Docket Nos.: 50-321 AUG 1 9 1987 and 50-366

Mr. James P. O'Reilly Senior Vice President - Nuclear Operations Georgia Power Company P. O. Box 4545 Atlanta, Georgia 30302 DISTRIBUTION: Docket File NRC PDR Local PDR PRC System MDuncan LCrocker OGC-Bethesda ACRS (10) JPartlow EJordan SVarga/GLainas

Dear Mr. O'Reilly:

Subject: License Amendment Corrections

Reference: Letter, L. P. Crocker (NRC) to J. P. O'Reilly (GPC), "Issuance of Amendment Nos. 143 and 78 to Facility Operating Licenses DPR-57 NPF-5 - Edwin I. Hatch Nuclear Plant, Units 1 and 2 (TACS 59542/ 56049)," July 14, 1987.

The following Hatch Units 1 and 2 Technical Specification pages (previously issued by the reference letter) were modified to include previously issued amendments.

Unit 1 (Amendment 143)

3.2-3 Incorporated Amendments 121 and 125 3.2-25 Incorporated Amendment 121 3.7-16 Incorporated Amendment 129

Unit 2 (Amendment 78)

3/43-11Incorporated Amendment713/43-15Incorporated Amendment713/43-16Incorporated Amendment67

Please replace the Technical Specification pages sent with amendments 143 and 78 with the enclosed revised pages.

These changes are effective as of the date of this letter and shall be implemented within 60 days.

Sincerely,

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Lawrence P. Crocker, Project Manager Project Directorate II-3 Division of Reactor Projects, I/II

Enclosure: As stated

cc: See next page PD#II-2/DRP-I/II PD#II-3/DRP-I/II MDuncan/mac LCrocker 08////87 08//9/87

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Mr. James P. O'Reilly Georgia Power Company Edwin I. Hatch Nuclear Plant, Units Nos. 1 and 2

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Mr. J. Leonard Ledbetter, Commissioner Department of Natural Resources 270 Washington Street, N.W. Atlanta, Georgia 30334

Chairman Appling County Commissioners County Courthouse Baxley, Georgia 31513 Table 3.2-1 (Cont.)

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ATCH - U	Ref. No. (a)	Instrument	Trip Condition Nomenclature	Required Operable Channels per Trip System (b)	Trip Setting	Action to be taken if number of channels is not met for both trip systems (c)	Remarks (d)
IN							
	4	Main Steam Line Radiation	High	2	≤3 times normal full power back- ground ^(*)	Initiate an orderly load reduction and close MSIVs within 8 hours.	Initiates Group 1 isolation.
	5	Main Steam Line Pressure	Low	2	≥825 psig	Initiate an orderly load reduction and close MSIVs within 8 hours.	Initiates Group 1 isolation. Only required in RUN mode, therefore activated when Mode Switch is i RUN position.
	6	Main Steam Line Flow	High	2	≤138% rated flow (≤115 psid)	Initiate an orderly load reduction and close MSIVs within 8 hours.	Initiates Group 1 isolation.
3.2-3	7	Main Steam Line Tunnel Temperature	High	2	≤194°F	Initiate an orderly load reduction and close MSIVs within 8 hours.	Initiates Group 1 isolation.
	8	Reactor Water Cleanup System Differential Flow	High	1	20-80 gpm	lsolate reactor water cleanup system.	Final trip setting will be determined during startup test program.
	9	Reactor Water Cleanup Area Temperature	High	2	≤124°F	isolate reactor water cleanup system.	
	10	Roactor Water Cleanup Area Ventilation Differential Temperature	High	2	≤67°F	lsolate reactor water cleanup systom,	(
Amend	11	Condenser Vacuum	Low	2	≥7" Hg. vacuum	Initiate an orderly load reduction and close MSIVs Within 8 hrs.	Initiate Group 1 isolation
ment No. 14	12	Drywell Radiation	High	1	≤138 R/HR.	Close the affected isolation valves within 24 hours or be in Hot Shutdown within the next 6 hours and in Cold Shutdown within the next 30 hours,	, Isolates containment purge and vent valves.

ndment No. 143

			Table 4.		
HATCH	Ref. No. <u>(a)</u>	Instrument	Instrument Check Minimum Frequency	Instrument Check Instrument Functional Test Minimum Frequency Minimum Frequency	
I UN	10	Reactor Water Cleanup Area Ventilation Differential Temperature	Once/shift	Once/month	(c) Once/operating cycle
	11 12	Condenser Vacuum Drywell Radiation	None Once/dav	(d) Once/month (변)	Every 3 months
					Once/operating cycle

Notes for Table 4.2-1

a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in Table 4.2-1 and items in Table 3.2-1.

b. Instrument functional tests are not required when the instruments are not required to be operable or are tripped. However, if functional tests are missed, they shall be performed prior to returning the instrument to an operable status.

- c. Calibrations are not required when the instruments are not required to be operable. However, if calibrations are missed, they shall be performed prior to returning the instrument to an operable status.
- d. Initially once per month or according to Figure 4.1-1 with an interval of not less than one month nor more than three months. The compilation of instrument failure rate date may include data obtained

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3.2-25

TABLE 3.7-1

PRIMARY CONTAINMENT ISOLATION VALVES WHICH RECEIVE A PRIMARY CONTAINMENT ISOLATION SIGNAL

Group (b)	Valve Identification (d)	Number Operate Inside	of Power <u>d Valves</u> Outside	Maximum Operating Time (sec)	Normal Position (a)	Action on Initiating Signal (a)
1	Main steam line (B21-F022 A,B,C,D; B21-F028 A,B,C,D)	4	4	3 <t<5< td=""><td>0</td><td>GC</td></t<5<>	0	GC
1	Main steam line drain (B21-F016, B21-F019)	1	1	15	С	sc
1	Reactor water sample line (B31-F019, B31-F020)	1	1	5	0	GC
2(f)	Drywell purge inlet (T48-F307, T48-F308)		2	5	с	SC
2(f)	Drywell main exhaust (T48-F319, T48-F320)		2	5	С	sc
2	Drywell exhaust valve bypass to standby gas treatment (T48-F341, T48-F340)		2	5	С	SC
2	Drywell nitrogen make-up line (normal operation) (T48-F118A)		1	5	c	sc
2(*)	Suppression chamber purge inlet (T48-F309, T48-F324)		2	5	с	SC
2(*)	Suppression chamber main exhaust (T48-F318, T48-F326)		2	5	С	sc

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3.7-16

TABLE 3.3.2-1

ISOLATION ACTUATION INSTRUMENTATION

TRIP_FUNCTION	VALVE GROUPS OPERATED BY SIGNAL(a)	MINIMUM NUMBER OPERABLE CHANNELS <u>PER TRIP SYSTEM(b)(c)</u>	APPLICABLE OPERATIONAL CONDITION	ACTION
 PRIMARY CONTAINMENT ISOLATION Reactor Vessel Water Level Low (Level 3) (2821-N680 A, B, C, D) Low-Low (Level 2) (2821-N682 A, B, C, D) Low-Low-Low (Level 1) (2821-N681 A, B, C, D) 	2, 6, 10, 11, 12 5, * 1	2 2 2	1, 2, 3 1, 2, 3 1, 2, 3	20 20 20
b. Drywell Pressure - High (2C71-N650 A, B, C, D)	2, 6, 7, 10, 12, *	2	1, 2, 3	20
<pre>c. Main Steam Line 1. Radiation - High (2D11-K603 A, B, C, D) 2. Pressure - Low (2B21-N015 A, B, C, D) 3. Flow - High (2B21-N686 A, B, C, D) (2B21-N688 A, B, C, D) (2B21-N688 A, B, C, D) (2B21-N689 A, B, C, D)</pre>	1, 12, (d) 1 1	2 2 2/line	1, 2, 3 1 1, 2, 3	21 22 21
d. Main Steam Line Tunnel Temperature - High (2B21-N623 A, B, C, D) (2B21-N624 A, B, C, D) (2B21-N625 A, B, C, D) (2B21-N626 A, B, C, D)	1	2/line ^(e)	1, 2, 3	21
e. Condenser Vacuum - Low (2B21-N056 A, B, C, D)	1	2	1, 2, (f), 3(f)	23
f. Turbine Building Area Temperature - High (2U61-R001, 2U61-R002, 2U61-R003, 2U61-R004)	1	2 ^(e)	• 1, 2, 3	21
g. Drywell Radiation - High (2D11-K621 A,B)	(1)	1	1, 2, 3	29

HATCH - UNIT 2

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Amendment No. 78

TABLE 3.3.2-1 (Continued)

✓ ISOLATION ACTUATION INSTRUMENTATION

ACTION

- ACTION 20 Be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.
- ACTION 21 Be in at least STARTUP with the main steam line isolation valves closed within 2 hours or be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.
- ACTION 22 Be in at least STARTUP within 2 hours.
- ACTION 23 Be in at least STARTUP with the Group 1 isolation valves closed within 2 hours or in at least HOT SHUTDOWN within 6 hours.
- ACTION 24 Establish SECONDARY CONTAINMENT INTEGRITY with the standby gas treatment system operating within one hour.
- ACTION 25 Isolate the reactor water cleanup system.
- ACTION 26 Close the affected system isolation valves and declare the affected system inoperable.
- ACTION 27 Verify power availability to the bus at least once per 12 hours or close the affected system isolation valves and declare the affected system inoperable.
- ACTION 28 Close the shutdown cooling supply and reactor vessel head spray isolation values unless reactor steam dome pressure \leq 145 psig.

ACTION 29 - Either close the affected isolation valves within 24 hours or be in HOT SHUTDOWN within the next 6 hours and in COLD SHUTDOWN within the next 30 hours.

NOTES

- * Actuates the standby gas treatment system.
- ** When handling irradiated fuel in the secondary containment.
- a. See Specification 3.6.3, Table 3.6.3-1 for valves in each valve group.
- b. A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.
- c. With a design providing only one channel per trip system, an inoperable channel need not be placed in the tripped condition where this would cause the Trip Function to occur. In these cases, the inoperable channel shall be restored to OPERABLE status within 2 hours or the ACTION required by Table 3.3.2-1 for that Trip Function shall be taken.
- d. Trips the mechanical vacuum pumps.
- e. A channel is OPERABLE if 2 of 4 instruments in that channel are OPERABLE.
- f. May be bypassed with all turbine stop valves closed.
- g. Closes only RWCU outlet isolation valve 2G31-F004.
- h. Alarm only.
- 1. Adjustable up to 60 minutes.
- j. Isolates containment purge and vent valves.

HATCH - UNIT 2

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	ISOLATION ACTUATION INSTRUMENTATION SETPOINTS				
TRIP	UNCTION RIMARY CONTAINMENT ISOLATION	TRIP SETPOINT	ALLOWABLE VALUE		
1	. Reactor Vessel Water Level 1. Low (Level 3) 2. Low Low (Level 2) 3. Low Low Low (Level 1)	≥ 10 inches* ≥ -47 inches* ≥ -113 inches*	≥ 10 inches# ≥ -47 inches# ≥ -113 inches#		
t	. Drywell Pressure - High	≤ 1.92 psig	≤ 1.92 psig		
C	. Main Steam Line 1. Radiation - High	≤ 3 x full power background	≤ 3 x full power background		
	2. Pressure - Low 3. Flow - High	≥ 825 psig ≤ 138% rated flow	≥ 825 psig ≤ 138% rated flow		
C	. Main Steam Line Tunnel Temperature - High	≤ 194°F	≤ 194°F		
e	. Condenser Vacuum - Low	≥ 7" Hg vacuum	≥ 7" Hg vacuum		
f	. Turbine Building Area TempHigh	≤ 200°F	≤ 200°F		
9	. Drywell Radiation - High	≤ 138 R/hr	≤ 138 R/hr		
2. <u>s</u>	ECONDARY CONTAINMENT ISOLATION				
a	. Reactor Building Exhaust Radiation - High	≤ 60 mr/hr	≲ 60 mr/hr		
b	. Drywell Pressure - High	≤ 1.92 psig	≤ 1.92 psig		
c	. Reactor Vessel Water Level - Low Low (Level 2)	≥ -47 inches*	≥ -47 inches#		
d	. Refueling Floor Exhaust Radiation - High	≤ 20 mr/hr	≤ 20 mr/hr		

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TABLE 3.3.2-2

*See Bases Figure B 3/4 3-1.

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