

November 19, 1985

Docket No. 50-321

Mr. J. T. Beckham, Jr.
Vice President - Nuclear Generation
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Beckham:

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The Commission has issued Amendment No.117 to Facility Operating License No. DPR-57 for the Edwin J. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated May 20, 1985.

The amendment revises the TSs to delete reporting requirements related to the secondary containment integrated leak rate test and primary coolant leakage into the drywell.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next Biweekly Notice.

Sincerely,

Original signed by

George W. Rivenbark, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 117
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. J. T. Beckham, Jr.
Georgia Power Company

Edwin J. Hatch Nuclear Plant,
Units Nos. 1 and 2

cc:
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Baxley, Georgia 31513



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-321
EDWIN J. HATCH NUCLEAR PLANT, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 117
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated May 20, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 117, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 19, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 117

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

6-19

Insert

6-19

TABLE 6.9.2-1

SPECIAL REPORTING REQUIREMENTS

<u>Area</u>	<u>Tech Spec Reference</u>	<u>Submittal Date</u>
a. Primary Containment Leak Rate Tests (1)	4.7.A	Within 3 months following conduct of test
b. In-Service Inspection Evaluation	4.6.K	5 years (2)
c. Reactor Coolant Radioactivity in excess of specified limits	4.6.F	Within 30 days of the occurrence

Notes:

1. Each integrated leak rate test of the primary containment shall be the subject of a summary technical report including results of the local leak rate tests since the last report. The report as described in the 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors, " shall include data, analysis, and interpretations of the results which demonstrate compliance in meeting the specified leak rate limits.
2. The report shall be submitted within the period of time listed based on the commercial service date as the starting period



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 117 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN J. HATCH NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-321

Introduction and Evaluation

By letter dated May 20, 1985, the licensee, Georgia Power Company (GPC) requested that plant Technical Specification (TS) 6.9.2 be revised to delete the special reporting requirements as listed in TS Table 6.9.2-1 for the secondary containment drawdown test (Item b) and for primary leakage to the drywell (Item c).

The first report, listed as Item "b" in Table 6.9.2-1, is a summary technical report of the secondary containment drawdown test which is required every refueling outage to measure the ability of the Standby Gas Treatment System to drawdown the secondary containment to negative one-quarter inch water gauge (- $\frac{1}{4}$ "w.g.). We do not currently require a report to be prepared after completion of this test and therefore there is no need for the reporting requirement listed in Technical Specification Table 6.9.2-1.

The second report, listed as Item "c" in Table 6.9.2-1, is a report of primary system leakage to the drywell which is required once every five years. The TS does not define or provide specific guidance on what should be included in this report, and we can identify no basis for the report. (This special report is not included in current BWR Standard Technical Specifications). It should also be noted that Item c has a typographical error in that it references TS 4.6.C (reactor head stud tensioning) rather than TS 4.6.G (reactor coolant leakage).

Hatch TS 3.6.G and 4.6.G currently contain limiting conditions for operation and surveillance requirements for monitoring reactor coolant leakage into the drywell. Should the leakage limits be exceeded, it would be reported per the licensee event report requirements of 10 CFR 50.73. Thus, the special reporting requirement of Item c is not necessary.

Accordingly, we conclude that the licensee's proposal to delete reporting requirements "b" and "c" of Table 6.9.2-1 is acceptable.

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Environmental Considerations

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 19, 1985

Principal Contributors: J. Lane and D. Katze