

Docket No.: 50-321

June 5, 1987

Mr. James P. O'Reilly  
Senior Vice President - Nuclear Operations  
Georgia Power Company  
P. O. Box 4545  
Atlanta, Georgia 30302

Dear Mr. O'Reilly:

Subject: Issuance of Amendment No.140 to Facility Operating License DPR-57  
- Edwin I. Hatch Nuclear Plant, Unit 1 (TAC 64785)

The Commission has issued the enclosed Amendment No. 140 to Facility Operating License DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit 1. The amendment consists of changes to the Technical Specifications in response to your application dated March 4, 1987 and supplemented by your letter of April 21, 1987.

The amendment modifies the Technical Specifications to delete certain valves listed in Table 3.7-4 as containment isolation valves requiring leak test, correct information errors in Table 3.7-4, and change the pressure at which main steam isolation valves are required to be tested.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

151

Lawrence P. Crocker, Project Manager  
Project Directorate II-3  
Division of Reactor Projects-I/II

Enclosures:

1. Amendment No. 140 to DPR-57
2. Safety Evaluation

cc w/enclosures:

See next page

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*BJ*  
PD#II-3/DRP-I/II  
BJYoungblood  
05/28/87

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P PDR

Mr. J. P. O'Reilly  
Georgia Power Company

cc:

Edwin I. Hatch Nuclear Plant,  
Units Nos. 1 and 2  
Docket Nos.: 50-366/321

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U.S. Nuclear Regulatory Commission,  
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Mr. J. Leonard Ledbetter, Commissioner  
Department of Natural Resources  
270 Washington Street, N.W.  
Atlanta, Georgia 30334

Chairman  
Appling County Commissioners  
County Courthouse  
Baxley, Georgia 31513

DATED June 5, 1987

AMENDMENT NO. 140 TO FACILITY OPERATING LICENSE DPR-57, EDWIN I. HATCH, UNIT 1

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA  
DOCKET NO. 50-321  
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 140  
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated March 4, 1987, and supplemented April 21, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 140, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.


FOR THE NUCLEAR REGULATORY COMMISSION


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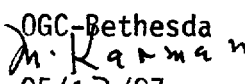

B. J. Youngblood, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II


Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 5, 1987

  
PD#II-3/DRP-I/II  
LCrocker/mac  
05/11/87

  
PD#II-3/DRP-I/II  
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BJYoungblood  
05/28/87

ATTACHMENT TO LICENSE AMENDMENT NO. 140

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove</u> <u>Page</u>	<u>Insert</u> <u>Page</u>
3.7-6a	3.7-6a
3.7-24	3.7-24
3.7-25	3.7-25
3.7-26	3.7-26
3.7-26a	3.7-26a
3.7-26b	3.7-26b

g. Acceptance Criteria for Type B and Type C Tests

The combined leakage rate of components subject to Type B and C tests shall be determined under the program established in Appendix J of 10 CFR Part 50 and shall not exceed  $0.6 L_a$ .

h. Main Steam Line Isolation Valves

The main steam line isolation valves shall be tested at a pressure of 28 psig for leakage at least once per operating cycle. If a total leak rate of 11.5 scf per hour for any one main steam line isolation valve is exceeded, repairs and retest shall be performed to correct this condition.

Table 3.7-4  
(Continued)

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>
X-18	G11-F019 & F020	(1) (2) (4) (5) (9)
X-19	G11-F003 & F004	(1) (2) (4) (5) (9)
X-20	P41-F049	(1) (2) (4) (5) (9)
X-21	P51-F513 & F514	(1) (2) (4) (5) (9)
X-22	P70-F004, F005	(1) (2) (4) (5) (10)
X-23	P42-F051	(1) (2) (4) (5) (10)
X-24	P42-F052	(1) (2) (4) (5) (10)
X-25	T48-F307, F308, F309, F103 & F324	(1) (2) (4) (5) (9)
X-25	T48-F113	(1) (2) (4) (5) (14)
X-25	T48-F321	(1) (2) (4) (5) (14)
X-25	T48-F104	(1) (2) (4) (5) (14)
X-25	T48-F114, F118A & F322	(1) (2) (4) (5) (10)
X-26	T48-F319 & F320	(1) (2) (4) (5) (9)
X-26	T48-F334A	(1) (2) (4) (5) (14)
X-26	T48-F334B	(1) (2) (4) (5) (14)
X-26	T48-F335A & F335B	(1) (2) (4) (5) (10)
X-26	T48-F340 & F341	(1) (2) (4) (5) (9)
X-26	P33-F002	(1) (2) (4) (5) (10)
X-26	P33-F010	(1) (2) (4) (5) (14)
X-27A	D11-F051 & F053	(1) (2) (4) (5) (10)
X-27F	P70-F066 & F067	(1) (2) (4) (5) (10)



Table 3.7-4  
(Continued)

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>
X-28F	P33-F003 & F011	(1) (2) (4) (5) (9)
X-28A	B31-F019 & F020	(1) (2) (4) (5) (9)
X-31D	P33-F004 & F012	(1) (2) (4) (5) (9)
X-31F	B31-F013A & F017A	(1) (2) (4) (5) (10)
X-33C	D11-F050 & F052	(1) (2) (4) (5) (10)
X-34E	P33-F005 & F013	(1) (2) (4) (5) (9)
X-35A	C51-Ball Valve	(1) (2) (4) (5) (10)
X-35B	C51-Ball Valve	(1) (2) (4) (5) (10)
X-35C	C51-Ball Valve	(1) (2) (4) (5) (10)
X-35D	C51-Ball Valve	(1) (2) (4) (5) (10)
X-35E	C51-Nitrogen Inerting Check Valve	(1) (2) (4) (5) (10)
X-39A	E11-F016A	(1) (2) (4) (5) (9)
X-39B	E11-F016B	(1) (2) (4) (5) (9)
X-40C	P70-F002 & F003	(1) (2) (4) (5) (9)
X-42	C41-F006	(1) (2) (4) (5) (10)
X-42	C41-F007	(1) (2) (4) (5) (10)
X-44	P41-F050	(1) (2) (4) (5) (9)
X-45F	T23-F004 & F005	(1) (2) (4) (5) (9)
X-46	P21-F353 & F420	(1) (2) (4) (5) (9)
X-52B	B21-F111 & F112	(1) (2) (4) (5) (10) (11)
X-59A	B31-F013B & F017B	(1) (2) (4) (5) (10)

Table 3.7-4  
(Continued)

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>
X-205	T48-F310 & F328A	(1) (2) (4) (5) (9)
X-205	T48-F115 & F116	(1) (2) (4) (5) (9)
X-205	T48-F311 & F328B	(1) (2) (4) (5) (9)
X-205	T48-F118B	(1) (2) (4) (5) (10)
X-205	T48-F325 & F327	(1) (2) (4) (5) (9)
X-250	T48-F104	(1) (2) (4) (5) (14)
X-206A	E41-F121 & F122	(1) (2) (4) (5) (9)
X-211A	E11-F027A & F028A	(1) (2) (4) (5) (9)

Table 3.7-4  
(Continued)

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>
X-211B	E11-F027B & E11-F028B	(1) (2) (4) (5) (9)
X-212	E51-F001 & F040	(1) (2) (4) (5) (9)
X-213	E51-F002 & F028	(1) (2) (4) (5) (9)
X-214	E41-F021 & F049	(1) (2) (4) (5) (9)
X-215	E41-F022 & F040	(1) (2) (4) (5) (9)
X-217	P33-F007 & F015	(1) (2) (4) (5) (9)

Table 3.7-4  
(Continued)

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>
X-220	P33-F006	(1) (2) (4) (5) (14)
X-220	P33-F014	(1) (2) (4) (5) (10)
X-220	T48-F318 & F326	(1) (2) (4) (5) (9)
X-220	T48-F332A	(1) (2) (4) (5) (14)
X-220	T48-F332B	(1) (2) (4) (5) (14)
X-220	T48-F333A & F333B	(1) (2) (4) (5) (10)
X-220	T48-F338 & F339	(1) (2) (4) (5) (9)
X-221C	E51-F104 & F105	(1) (2) (4) (5) (9)
X-222A	E41-F111 & F104	(1) (2) (4) (5) (9)
X-223A	T48-Air Cylinder	(1) (2) (4) (5) (11)
X-223A	T48-F342G-L	(1) (2) (4) (5) (11)
X-223B	T48-Air Cylinder	(1) (2) (4) (5) (11)
X-223B	T489-F342A-F	(1) (2) (4) (5) (11)



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 140TO

FACILITY OPERATING LICENSE DPR-57

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-321

INTRODUCTION

By letter dated March 4, 1987, (Reference 1), and supplemented by letter dated April 21, 1987 (Reference 2), the licensee requested a revision to portions of the Hatch Unit 1, Technical Specifications relating to primary containment isolation valves. Principally, the licensee's proposal utilizes the findings and recommendations in a staff SER dated October 30, 1986 (Reference 3). This evaluation assesses the Hatch, Unit 1, updated containment leak rate test program and its conformance to the staff SER of October 30, 1986, and to 10 CFR 50, Appendix J.

As discussed in the SER dated October 30, 1986, the staff found the licensee's updated containment leak rate test program and its associated modifications to the Hatch, Unit 1, Technical Specifications to be acceptable, except for the proposal to delete several Technical Specification tables which listed the containment isolation components. The staff also found acceptable the proposed exemption request for Appendix J regarding the MSIV testing pressure.

EVALUATION

The staff has reviewed the licensee's submittal dated March 4, 1987. The majority of the proposed changes involve modifications to Technical Specifications (TS) Table 3.7-4, "Primary Containment Testable Isolation Valves." In its submittal, the licensee organized the requested changes into five separate items.

Regarding proposed change Items 1 and 2, the licensee requested deletion of selected containment isolation valves from TS Table 3.7-4. This table specifies valves that are required to be tested pursuant to Appendix J requirements. The licensee cites the staff's SER dated October 30, 1986 as a basis for the proposed deletions. As discussed in the staff's SER, piping systems that penetrate the torus and terminate below the water line of the torus do not have to satisfy Appendix J requirements since a supply of water in the torus is assured during post accident conditions. Consequently, the licensee proposed to delete the specified isolation valves (as listed in its

March 4, 1987 submittal) from TS Table 3.7-4. Moreover, the licensee indicated that these selected valves will be tested in accordance with ASME code, Section XI, requirements. Thus, the staff finds the proposed changes are appropriate and consistent with our SER.

The licensee's letter of April 21, 1987 (Reference 2) withdrew the request for the changes originally proposed as change 3 in the letter of March 4, 1987 (Reference 1). Therefore, no change to the Technical Specifications is required for this item.

Regarding proposed changes for item 4, the licensee requested a number of changes to Table 3.7-4. These changes are editorial in nature and would correct several inaccuracies. Also, these changes do not alter testing requirements. The staff finds the proposed changes do clarify the TS and, therefore, are acceptable.

Lastly, regarding item 5, the licensee requested that a value of 28 psig pressure be inserted into TS Section 4.7.A.2.h, replacing the 1/2 Pa term. This TS section specifies the test pressure and the leakage criteria for leak testing the MSIVs. The licensee has indicated that 1/2 Pa is equal to 29.5 psig. As discussed in the SER dated October 30, 1986, the staff provided an evaluation allowing the MSIVs to be tested at a reduced pressure (an exemption to 10 CFR 50, Appendix J) of 28 psig, but erroneously indicated the value is 1/2 Pa. But the evaluation was based on 28 psig. In addition, the licensee indicates that Hatch Unit 1 was originally licensed with a requirement that the MSIVs be tested at a test pressure of 28 psig. Moreover, the licensee notes that the inboard MSIV would begin to lift at a pressure of about 28.5 psig, resulting in a meaningless test.

Even though 28 psig is not equal to 1/2 Pa, the technical assessment as documented by the staff remains unchanged. Therefore, the staff concurs with the licensee's proposal to appropriately insert 28 psig in TS Section 4.7.A.2.h.

#### ENVIRONMENTAL CONSIDERATIONS

Pursuant to 10 CFR 51.32, the Commission has determined that the issuance of this amendment will have no significant impact on the environment (52 FR 21398 ).

#### CONCLUSION

Notice of opportunity for a prior hearing was published in the Federal Register on March 27, 1987 (52 FR 9980). No requests for a hearing were received.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

REFERENCES

1. Letter from J. P. O'Reilly, Georgia Power Company, to U. S. Nuclear Regulatory Commission, dated March 4, 1987.
2. Letter from L. T. Gucwa, Georgia Power Company, to U. S. Nuclear Regulatory Commission, dated April 21, 1987.
3. Letter from G. W. Rivenbark, NRC, to J. T. Beckham, Jr., Georgia Power Company, dated October 30, 1986.

Principal Contributors: A. Notafrancesco  
L. Crocker

Dated: June 5, 1987

U. S. NUCLEAR REGULATORY COMMISSIONGEORGIA POWER COMPANY, ET. AL.DOCKET NO. 50-321NOTICE OF ISSUANCE OF AMENDMENT TO  
FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 140 to Facility Operating License No. DPR-57, issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensee), which revised the Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit 1, (the facility) located in Appling County, Georgia. This amendment was effective as of the date of its issuance.

The amendment modified the Technical Specifications to delete certain valves listed in Table 3.7-4 as containment isolation valves requiring leak test, correct information errors in Table 3.7-4, and change the pressure at which main steam isolation valves are required to be tested.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Prior Hearing in connection with this action was published in the FEDERAL REGISTER on March 27, 1987 (52 FR 9980). No request for a hearing or petition for leave to intervene was filed following this notice.

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The Commission has prepared an Environmental Assessment and Finding of No Significant Impact related to the action and has concluded that an environmental impact statement is not warranted because there will be no environmental impact attributable to the action beyond that which has been predicted and described in the Commission's Final Environmental Statement for the facility dated October 1972.

For further details with respect to the action see (1) the application for amendment dated March 4, 1987, as supplemented April 21, 1987, (2) Amendment No. 140 to License No. DPR-57, (3) the Commission's related Safety Evaluation, and (4) the Commission's related Environmental Assessment and Finding of No Significant Impact. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513. A copy of items (2), (3), and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C., 20555, Attention: Director, Division of Reactor Projects-I/II.

Dated at Bethesda, Maryland this 5th day of June 1987.

FOR THE NUCLEAR REGULATORY COMMISSION

151  
Lawrence P. Crocker, Project Manager  
Project Directorate II-3  
Division of Reactor Projects-I/II

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PD#II-3/DRP-I/II  
LCrocker/mac  
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PD#II-3/DRP-I/II  
BJYoungblood  
05/01/87

U. S. NUCLEAR REGULATORY COMMISSION  
GEORGIA POWER COMPANY, ET. AL.  
DOCKET NO. 50-321  
ENVIRONMENTAL ASSESSMENT AND FINDING OF  
NO SIGNIFICANT IMPACT

The U. S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to the Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) for the Edwin I. Hatch Nuclear Plant, Unit 1, located in Appling County, Georgia.

Environmental Assessment

Identification of Proposed Action: The amendment would revise the Technical Specifications for Hatch, Unit 1, to: (1) delete certain valves from the Table 3.7-4 that lists containment isolation valves subject to leak rate testing pursuant to the requirements of Appendix J to 10 CFR Part 50 (i.e., remove the Technical Specification requirements for leak testing these valves), (2) correct erroneous information in Table 3.7-4, and (3) change the pressure at which the main steam isolation valves are required to be leak tested.

These revisions to the Technical Specifications would be made in response to the licensee's application for amendment dated March 4, 1987, as revised April 21, 1987.

The Need for the Proposed Action: Paragraph III.C.2 of Appendix J to 10 CFR 50 requires that containment isolation valves be pressurized with air or nitrogen for leak testing purposes unless such valves are sealed with fluid from a seal system. Table 3.7-4 of the Technical Specifications for Hatch Unit 1 lists the valves that must be pressure tested for leakage. The valves

that would be deleted from the table by this amendment are valves that would remain covered by water following a LOCA. These valves, thus, are sealed by a fluid in a seal system and are not required to be pressure tested with air or nitrogen and should not be listed in the Table 3.7-4. The amendment also would incorporate revisions to the table which correct erroneous nomenclature and valve/penetration relationships. The amendment also would correct the test pressure for the MSIVs to specify a pressure of 28 psig instead of 1/2 Pa. The original staff evaluation (October 30, 1986) used a test pressure of 28 psig, but the term 1/2 Pa was erroneously entered in the Technical Specification.

Environmental Impacts of the Proposed Action:

A. Occupational Radiological Exposure

Correction of the test pressure for the MSIVs and correction of the nomenclature and valve/penetration relationships would have no impact on occupational exposure. Deletion of the valves required to be pressure tested would have a minor impact on occupational exposure by reducing the number of valves needing testing. The change, if any, would be a decrease in occupational exposure.

B. Radiological Impacts During Plant Operation and Accidents

The amendments would have no impact on radiological release during normal plant operation or under accident conditions. The valves to be deleted from the Table 3.7-4 do not constitute leakage pathways since they are sealed by water. The staff analysis for leakage through the MSIVs was originally performed (SER dated October 30, 1986) at a test pressure of 28 psig. The amendment to the Technical Specification would merely correct an erroneous entry of 1/2 Pa to the 28 psig used in the staff analysis.

C. Non-Radiological Impacts

The proposed amendments involve systems located entirely within the restricted area as defined in 10 CFR Part 20. They do not affect non-radiological plant effluents and have no other environmental impact. Therefore, the Commission concludes that there are no significant non-radiological environmental impacts associated with the proposed amendments.

Alternative to the Proposed Actions: Since we have concluded that no adverse environmental impacts are associated with the proposed action, any alternatives would have equal or greater environmental impact and need not be selected.

The principal alternative would be to deny the amendments. This, in effect, would require continuation of unnecessary tests of those valves proposed to be deleted from Table 3.7-4, with an attendant increased exposure potential to plant workers. It would also perpetuate erroneous nomenclature and valve/penetration relationships in the Technical Specifications and a reference to an incorrect test pressure for the MSIVs.

Alternative Use of Resources: This action does not involve the use of resources not previously considered in connection with the Nuclear Regulatory Commission's Final Environmental Statement dated October 1972 related to this facility.

Agencies and Persons Consulted: The NRC staff reviewed the licensee's requests of March 4, 1987, and April 21, 1987, and did not consult other agencies or persons.

Finding of No Significant Impact: The Commission has determined not to prepare an environmental impact statement for the proposed license amendment.


Based upon this environmental assessment, we conclude that the proposed action will not have a significant adverse effect on the quality of the human environment.


For further details with respect to this action, see the request for amendments dated March 4, 1987 as supplemented by request dated April 21, 1987, which are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., 20555 and at the Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513.

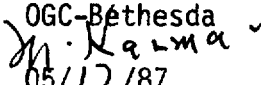
FOR THE NUCLEAR REGULATORY COMMISSION


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B. J. Youngblood, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II

  
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MDuncan  
05/8/87

OGC-Bethesda  
  
05/12/87

  
PD#II-3/DRP-I/II  
BJYoungblood  
05/8/87