee added the words "except for the section on the Radiological Environmental Monitoring Program in the ODCM" which is now the responsibility of the Manager Radiological Safety.
- In specification 6.5.1.6m, the title "Manager, Nuclear Engineering and Chief Nuclear Engineer" has been deleted and replaced by the "Manager Radiological Safety."

The above changes to the Plant Review Board's responsibilities are acceptable, in that they are consistent with the responsibilities of the new position, Manager Radiological Safety, described in Section 2.2 above, and are consistent with the acceptance criteria of SRP Section 13.4.

4.c Plant Review Board Authority and Records, Specifications 6.5.1.7 and 6.5.1.8

These specifications have been changed to delete the title of Manager of Nuclear Generation and add the titles of Vice President Plant Hatch and Senior Vice President Nuclear Operations. This is acceptable in that it provides notification to higher level managers and is consistent with the changes described in Section 2.2 above.

4.d Revisions to the Safety Review Board (SRB), Specification 6.5.2

The proposed changes to the Safety Review Board (SRB) include:

- Reducing the SRB membership from seven persons (Specification 6.5.2.2) and quorum requirements from four persons (Specification 6.5.2.6) to the levels specified in Sections 4.3.2.1 and 4.3.2.3 of ANSI Standard N18.7-1976. The SRB will be composed of a minimum of five persons. The requested revision does not change the minimum qualification requirements for membership on the SRB.
- Shifting the corporate management responsibility for the SRB (Specification 6.5.2.3) from the Executive Vice President Power Supply position which has been deleted, to the new position Senior Vice President Nuclear Operations since the latter position now has full direct responsibility for matters pertaining to GPC operating nuclear facilities.
- Rewording of the lead statement for Specification 6.5.2.7 from "The SRB shall review:" to "The SRB shall be responsible for the review of:". This change would allow the SRB to delegate certain review tasks while retaining the overall responsibility for the performance of those reviews.
- Spelling out of the acronyms "ODCM" to Offsite Dose Calculation Manual and "PCP" to Process Control Program and replacing "biennially" with "at least once per 24 months."

The proposed changes to the SRB would improve the consistency between the Hatch Technical Specifications and the Vogtle Technical Specifications regarding the SRB organization and Functions. Consistency in the SRB organization and functions for both Plant Hatch and Plant Vogtle is needed, since a single SRB will function for both Hatch and Vogtle.

The above changes are acceptable since they conform to the new organization as discussed in Section 2.2, provide for independent review as described in Section 4.3 of ANS 3.2, ANSI N18.7-1976 and meet the acceptance criteria of SRP Section 13.4.

5. Section 6.6, Reportable Occurrence Action, Units 1 and 2 Appendix A TS

This specification concerning reportable occurrence action has been changed to be consistent with the change in specification 6.5.1.6f as described in Item 4.b above and is, therefore, acceptable. In addition, the Vice President Plant Hatch and the Senior Vice President Nuclear Operations have been added to receive notification. This provides for higher level management attention and is, therefore, acceptable.

6. Section 6.7, Safety Limit Violation, Units 1 and 2 Appendix A TS

In specifications 6.7.1b and 6.7.1d the "Senior Vice President Nuclear Operations" has been added to receive reports of safety limit violations. This is acceptable in that such violations will receive higher level management attention.

7. Section 6.8, Procedures, Units 1 and 2 Appendix A TS

- In specification 6.8.2 and 6.8.3 concerning approval of procedures, the title General Manager-Plant Hatch has been replaced with the Plant Manager or Plant Support Manager. However, for approval of the Emergency Plan implementing procedures (formerly the responsibility of the General Manager-Plant Hatch), the Manager Plant Training and Onsite Emergency Preparedness has been designated. The Manager Plant Training and Onsite Emergency Preparedness reports directly to the Vice President Plant Hatch. This change should provide greater attention to emergency preparedness and does not diminish management responsibility for this area. We find this change acceptable in that it is consistent with the changes in organization and reporting requirements described in Item 3.a above.
- Specification 6.8.4, concerning review and approval of changes to the Radiological Environmental Monitoring Program procedures, has been added to be consistent with the change in specification 6.1.2 concerning responsibilities of the Manager Radiological Safety as described in Item 3.a above and is, therefore, acceptable.

8. Section 5, Administrative Controls, Units 1 and 2 Appendix B TS

- Specifications 5.0, 5.1, 5.3 and 5.5 would be changed to be consistent with the revised organization and titles as described and evaluated in items 2 and 3 above. These changes provide for internal consistency in the TS and are, therefore, acceptable.
- Specification 5.2 would be revised to eliminate the reference to Figure 5.2-1 and Figure 5.2-1 would be deleted. As discussed in item 3.b above, this matter of organization charts in the TS is under generic review by the staff. Deletion of the charts at this time is premature. Therefore, no action is being taken on this request.
- Specifications 5.5 and 5.6 would be revised to eliminate the references to the Environmental Program Description Document (EPDD). The environmental program described by this document is now complete, and continued reference to the document in the TS is neither necessary nor desirable. These changes are, therefore, acceptable. Elimination of the references to the EPDD results in renumbering subsections of Specifications 5.5, 5.6 and 5.7. These changes are consistent with the elimination of the references to the EPDD and are acceptable.

ENVIRONMENTAL CONSIDERATIONS

These amendments relate to changes in administrative procedures or requirements.

Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

CONCLUSION

The Commission made proposed determinations that the amendments involve no significant hazards consideration which were published in the Federal Register (51 FR 16928) on May 7, 1986, (51 FR 24255) on July 2, 1986, (51 FR 41854) on October 19, 1986, and (51 FR 45200) on December 17, 1986, and consulted with the state of Georgia. No public comments were received, and the state of Georgia did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

REFERENCES

1. Letter from J. T. Beckham, GPC, to D. Muller, NRC, dated February 17, 1986.
2. Letter from J. P. O'Reilly, GPC, to D. Muller, NRC, dated May 16, 1986.
3. Letter from J. T. Beckham, GPC, to D. Muller, NRC, dated August 27, 1986.
4. Letter from J. P. O'Reilly, GPC, to D. Muller, NRC, dated October 27, 1986.
5. Letter from J. P. O'Reilly, GPC, to G. Rivenbark, NRC, dated November 24, 1986.

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