

July 7, 1987

Docket No.: 50-321

Mr. James P. O'Reilly
Senior Vice President - Nuclear Operations
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. O'Reilly:

Subject: Issuance of Amendment No.142 to Facility Operating License DPR-57
- Edwin I. Hatch Nuclear Plant, Unit 1 (TAC 64778)

The Commission has issued the enclosed Amendment No. 142 to Facility Operating License DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated May 8, 1987.

The amendment adds a new TS to require analysis for Boron-10 concentration prior to startup from each refueling outage.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,



Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

Enclosures:

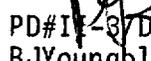
1. Amendment No. 142 to DPR-57
2. Safety Evaluation

cc w/enclosures:

See next page


PD#II-3/DRP-I/II
MDuncan/rad
05/20/87


PD#II-3/DRP-I/II
LCrocker
05/29/87


PD#II-3/DRP-I/II
BJYoungblood
05/25/87

DATED July 7, 1987

AMENDMENT NO. ¹⁴² TO FACILITY OPERATING LICENSE DPR-57, EDWIN I. HATCH, UNIT 1

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Mr. James P. O'Reilly
Georgia Power Company

Edwin I. Hatch Nuclear Plant,
Units Nos. 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-321
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated May 8, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 142, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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B. J. Youngblood, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 7, 1987

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6/10/87

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no note
6/19/87

PD#II-3/DRP-I/II
MDuncan/rad
05/28/87

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PD#II-3/DRP-I/II
LCrocker
05/29/87

~~OGC-Bethesda
M. Kamm
05/29/87~~
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BJYoungblood
06/25/87

ATTACHMENT TO LICENSE AMENDMENT NO. 142

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

Revised
Page

3.4-2

4.4.A.2. Each Operating Cycle (Continued)

- c. vessel. This test checks the explosive charge, proper operation of the associated valves and selected pump operability. The replacement charge to be installed will be selected from a manufactured batch which has been tested.
- d. Both loops including both explosive valves should be tested in the course of two operating cycles.
- e. Prior to startup, verify (by analysis) that the sodium pentaborate enrichment is within prescribed limits.

3.4.B. Operating with Inoperable Components

If one Standby Liquid Control redundant component is inoperable the reactor may remain in operation for a period not to exceed seven (7) days provided the redundant component is operable.

C. Sodium Pentaborate Solution

At all times when the Standby Liquid Control System is required to be operable the following conditions shall be met:

1. Volume

The volume of the liquid control solution in the liquid control tank shall be maintained as required in Figure 3.4-1.

2. Concentration

The concentration of the liquid control tank shall be maintained as required in Figure 3.4-1.

B. Surveillance with Inoperable Components

When a component is found to be inoperable, its redundant component shall be demonstrated to be operable immediately and daily thereafter until the inoperable component is repaired. Continuity of the explosive charge is considered a demonstration of operability.

C. Sodium Pentaborate Solution

The following tests shall be performed to verify the availability of the liquid control solution:

1. Volume

Check the standby liquid control tank volume at least once per day.

2. Concentration

Check the concentration of the liquid in the standby liquid control tank by chemical analysis:



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 142 TO

FACILITY OPERATING LICENSE DPR-57

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-321

INTRODUCTION

On May 28, 1987, Amendment No. 138 to Facility Operating License DPR-57 (Reference 1) was issued to Georgia Power Company (the licensee) changing certain of the Plant Hatch, Unit 1, Technical Specifications (TS). These changes pertained to the use of sodium pentaborate enriched in the isotope Boron-10 in the Standby Liquid Control System (SLCS). The changes, proposed by the licensee's letter of February 6, 1987 (Reference 2) were made to meet the requirements of the Anticipated Transient Without Scram (ATWS) rule, 10 CFR 50.62.

The staff evaluated and accepted the licensee's proposal, as documented in the Safety Evaluation supporting issuance of Amendment 138. However, the staff requested that the licensee make provisions to periodically analyze the concentration of Boron-10 in the sodium pentaborate solution in the SLCS. By letter dated May 8, 1987 (Reference 3), the licensee proposed to add a new surveillance requirement to the TS to require this analysis. The licensee proposed to add a new TS 4.4.A.2.e which would require the analysis for Boron-10 concentration prior to startup from each refueling outage.

EVALUATION

This additional Technical Specification provides added assurance that the concentration of Boron-10 in the sodium pentaborate solution continues to meet the minimum of 60 weight percent B-10 upon which the TS for the Standby Liquid Control System are based. It satisfies the staff's recommendation regarding continuing assurance of Boron-10 concentration in the SLCS and, therefore, is acceptable.

ENVIRONMENTAL CONSIDERATIONS

The amendment involves a change in surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or

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cumulative occupational exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there have been no public comments on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (52 FR 20800) on June 3, 1987, and consulted with the state of Georgia. No public comments were received, and the state of Georgia did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

REFERENCES:

1. Amendment No. 138 to Facility Operating License DPR-57, issued May 28, 1987.
2. Letter from J.P. O'Reilly, Georgia Power Company, to U.S. Nuclear Regulatory Commission, dated February 6, 1987.
3. Letter from L.T. Gucwa, Georgia Power Company, to U.S. Nuclear Regulatory Commission, dated May 8, 1987.

Principal Contributors: L. Crocker

Dated: July 7, 1987