

MAR 18 1985

Dockets Nos. 50-321  
and 50-366

Mr. J. T. Beckham, Jr.  
Vice President - Nuclear Generation  
Georgia Power Company  
P. O. Box 4545  
Atlanta, Georgia 30302

Dear Mr. Beckham:

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The Commission has issued the enclosed Amendments Nos. 108 and 45 to Facility Operating Licenses Nos. DPR-57 and NPF-5 for the Edwin I. Hatch Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your applications dated February 6 and August 7, 1984.

The amendments revise the TSs for both Hatch Units 1 and 2 to make them consistent with the guidelines provided in the NRC's Generic Letter 83-36 concerning NUREG-0737 Items II.R.3, Post-Accident Sampling; II.F.1.1, Noble Gas Effluent Monitors; II.F.1.2, Sampling and Analysis of Plant Effluents; II.F.1.3, Containment High Range Monitor; and II.F.1.4, Containment Pressure Monitor.

Your application of February 6, in addition to providing TSs in response to the Generic Letter 83-36 guidance on the above noted NUREG-0737 items, also provided responses to the guidance provided on NUREG-0737 Items II.B.1, Reactor Coolant System Vents; II.F.1.5, Containment Water Level Monitor; II.F.1.6, Containment Hydrogen Monitors; and III.D.3.4, Control Room Habitability. In our evaluation of these responses, we concluded that TSs were not required for Item II.B.1 for Hatch Units 1 and 2 and that the current TSs for Hatch Units 1 and 2 meet the intent of, or are consistent with the Generic Letter 83-36 guidelines for Items II.F.1.5, II.F.1.6 and III.D.3.4 and are acceptable.

8504040029 850318  
PDR ADOCK 05000321  
P PDR

Mr. Beckham, Jr.

-2-

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next Monthly Notice.

Sincerely,

**Original signed by**

George W. Rivenbark, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendments Nos. 108 and 45
2. Safety Evaluation

cc w/enclosures:  
See next page

ORB#4:DL  
RIngram  
3/11/85

ORB#4:DL  
GRivenbark;  
3/11/85

C-ORB#4:DL  
JStolz  
3/11/85

GOELD  
Goddard  
3/14/85

AD:OR:DL  
GLaznas  
3/18/85

Hatch 1/2  
Georgia Power Company

50-321/366

cc w/enclosure(s):

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 108  
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by Georgia Power Company, et al., (the licensee) dated February 6 and August 7, 1984, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

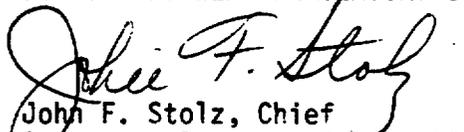
8504040033 850318  
PDR ADOCK 05000321  
P PDR

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 108, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 18, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 108

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

<u>Remove</u>	<u>Insert</u>
3.2-22	3.2-22
----	3.2-22a
3.2-23	3.2-23
3.2-23a	3.2-23a
3.2-23b	3.2-23b
3.2-23c	3.2-23c
----	3.2-23d
----	3.2-48a
3.2-49	3.2-49
3.2-68	3.2-68
----	6-23

Table 3.2-11

## INSTRUMENTATION WHICH PROVIDES SURVEILLANCE INFORMATION

Ref. No. (a)	Instrument (b)	Required Operable Instrument Channels	Type and Range	Action	Remarks
1	Reactor Vessel Water Level	1	Recorder	(c)	(d)
		2	Indicator 0 to 60"	(c)	(d)
2	Shroud Water Level	1	Recorder	(c)	(d)
		1	Indicator -317" to -17"	(c)	(d)
3	Reactor Pressure	1	Recorder 0 to 1500 psig	(c)	(d)
		2	Indicator 0 to 1200 psig	(c)	(d)
4	Drywell Pressure	2	Recorder -10 to +90 psig	(c)	(d)
5	Drywell Temperature	2	Recorder 0 to 500°F	(c)	(d)
6	Suppression Chamber Air Temperature	2	Recorder 0 to 500°F	(c)	(d)
7	Suppression Chamber Water Temperature	2	Recorder 0 to 250°F	(c)	(d)
8	Suppression Chamber Water Level	2	Indicator 0 to 300"	(c)	(d)
		2	Recorder 0 to 30"	(c)(e)	(d)
9	Suppression Chamber Pressure	2	Recorder -10 to +90 psig	(c)	(d)
10	Rod Position Information System (RPIS)	1	28 Volt Indicating Lights	(c)	(d)
11	Hydrogen and Oxygen Analyzer	1	Recorder 0 to 52	(c)	(d)
12	Post LOCA Radiation Monitoring System	1	Recorder	(c)	(d)
			Indicator 1 to 10 <sup>6</sup> R/hr	(c)	(d)
13	a) Safety/Relief Valve Position Primary Indicator	1/SRV	Indicating Light at 85 psig	(f)	
	b) Safety/Relief Valve Position Secondary Indicator	1	Recorder 0 to 600°F	(f)	

TABLE 3.2-11 (Continued)

INSTRUMENTATION WHICH PROVIDES SURVEILLANCE INFORMATION

Ref. No. (a)	Instrument (b)	Required Operable Instrument Channels	Type and Range	Action	Remarks
14	Drywell High Range Pressure	2	Recorder 0 to 250 psig	(c)	(d)
15	Drywell High Range Radiation	2 2	Indicator 1 to $10^7$ R/Hr Recorder 1 to $10^7$ R/Hr	(g) (g)	
16	Main Stack Post-Accident Effluent Monitor	1	Recorder $5 \times 10^{-3}$ to $1 \times 10^5$ $\mu\text{Ci/cc}$	(g)	(h)
17	Reactor Building Vent Plenum Post-Accident Effluent Monitor	1	Recorder $5 \times 10^{-3}$ to $1 \times 10^5$ $\mu\text{Ci/cc}$	(g)	(h)

Amendment No. 108

PLANT HATCH UNIT 1

3.2-22a

NOTES FOR TABLE 3.2-11

- a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in Table 3.2-11 and items in Table 4.2-11
- b. Limiting Conditions for Operation for the Neutron Monitoring System are listed in Table 3.2-7.
- c. From and after the date that one of these parameters is reduced to one indication, continued operation is permissible during the succeeding thirty days unless such instrumentation is sooner made operable.

Continued operation is permissible for seven days from and after the date that one of these parameters is not indicated in the control room. Surveillance of local panels will be substituted for indication in the control room during the seven days.

- d. Drywell and Suppression Chamber Pressure are each recorded on the same recorders. Each output channel has its own recorder.

Drywell and Suppression Chamber air temperature and suppression chamber water temperature are all recorded on the same recorders. Each output channel has its own recorder. Each recorder takes input from several temperature elements.

Hydrogen and Oxygen are indicated on one recorder. The recorder has two pens, one pen for each parameter.

Each channel of the post LOCA radiation monitoring system includes two detectors; one located in the drywell and the other in the suppression chamber. Each detector feeds a signal to a separate log count rate meter. The meter output goes to a two pen recorder. One high radiation level alarm is provided per channel and annunciation of alarm is provided in the control room.

High Range Drywell Pressure and High Range Drywell Radiation are recorded on the same recorders. Each output channel has its own recorder.

- e. In the event that all indications of this parameter is disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.
- f. If either the primary or secondary indication is inoperable, the torus temperature will be monitored at least once per shift to observe any unexplained temperature increase which might be indicative of an open SRV. With both the primary and secondary monitoring channels of two or more SRVs inoperable either restore sufficient inoperable channels such that no more than one SRV has both primary and secondary channels inoperable within 7 days or be in at least hot shutdown within the next 12 hours.

NOTES FOR TABLE 3.2-11 (Continued)

- g. With the plant in the power operation, startup, or hot shutdown condition and with the number of operable channels less than the required operable channels, initiate the preplanned alternate method of monitoring the appropriate parameter within 72 hours and:
  - 1. either restore the inoperable channel(s) to operable status within 7 days of the event, or
  - 2. prepare and submit a special report to the NRC pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability, and the plans and schedule for restoring the system to operable status.
- h. A channel contains two detectors: one for mid-range noble gas, and one for high range noble gas. Both detectors must be operable to consider the channel operable.

TABLE 3.2-12

INSTRUMENTATION WHICH INITIATES THE DISCONNECTION  
OF OFFSITE POWER SOURCES

<u>Ref. No.</u> (a)	<u>Instrument</u> (b)	<u>Required Operable Channels</u>	<u>Channels Required To Trip</u>	<u>Trip Setting</u>	<u>Action to be Taken If the Number of Required Operable Channels Is Not Met</u>
1	4.16 kv Emergency Bus Undervoltage Relay (Loss of Voltage Condition)	2/Bus	2/Bus	greater than or equal to 2800 volts. At 2800 volts time delay will be less than or equal to 6.5 sec.	(c)
2	4.16 kv Emergency Bus Undervoltage Relay (Degraded Voltage Condition)	2/Bus	2/Bus	greater than or equal to 3280 volts. At 3280 volts time delay will be less than or equal to 21.5 sec.	(c)

## NOTES FOR TABLE 3.2-12

- a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in Table 3.2-12 and items in Table 4.2-12.
- b. This instrumentation is required to be operable during reactor startup, power operation, and hot shutdown.
- c. With the number of operable channels one less than the required operable channels, operation may proceed until performance of the next required instrument functional test provided a trip signal is placed in the LUSP lock-out relay logic for the applicable inoperable channel.

TABLE 3.2-13

INSTRUMENTATION WHICH INITIATES ENERGIZATION BY  
ONSITE POWER SOURCES

<u>Ref. No.</u> (a)	<u>Required Instrument</u> (b)	<u>Required Operable Channels</u>	<u>Channels Required To Trip</u>	<u>Trip Setting</u>	<u>Action to be Taken if the Number of Required Operable Channels Is Not Met</u> (c)
1	Start up auxiliary transformer IC loss of voltage condition	2	1	Trip setting greater than or equal to 3280 volts. At 3280 volts trip of relay will be instantaneous (no time delay).	

NOTES FOR TABLE 3.2-13

- a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in Table 3.2-13 on items in Table 4.2-13.
- b. This instrumentation is required to be operable during reactor startup, power operation, and hot shutdown.
- c. With the number of operable channels one less than the required operable channels, operation may proceed provided the relay is removed from its case. Removing the relay accomplishes the same action as an operable relay operating to open its trip circuit.

Table 3.2-14

## INSTRUMENTATION WHICH ARMS LOW LOW SET S/RV SYSTEM

Ref No. (a)	Instrument	Trip Condition Nomenclature	Required Operable Channels per Trip System	Trip Setting	Remarks
1.	Reactor Vessel Steam Dome Pressure	High	2 <sup>(b)</sup>	≤ 1054 psig	
2.	Relief/Safety Valve Tailpipe Pressure	High	2/valve	85, +15, -5 psig	The limiting condition of operation of these switches is provided in Specification 3.6.H.1

a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in table 3.2-14 and items in table 4.2-14.

b. With the requirements for the minimum number of OPERABLE channels not satisfied for one trip system, place the inoperable channel in the tripped condition or declare the associated system inoperable within one hour. With the requirements for the minimum number of OPERABLE channels not satisfied for both trip systems, declare the associated system inoperable within one hour.

TABLE 4.2-11 (Continued)

CHECK AND CALIBRATION MINIMUM FREQUENCY FOR INSTRUMENTATION  
WHICH PROVIDES SURVEILLANCE INFORMATION

Ref. No. (a)	Instrument	Instrument Check Minimum Frequency (b)	Instrument Functional Test Minimum Frequency (d)	Instrument Calibration Minimum Frequency (c)
14	Drywell High Range Pressure	Each Shift	N.A.	Every 6 Months
15	Drywell High Range Radiation	Each Shift	N.A.	Every 18 Months (e)
16	Main Stack Post-Accident Effluent Monitor	N.A.	Every 6 Months	Every 18 Months
17	Reactor Building Vent Plenum Post-Accident Effluent Monitor	N.A.	Every 6 Months	Every 18 Months

Notes for Table 4.2-11

- a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in Table 4.2-11 and items in Table 3.2-11.
- b. Instrument checks are not required when the instruments are not required to be operable or are tripped. However, if instrument checks are missed, they shall be performed prior to returning the instrument to an operable status.
- c. Calibrations are not required when the instruments are not required to be operable or are tripped. However, if calibrations are missed, they shall be performed prior to returning the instrument to an operable status.
- d. Functional tests are not required when the instruments are not required to be operable or are tripped. However, if functional tests are missed, they shall be performed prior to returning the instrument to an operable status.
- e. Calibration of a drywell high range monitor shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/hr and a one point calibration check of the detector below 10 R/hr with an installed or portable gamma source.

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BASES FOR LIMITING CONDITIONS FOR OPERATION

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3.2.J.4. Scintillation Detector For Monitoring Radioiodine (Continued)

level reading is indicative of a leak in the nuclear system process barrier in the primary containment. A sample that is continuously drawn from the primary containment is collected on an iodine filter and monitored by a gamma sensitive scintillation detector. Radiation levels are read out by a log rate meter and recorded on a strip chart located in the control room. A high radiation level alarm and a failure alarm are also provided and are annunciated in the control room. Also, a high-low flow alarm is annunciated in the control room.

5. GM Tubes for Monitoring Noble Gases

A set of GM tubes contained in an instrument rack are used to monitor the release of noble gases in the drywell and torus. A high radiation level reading is indicative of a leak in the nuclear system process barrier in the primary containment. A sample that's continuously drawn from the primary containment is passed through a shielded sample chamber which contains the beta sensitive GM tubes. Radiation levels are read out by a log rate meter and recorded on a strip chart located in the control room. A high radiation level alarm and failure alarm are provided and are annunciated in the control room. Also, a high-flow alarm is annunciated in the control room.

K. Instrumentation Which Provides Surveillance Information (Table 3.2-11)

For each parameter monitored, as listed in Table 3.2-11, there are two channels of instrumentation except for the control rod positions indicating system and the Post-Accident Effluent Monitors. By comparing readings between the two channels, a near continuous surveillance of instrument performance is available. Any significant deviation in readings will initiate an early recalibration, thereby maintaining the quality of the instrument readings.

The hydrogen and oxygen analyzing systems consist of two redundant, separate systems and are each capable of analyzing the hydrogen and oxygen content of the drywell-torus simultaneously. They are designed to be completely testable at both the analyzer rack and in the control room. With an oxygen concentration of less than 4% by volume, a flammable mixture with hydrogen is not possible.

L. Instrumentation Which Initiates Disconnection of Offsite Power Sources (Table 3.2-12)

The undervoltage relays shall automatically initiate the disconnection of offsite power sources whenever the voltage setpoint and time delay limits have been exceeded. This action shall provide voltage protection for the emergency power systems by preventing sustained degraded voltage conditions due to the offsite power source and interaction between the offsite and onsite emergency power systems. The undervoltage relays have a time delay characteristic that provides protection against both a loss of voltage and degraded voltage condition and thus minimizes the effect of short duration disturbances without exceeding the maximum time delay, including margin, that is assumed in the FSAR accident analyses.

ADMINISTRATIVE CONTROL

6.15 POST-ACCIDENT SAMPLING AND ANALYSIS

A program shall be established, implemented, and maintained to ensure the capability to obtain and analyze samples of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere under accident conditions.

The program shall include the following:

- (1) Training of personnel,
- (2) Procedures for sampling and analysis, and
- (3) Provisions for maintenance of sampling and analysis equipment.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA  
DOCKET NO. 50-366  
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.45  
License No. NPF-5

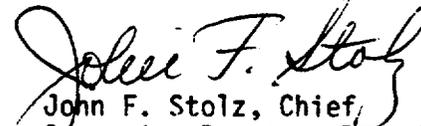
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by Georgia Power Company, et al., (the licensee) dated February 6 and August 7, 1984, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 45, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 18, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf pages are provided to maintain document completeness.

Remove

3/4 3-53

3/4 3-54

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3/4 3-55

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Insert

3/4 3-53

3/4 3-54

3/4 3-54a

3/4 3-55

6-22

INSTRUMENTATION

POST-ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

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3.3.6.4 The post-accident monitoring instrumentation channels shown in Table 3.3.6.4-1 shall be OPERABLE.

APPLICABILITY: CONDITIONS 1, 2, and 3 \*

ACTION:

- a. With one or more of the above required post-accident monitoring channels inoperable, either restore the inoperable channel(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.
- b. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

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4.3.6.4. Each of the above required post-accident monitoring instrumentation channels shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST, and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.6.4-1

\* Condition 3 is applicable only to Items 12, 13, and 14 of Table 3.3.6.4-1.

TABLE 3.3.6.4-1

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Vessel Pressure (2B21-R623 A, B)	2
2. Reactor Vessel Shroud Water Level (2B21-R610, 2B21-R615)	2
3. Suppression Chamber Water Level (2T48-R622 A, B)	2
4. Suppression Chamber Water Temperature (2T47-R626, 2T47-R627)	2
5. Suppression Chamber Pressure (2T48-R608, 2T48-R609)	2
6. Drywell Pressure (2T48-R608, 2T48-R609)	2
7. Drywell Temperature (2T47-R626, 2T47-R627)	2
8. Post-LOCA Gamma Radiation (2D11-K622 A, B, C, D)	2
9. Drywell H <sub>2</sub> -O <sub>2</sub> Analyzer (2P33-R601 A, B)	2
10. a) Safety/Relief Valve Position Primary Indicator (2B21-N301 A-II and K-M)	(a)
b) Safety/Relief Valve Position Secondary Indicator (2B21-N004 A-II and K-M)	(a)
11. Drywell High Range Pressure (2T48-R601 A, B)	2
12. Drywell High Range Radiation (2D11-K621 A, B, 2T48-R601A, B)	2 (b)
13. Main Stack Post-Accident Effluent Monitor (D11-R631)	1 (b)(c)
14. Reactor Building Vent Plenum Post-Accident Effluent Monitor (2D11-R631)	1 (b)(c)

PLANT SAFETY UNIT 2

3/4 3-54

Amendment No. 18, 23, 29, 45

TABLE 4.3.6.4-1

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure	M	N.A.	Q
2. Reactor Vessel Water Level	M	N.A.	Q
3. Suppression Chamber Water Level	M	N.A.	R
4. Suppression Chamber Water Temperature	M	N.A.	R
5. Suppression Chamber Pressure	M	N.A.	R
6. Drywell Pressure	M	N.A.	Q
7. Drywell Temperature	M	N.A.	R
8. Post-LOCA Gamma Radiation	M	N.A.	R
9. Drywell H <sub>2</sub> -O <sub>2</sub> Analyzer	M	N.A.	Q
10. a) Safety/Relief Valve Position Primary Indication	M*	N.A.	R
b) Safety/Relief Valve Position Secondary Indication	M*	N.A.	R
11. Drywell High Range Pressure	M	N.A.	Q
12. Drywell High Range Radiation	M	N.A.	R
13. Main Stack Post-Accident Effluent Monitor	N.A.	SA	R
14. Reactor Building Vent Plenum Post-Accident Effluent Monitor	N.A.	SA	R

\* See 4.4.2.1 a

## INSTRUMENTATION

### SOURCE RANGE MONITORS

#### LIMITING CONDITION FOR OPERATION

3.3.6.5 Three source range monitors shall be OPERABLE.

APPLICABILITY: CONDITIONS 2\*, 3 and 4.

#### ACTION:

- a. In CONDITION 2\* with one of the above required source range monitors inoperable, restore 3 source range monitors to OPERABLE status within 4 hours or be in at least HOT SHUTDOWN within the next 6 hours.
- b. In CONDITION 3 or 4, with two or more of the above required source range monitors inoperable, verify all control rods to be fully inserted in the core and lock the reactor mode switch in the Shutdown position within one hour.

#### SURVEILLANCE REQUIREMENTS

4.3.6.5 Each of the above required source range monitors shall be demonstrated OPERABLE by:

- a. Performance of a:
  1. CHANNEL CHECK at least once per:
    - (a) 12 hours in CONDITION 2\*, and
    - (b) 24 hours in CONDITION 3 or 4.
  2. CHANNEL CALIBRATION\*\* at least once per 18 months.
- b. Performance of a CHANNEL FUNCTIONAL TEST:
  1. Within 24 hours prior to moving the reactor mode switch from the Shutdown position if not performed within the previous 7 days, and
  2. At least once per 31 days.
- c. Verifying, prior to withdrawal of control rods, that the SRM count rate is at least 3 cps with the detector fully inserted.

\* With IRMs on range 2 or below.

\*\* May exclude neutron detectors.

ADMINISTRATIVE CONTROL

6.15 ENVIRONMENTAL QUALIFICATION

6.15.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" [DOR Guidelines]; or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of Licenses DPR-57 and NPF-5 dated October 24, 1980.

6.15.2 By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

## ADMINISTRATIVE CONTROL

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### 6.16 POST-ACCIDENT SAMPLING AND ANALYSIS

A program shall be established, implemented and maintained to ensure the capability to obtain and analyze samples of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere under accident conditions. The program shall include the following:

- (1) Training of personnel,
- (2) Procedures for sampling and analysis, and
- (3) Provisions for maintenance of sampling and analysis equipment.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 108 AND 45 TO

FACILITY OPERATING LICENSES NOS. DPR-57 AND NPF-5

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNITS NOS. 1 AND 2

DOCKETS NOS. 50-321 AND 50-366

1.0 INTRODUCTION AND BACKGROUND

In November 1980, the NRC staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements" which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications (TSs) are required. A number of items which require TSs were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of TSs for all of these items in Generic Letter 83-36. Generic Letter 83-36 was issued to all Boiling Water Reactor (BWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's TSs to determine if they were consistent with the guidance provided in the Generic Letter, and
2. submit an application for a license amendment where deviations or absence of TSs were found.

By letters dated February 6 and August 7, 1984, Georgia Power Company (the licensee) responded to Generic Letter 83-36 by submitting TS change requests for Edwin I. Hatch Nuclear Plant, Units 1 and 2. These change requests responded to all nine items for which we transmitted guidance to the licensee.

These nine items are:

1. Reactor Coolant System Vents (II.B.1)
2. Post-Accident Sampling (II.B.3)
3. Noble Gas Effluent Monitors (II.F.1.1)
4. Sampling and Analysis of Plant Effluents (II.F.1.2)
5. Containment High Range Monitor (II.F.1.3)
6. Containment Pressure Monitor (II.F.1.4)
7. Containment Water Level Monitor (II.F.1.5)
8. Containment Hydrogen Monitor (II.F.1.6)
9. Control Room Habitability Requirements (III.D.3.4)

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## 2.0 EVALUATION

### 1. Reactor Coolant System Vents (II.B.1)

Our guidance states that plants which do not have isolation condensers need not revise the TSs. Neither Hatch 1 or 2 have isolation condensers; therefore, no response was required for this item. We conclude that Item II.B.1 TSs are not required for the Hatch plant.

### 2. Post-Accident Sampling (II.B.3)

The guidance provided by Generic Letter 83-36 requested that an administrative program should be established, implemented and maintained to ensure that the licensee has the capability to obtain and analyze reactor coolant and containment atmosphere samples under accident conditions. The Post-Accident Sampling System is not required to be operable at all times. Administrative procedures are to be established for returning inoperable instruments to operable status as soon as practicable.

The licensee has provided a proposed revision to the TSs which is consistent with the guidelines provided in our Generic Letter 83-36. We conclude that the proposed TS for the Post-Accident Sampling System is acceptable.

### 3. Noble Gas Effluent Monitors (II.F.1.1)

The licensee has supplemented the existing normal range monitors to provide noble gas monitoring in accordance with Item II.F.1.1. Proposed TSs were submitted that are consistent with the guidelines provided in our Generic Letter 83-36. We conclude that the TSs for Item II.F.1.1 are acceptable.

### 4. Sampling and Analysis of Plant Effluents (II.F.1.2)

The guidance provided by Generic Letter 83-36 requested that an administrative program should be established, implemented and maintained to ensure the capability to collect and analyze or measure representative samples of radioactive iodines and particulates in plant gaseous effluents during and following an accident. We conclude that the TSs for sampling and analysis of plant effluents are acceptable.

### 5. Containment High Range Monitor (II.F.1.3)

The licensee has installed two in-containment monitors in each Hatch Unit that are consistent with the guidance of Item II.F.1.3. Generic Letter 83-36 provided guidance for limiting conditions of operation and surveillance TSs for these monitors. The licensee proposed TSs which are consistent with the guidance provided in our Generic Letter 83-36. We conclude that the TSs for Item II.F.1.3 are acceptable.

6. Containment Pressure Monitor (II.F.1.4)

Each Hatch Unit was provided with two supplementary channels of drywell pressure monitoring in response to this item. The licensee proposed TSs that are consistent with the guidelines contained in Generic Letter 83-36. We conclude that the TSs for Item II.F.1.4 are acceptable.

7. Containment Water Level Monitor (II.F.1.5)

Previously existing wide range suppression pool water level monitors provided the capability required by Item II.F.1.5 with the exception of environmental qualification. The monitors have been upgraded to the appropriate environmental standards. Moreover, the current TSs for both Hatch Units contain limiting conditions of operation and surveillance requirements that are consistent with the guidance contained in Generic Letter 83-36. We conclude that the current TSs covering Item II.F.1.5 are acceptable.

8. Containment Hydrogen Units (II.F.1.6)

Previously existing containment hydrogen/oxygen monitors provided the capability required by Item II.F.1.6 with the exception of environmental qualification. These monitors are presently being modified to meet the appropriate standards; however, this is the subject of a separate, ongoing, NRC review. This evaluation, based on responses to Generic Letter 83-36, treats only the TSs for the monitors. The current Hatch Units 1 and 2 TSs contain appropriate limiting conditions of operation and surveillance for these monitors. We conclude that the current TS sections on surveillance and limiting conditions of operation for Item II.F.1.6 are acceptable as they are consistent with the guidance contained in Generic Letter 83-36.

9. Control Room Habitability (III.D.3.4)

The guidance of NUREG-0737 requires assurance on the part of the licensee that control room operators will be adequately protected against the effects of an accidental release of toxic and/or radioactive gases from sources either onsite or offsite. Generic Letter 83-36 provided the guidance on the TSs for a toxic gas detection system and a control room emergency air filtration system.

The licensee, by letter dated December 31, 1980, provided an evaluation of the Hatch control room habitability. The evaluation resulted in the conclusion that the existing systems and TSs met the intent of Item III.D.3.4. The NRC letter of March 9, 1982, transmitted a Safety Evaluation in which the licensee's position on III.D.3.4 was found acceptable. We conclude that the current TSs covering item III.D.3.4 are acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

These amendments involve changes in the installation or use of a facility

component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

#### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that:

- (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
- (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 18, 1985

Principal Contributors: C. Patel, M. Fairtile