December 23, 1985

Dockets Nos. 50-321
and 50-366

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Mr. J. T. Beckham, Jr.
Vice President - Nuclear Generation
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendment No. 119 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in partial response to your application dated August 1, 1985.

The amendment revises the TSs for Hatch Unit 1 to update the inservice inspection requirements pursuant to 10 CFR 50.55a(g)(4)(ii). These changes are needed, in part, to help support completion of the first ten-year inservice inspection interval during the Hatch Unit 1 maintenance/refueling outage scheduled to begin November 30, 1985. Other changes requested in your August 1, 1985, application are under review and will be addressed separately.

A copy of our Safety Evaluation and the Notice of Issuance are enclosed.

Sincerely,

Original signed by

George W. Rivenbark, Project Manager
Project Directorate #2
Division of BWR Licensing

Enclosures:
1. Amendment No. 119
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page



*See previous white for concurrences.

| DBL:PD#2 | DBL:PD#2 | DBL:PD#2 | OELD |
|---|---|---|---|
| SNorris* | GRivenbark;cr* | DMuller* | Goddard* |
| 11/18/85 | 11/19/85 | 11/19/85 | 11/25/85 |

12/9/85

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al., (the licensee) dated August 1, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Daniel R. Muller, Director
Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 23, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 179

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

| Remove | Insert |
|---|---|
| iv | iv |
| ix | ix |
| 3.6-9b | 3.6-9b |
| 3.6-10 | 3.6-10 |
| 3.6-11 | 3.6-11 through 3.6-14 (blank page) |
| 3.6-12 | |
| 3.6-13 | |
| 3.6-14 | |
| 3.6-23 | 3.6-23 |
| 3.6-24 | 3.6-24 through 3.6-30 (blank page) |
| 3.6-25 | |
| 3.6-26 | |
| 3.6-27 | |
| 3.6-28 | |
| 3.6-29 | |
| 3.6-30 | |

LIST OF TABLES
(Continued)

D. With the relief valve ... and/or the low low set function of more than one of the above required reactor coolant system relief/safety valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.

3.6.I Jet Pumps

Whenever the reactor is in the Start & Hot Standby or Run Mode with both recirculating pumps operating, all jet pumps shall be operable. If it is determined that a jet pump is inoperable, an orderly shutdown shall be initiated and the reactor shall be in the Cold Shutdown Condition within 24 hours.

4.6 I. Jet Pumps

Whenever both recirculating pumps are operating with the reactor in the Start & Hot Standby or Run Mode, jet pump operability shall be checked daily by verifying that the following conditions do not occur simultaneously.

1. The two recirculation loops have a flow imbalance of 15% or more when the pumps are operated at the same speed.
2. The indicated value of core flow rate varies from the value derived from loop flow measurements by more than 10%.
3. The diffuser to lower plenum differential pressure reading on an individual jet pump vary from the mean of all jet pump differential pressures by more than 10%.

3.6.J. Recirculation Pump Speeds

1. Core thermal power shall not exceed 1% of rated thermal power without forced recirculation.
2. Operation with a single recirculation pump is permitted for 24 hours unless the recirculation pump is sooner made operable. With one recirculation pump not in operation, initiate action within 15 minutes or continue action to reduce reactor power to or below the limit specified in Figure 3.6-5 within 2 hours. If the pump cannot be made operable or the limit of Figure 3.6-5 cannot be met within the required time, the reactor shall be in cold shutdown within 24 hours.
3. Following one pump operation the discharge valve of the low speed pump may not be opened unless the speed of the faster pump is less than 50% of its rated speed.

4.6.J. Recirculation Pump Speeds

Recirculation pump speeds shall be recorded at least once per day.

## 3.6.K STRUCTURAL INTEGRITY

1. Normal Condition

   The structural integrity of ASME Code Class 1, 2, and 3 (equivalent) components shall be maintained in accordance with the Surveillance Requirements of Specification 4.6.K.

2. Off-Normal Conditions

   a. With the structural integrity of any ASME Code Class 1 component not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDT considerations.

   b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 212°F.

   c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.

## 4.6.K STRUCTURAL INTEGRITY

Surveillance Requirements for in-service inspection and testing of ASME Code Class 1, 2, and 3 (equivalent) components shall be applicable as follows:

1. In-service inspection of ASME Code Class 1, 2, and 3 (equivalent) components and in-service testing of ASME Code Class 1, 2, and 3 (equivalent) pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where specific relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i).

2. Performance of the above in-service inspection and testing activities shall be in addition to other specified Surveillance Requirements.

3. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

These pages have been left blank.

3.6.K STRUCTURAL INTEGRITY

In-service inspection of ASME Code Class 1, 2, and 3 (equivalent) components and in-service testing of ASME Code Class 1, 2, and 3 (equivalent) pumps and valves shall be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda as required by 10CFR50.55a(g). This objective will maintain the structural integrity of safety-related components, pumps, and valves which are necessary to safely shut down the plant or mitigate the consequences of an accident.

These pages have been left blank.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

## SUPPORTING AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-321

## 1.0 INTRODUCTION

In the Spring and Fall of 1976, the Nuclear Regulatory Commission sent guidance to power plant licensees regarding the implementation of 10 CFR 50.55a(g) - Inservice Inspection requirements. That guidance directed licensees to submit proposed Technical Specification changes to incorporate standard language referencing 50.55a(g).

In accordance with the provisions of 10 CFR 50.90, as required by 10 CFR 50.59(c), Georgia Power Company (GPC) submitted a request dated August 1, 1985, to amend the Edwin I. Hatch Nuclear Plant, Unit 1, Technical Specifications (Appendix A to Facility Operating License DPR-57) to update the inservice inspection requirements pursuant to 10 CFR 50.55a(g)(4)(ii). The proposed Technical Specification changes are needed, in part, to help support completion of the first ten-year inservice inspection interval during the Hatch Unit 1 maintenance/ refueling outage scheduled to begin November 30, 1985.

## 2.0 EVALUATION

The August 1, 1985, submittal includes a request for changes to other surveillance requirements in addition to those directed at updating the inservice inspection requirements pursuant to 10 CFR 50.55a(g)(4)(ii). This evaluation concerns only those portions of the submittal related to the 10 CFR 50.55a(g)(4)(ii) inservice inspection update. Specifically, we have reviewed the proposed changes on pages 3.6-10 through 3.6-14 and 3.6-23 through 3.6-30 of the Technical Specifications. The other changes requested by the August 1, 1985, submittal are being handled separately.

The proposed changes replace current requirements which do not specify the Code requirements with new requirements that specifically state that inservice inspection of ASME Code Class 1, 2 and 3 (equivalent) components and inservice testing of ASME Code Class 1, 2 and 3 (equivalent) pump and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda.

These proposed changes ensure that inservice inspection of components and piping will be performed in accordance with periodically updated editions and addenda of Section XI of the ASME Boiler and Pressure Vessel Code. The inspection and testing programs include ASME Code Class, 1, 2 and 3 components and piping (including supports) and will provide assurance that the structural integrity of these components and piping will be maintained at an acceptable level throughout the life of the plant.

Based upon the above evaluation, we conclude that the proposed changes represent an increase in the safety of the plant and therefore are acceptable.

3. ENVIRONMENTAL CONSIDERATION

An Environmental Assessment and Finding of No Significant Impact has been issued for this amendment.

4. CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 23, 1985

Principal Contributor: B. Turovlin

7590-01

U.S. NUCLEAR REGULATORY COMMISSION

GEORGIA POWER COMPANY, ET AL

DOCKET NO. 50-321

NOTICE OF ISSUANCE OF AMENDMENT TO

FACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No.119 to Facility Operating License No. DPR-57, issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority for Georgia, City of Dalton, Georgia (the licensees), which revised the Technical Specifications (TSs) for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1 (the facility) located in Appling County, Georgia. The amendment is effective as of the date of its issuance and shall be implemented within 30 days.

This amendment revises the TSs for Hatch Unit 1 to update the inservice inspection requirements pursuant to 10 CFR 50.55a(g)(4)(ii). These changes are needed, in part, to help support completion of the first ten-year inservice inspection interval during the Hatch Unit 1 maintenance/refueling outage scheduled to begin November 30, 1985. Other changes requested in the licensee's August 1, 1985, application are being addressed separately.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Prior Hearing in connection with this action was published in the FEDERAL REGISTER on August 26, 1985, 50 FR 34560. No request for a hearing or petition for leave to intervene was filed following this notice.

Also, in connection with this action, the Commission prepared an Environmental Assessment and Finding of No Significant Impact which was published in the FEDERAL REGISTER on December 16, 1985 (50FR51315).

For further details with respect to this action, see (1) the application for amendment dated August 1, 1985, (2) Amendment No. 119 to License No. DPR-57, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and at the Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 23rd day of December 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

Daniel R. Muller, Director
Project Directorate #2
Division of BWR Licensing