

March 31, 1987

Docket No. 50-321

Mr. James P. O'Reilly
Senior Vice President - Nuclear Operations
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

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Dear Mr. O'Reilly:

The Commission has issued the enclosed Amendment No. 135 to Facility Operating License No. DPR-57, for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated September 9, 1986.

The amendment modifies the description of the refueling interlock surveillance requirements to clarify them.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

George W. Rivenbark, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:

1. Amendment No. 135 to DPR-57
2. Safety Evaluation

cc w/enclosures:
See next page

DBL:PD#2
SNorris
3/6/87


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Mr. J. P. O'Reilly
Georgia Power Company

Edwin I. Hatch Nuclear Plant,
Units Nos. 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-321
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 135
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated September 9, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 135, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 31, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 135

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of the Appendix A Technical Specifications with enclosed page. The revised areas are indicated by marginal lines.

Pages

3.10-1

3.10. REFUELINGApplicability

The Limiting Conditions for Operation apply to the fuel handling and associated core reactivity limitations.

Objective

The objective of the Limiting Conditions for Operation is to assure that core reactivity is within the capability of the control rods and to prevent criticality during refueling.

SpecificationsA. Refueling Interlocks1. Reactor Mode Switch

The Mode Switch shall be locked in the REFUEL position during core alterations and the refueling interlocks shall be operable except as stated in Specification 3.10.E.

2. Fuel Grapple Hoist Load Setting Interlock

The fuel grapple hoist load setting interlock switch shall be set at 485 ± 30 lbs.

3. Auxiliary Hoists Load Setting Interlock

If the frame-mounted auxiliary hoist, the monorail-mounted auxiliary hoist, or the service platform hoist is to be used for handling fuel with the head off the reactor vessel, the load setting interlock on the hoist to be used shall be set at 485 ± 30 lbs.

B. Fuel Loading

Fuel shall not be loaded into the reactor core unless all control rods are fully inserted.

4.10. REFUELINGApplicability

The Surveillance Requirements apply to the periodic testing of those interlocks and instrumentation used during refueling and core alterations.

Objective

The objective of the Surveillance Requirements is to verify the operability of instrumentation and interlocks used in refueling and core alterations.

SpecificationsA. Refueling Interlocks1. Reactor Mode Switch

Prior to any core alterations, the reactor mode switch refueling interlocks shall be functionally tested. They shall be tested at weekly intervals thereafter until no longer required. They shall also be tested following any repair work associated with the interlocks.

2. Fuel Grapple Hoist Load Setting Interlock

Prior to any movement of fuel assemblies or control rods within the reactor pressure vessel, this refueling interlock shall be functionally tested. It shall be tested at weekly intervals thereafter until no longer required and following any repair work associated with this interlock.

3. Auxiliary Hoists Load Setting Interlock

Prior to any movement of fuel assemblies or control rods within the reactor pressure vessel, this refueling interlock shall be functionally tested. It shall be tested at weekly intervals thereafter until no longer required and following any repair work associated with this interlock.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 135 TO

FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-321

1.0 INTRODUCTION

Hatch Nuclear Plant Unit 1 was licensed in 1974 and the plant Technical Specifications are in the old format. The Unit 2 Technical Specifications are in the new format since the plant was licensed in 1978. Due to the differences in the format, as well as in the content of the specifications, there are some differences between the Units 1 and 2 Technical Specifications. In a submittal dated September 9, 1986, the Georgia Power Company (the licensee) has requested changes to the Unit 1 Technical Specifications in the area of the refueling interlocks surveillance requirements. The proposed change would clarify the Unit 1 Technical Specifications and make them conform more closely to the Unit 2 Technical Specifications. The specification being changed, Technical Specification 3/4.10.A, is related to the reactor mode switch, the fuel grapple hoist load setting interlock, and the auxiliary hoists load setting interlock. This evaluation is concerned with the surveillance requirements associated with the aforementioned components.

2.0 EVALUATION

The current Technical Specification 3.10.A (limiting conditions for operation) has three subsections which deal with the reactor mode switch (3.10.A.1), the fuel grapple hoist load setting interlocks (3.10.A.2), and the auxiliary hoists load setting interlocks (3.10.A.3). The corresponding surveillance requirements are specified on a single generic section (4.10.A) which deals with refueling interlocks. This surveillance requirements specification states that "prior to any fuel handling with the head off the reactor vessel, the refueling interlocks shall be functionally tested. They shall be tested at weekly intervals thereafter until no longer required. They shall also be tested following any repair work associated with the interlocks."

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The licensee has proposed to revise the surveillance requirements, such that there is a specific surveillance requirement section to correspond to the section on limiting conditions for operation. The proposed revision would have Sections 4.10.A.1, 4.10.A.2, and 4.10.A.3 correspond to Sections 3.10.A.1, 3.10.A.2, and 3.10.A.3, respectively. The original surveillance Section 4.10.A has been proposed as Section 4.10.A.1. With respect to surveillance of the reactor mode switch, the first line of the paragraph has been revised to read: "Prior to any core alterations, the reactor mode switch refueling interlocks..." (The revised words are underlined). The Licensee has not proposed any revision to the corresponding limiting condition for operation. Since the revision for Section 4.10.A.1 clarifies and makes the surveillance requirements more specific, the proposed revision of this section is acceptable.

The licensee has proposed two new sections in the surveillance requirements, namely Sections 4.10.A.2 and 4.10.A.3, relating to the fuel grapple and the auxiliary hoists load setting interlocks, respectively. Both new proposed paragraphs state that "prior to any movement of fuel assemblies or control rods within the reactor pressure vessel, this refueling interlock shall be functionally tested. It shall be tested at weekly intervals thereafter until no longer required and following any repair work associated with this interlock." This proposed modification of the plant Technical Specifications places the surveillance requirements in a one to one relationship with the appropriate limiting conditions for operation. Furthermore, there is no proposed deviation from the original intent of the refueling interlock surveillance requirements. On the basis of the above, we conclude that the addition of these two new sections of surveillance requirements is acceptable.

The licensee has proposed modifications to the Unit 1 Technical Specifications which clarify the surveillance requirements for the refueling interlocks and make the Technical Specifications similar to the Unit 2 Technical Specifications. On the basis that the proposed Technical Specification clarifies the surveillance requirements and, thereby, tends to reduce the potential for operator error, we conclude that the proposed modifications to the Unit 1 Technical Specifications 3/4.10.A are acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment changes surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Ridgely

Dated: March 31, 1987