LIMITING CONDITION FOR OPERATION

3.4.10 The structural integrity of ASME Code Class 1, 2, and 3 components shall be maintained in accordance with Specification 4.4.10.

APPLICABILITY: All MODES.

ACTION:

a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDT considerations.

b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F.

c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.

SURVEILLANCE REQUIREMENTS

4.4.10 In addition to the requirements of Specification 4.0.5, each reactor coolant pump flywheel shall be inspected at least once every 10 years. This inspection shall be by either of the following examinations:

a. An in-place examination, utilizing ultrasonic testing, over the volume from the inner bore of the flywheel to the circle of one-half the outer radius; or

b. A surface examination, utilizing magnetic particle testing and/or penetrant testing, of the exposed surfaces of the disassembled flywheel.

3/4.4.10 STRUCTURAL INTEGRITY

The inservice inspection and testing programs for ASME Code Class 1, 2, and 3 components ensure that the structural integrity and operational readiness of these components will be maintained at an acceptable level throughout the life of the plant. These programs are in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a(g) except where specific written relief has been granted by the Commission pursuant to 10 CFR 50.55a(g)(6)(i).

Components of the Reactor Coolant System were designed to provide access to permit inservice inspections in accordance with Section XI of the ASME Boiler and Pressure Vessel Code.

As stated in Appendix H of WCAP-14535A (November 1996), Appendix VIII of Section XI of the ASME Boiler and Pressure Vessel Code is not applicable when examining the reactor coolant pump flywheels.

3/4.4.11 REACTOR COOLANT SYSTEM VENTS

Reactor Coolant System vents are provided to exhaust noncondensible gases and/or steam from the Reactor Coolant System that could inhibit natural circulation core cooling. The OPERABILITY of least one Reactor Coolant System vent path from the reactor vessel head and the pressurizer steam space ensures that the capability exists to perform this function.

The valve redundancy of the Reactor Coolant System vent paths serves to minimize the probability of inadvertent or irreversible actuation while ensuring that a single failure of a vent valve, power supply, or control system does not prevent isolation of the vent path.

The function, capabilities, and testing requirements of the Reactor Coolant System vents are consistent with the requirements of Item II.B.1 of NUREG-0737, "Clarification of TMI Action Plant Requirements," November 1980.

6.4.1.7 The SORC shall:

a. Recommend in writing to the Station Director approval or disapproval of items considered under Specification 6.4.1.6a. through d;

b. Render determinations in writing with regard to whether or not each item considered under Specification 6.4.1.6a., b. and d. constitutes a need for a license amendment; and

c. Provide written notification within 24 hours to the Executive Vice President & Chief Nuclear Officer and the NSARC of disagreement between the SORC and the Station Director however, the Station Director shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

RECORDS

6.4.1.8 The SORC shall maintain written minutes of each SORC meeting that, at a minimum, document the results of all SORC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Executive Vice President & Chief Nuclear Officer and the NSARC.

6.4.2 STATION QUALIFIED REVIEWER PROGRAM

FUNCTION

6.4.2.1 The Station Director may establish a Station Qualified Reviewer Program whereby required reviews of designated procedures or classes of procedures required by Specification 6.4.1.6.a are performed by Station Qualified Reviewers and approved by the designated department heads. These reviews are in lieu of reviews by the SORC. However, procedures which require a 10 CFR 50.59 evaluation must be reviewed by the SORC.

RESPONSIBILITIES

6.4.2.2 The Station Qualified Reviewer Program shall:

a. Provide for the review of designated procedures, programs, and changes thereto by a Qualified Reviewer(s) other than the individual who prepared the procedure, program, or change.

b. Provide for cross-disciplinary review of procedures, programs, and changes thereto when organizations other than the preparing organization are affected by the procedure, program, or change.

c. Ensure cross-disciplinary reviews are performed by a Qualified Reviewer(s) in affected disciplines, or by other persons designated by cognizant department heads as having specific expertise required to assess a particular procedure, program or change. Cross-disciplinary reviewers may function as a committee.

d. Provide for a screening of designated procedures, programs and changes thereto to determine if an evaluation should be performed in accordance with the provisions of 10 CFR 50.59 to verify that a need for a license amendment does not exist. This screening will be performed by personnel trained and qualified in performing 10 CFR 50.59 screenings.

e. Provide for written recommendation by the Qualified Reviewer(s) to the responsible department head for approval or disapproval of procedures and programs considered under Specification 6.4.1.6a and that the procedure or program was screened by a qualified individual and found not to require a 10 CFR 50.59 evaluation.

6.4.2.3 If the responsible department head determines that a new program, procedure, or change thereto requires a 10 CFR 50.59 evaluation, that designated department head will ensure the required evaluation is performed to determine if the new procedure, program, or change involves a need for a license amendment. The new procedure, program, or change will then be forwarded with the 10 CFR 50.59 evaluation to SORC for review.

6.4.2.4 Personnel recommended to be Station Qualified Reviewers shall be designated in writing by the Station Director for each procedure, program, or class of procedure or program within the scope of the Station Qualified Reviewer Program.

6.4.2.5 Temporary procedure changes shall be made in accordance with Specification 6.7.3 with the exception that changes to procedures for which reviews are assigned to Qualified Reviewers will be reviewed and approved as described in Specification 6.4.2.2.

RECORDS

6.4.2.6 The review of procedures and programs performed under the Station Qualified Reviewer Program shall be documented in accordance with administrative procedures.

TRAINING AND QUALIFICATION

6.4.2.7 The training and qualification requirements of personnel designated as a Qualified Reviewer in accordance with the Station Qualified Reviewer Program shall be in accordance with administrative procedures. Qualified reviewers shall have:

a. A Bachelors degree in engineering, related science, or technical discipline, and two years of nuclear power plant experience;

OR

b. Six years of nuclear power plant experience;

OR

c. An equivalent combination of education and experience as approved by the designated department head.