December 26, 1985

Dockets Nos. 50-321
and 50-366

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Mr. J. T. Beckham, Jr.
Vice President - Nuclear Generation
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendments Nos. 120 and 59 to Facility Operating Licenses Nos. DPR-57 and NPF-5, for the Edwin I. Hatch Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated August 23, 1985.

The amendments revise the TSs for Hatch Units 1 and 2 to add and delete valves listed in the containment isolation valve tables to reflect drywell pneumatic system modifications that were made to Unit 2 and that will be made to Unit 1 during the outage scheduled to begin in December, 1985.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next biweekly notice.

Sincerely,

Original signed by

George W. Rivenbark, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:
1. Amendment No. 120 to DPR-57
2. Amendment No. 59 to NPF-5
3. Safety Evaluation

cc w/enclosures:
See next page

BWR:PD#2	BWR:PD#2	BWR:PD#2	OELD
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12/13/85	12/13/85	12/9/85	12/5/85

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 120
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al., (the licensee) dated August 23, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 120, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 26, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 120

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove	Insert
3.7-24	3.7-24

Table 3.7-4
(Continued)

Primary Containment Testable Isolation Valves

Penetration Number	Valve Designation	Notes
X-21	P51-F513 & F514	(1) (2) (4) (5) (9)
X-22	P70-F004, F005	(1) (2) (4) (5) (10)
X-25	T48-F307, F308, F309, F103 & F324	(1) (2) (4) (5) (9)
X-25	T48-F113 & F114	(1) (2) (4) (5) (9)
X-25	T48-F321 & F322	"
X-25	T48-F104, F118A, F118B	(1) (2) (4) (5) (9)
X-26	T48-F319 & F320	(1) (2) (4) (5) (9)
X-26	T48-F334A & F335A	"
X-26	T48-F334B & F335B	"
X-26	T48-F340 & F341	"
X-26	P33-F002 & F010	"
X-27F	P70-F066, F067	(1) (2) (4) (5) (10)
X-28	P33-F003 & F011	(1) (2) (4) (5) (9)
X-31	P33-F004 & F012	"
X-36	C11-F086	(1) (2) (4) (5) (10)
X-36	C11-F083	"
X-39A	E11-F016A & F021A	(1) (2) (4) (5) (9)
X-39A	E11-F016B & F021B	"
X-40	P70-F002 & F003	"
X-41	B21-F019 & F020	"
X-42	C41-F006	(1) (2) (4) (5) (10)
X-42	C41-F007	"
X-46	P21-F353 & F406	(1) (2) (4) (5) (9)
X-203	E-51-F003 & F031	"
X-204A	E11-F065A & F004A	(1) (2) (5) (9) (12)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al., (the licensee) dated August 23, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 59 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 26, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 59

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove	Insert
3/4 6-24	3/4 6-24
3/4 6-25	3/4 6-25
3/4 6-26	3/4 6-26
3/4 6-27	3/4 6-27
3/4 6-28	3/4 6-28

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER	VALVE GROUP[a]	ISOLATION TIME (Seconds)
A. Automatic Isolation Valves (Continued)		
25. Traversing Incore Probe Isolation Valve Ball Valves	*	NA
26. Vacuum Relief Isolation Valves		
2T48-F309	6	5
2T48-F324	6	5

[a] See Specification 3.3.2, Table 3.3.2-1, for isolation signals that operate each valve group.

*Closes upon withdrawal of TIP. TIP automatic withdrawal is actuated by either low reactor vessel water level or high drywell pressure.

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER
B. MANUAL ISOLATION VALVES(e)
1. Main steam isolation valves
2E32-F001B, F, K, P
2. RHR return to recirculation loop isolation valves
2E11-F015A, B
3. LOCA H_2 recombiner isolation valves
2T49-F002 A, B
2T49-F004 A, B
4. Core spray isolation valves
2E21-F005A, B
5. Service air isolation valves
2P51-F651
2P51-F513
6. RBCCW supply and return isolation valves
2P42-F051
2P42-F052

(e) Includes power operated valves which do not isolate automatically.

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER
B. MANUAL ISOLATION VALVES[e] (Continued)
7. Drywell pressure instrumentation line isolation valves
2E11-F041A, B, C, D
2T48-F363A, B
8. ILRT verification flow isolation valves
2T23-F004
2T23-F005
9. Traversing incore probe isolation valve
Shear valve (explosive)
10. N_2 makeup inlet isolation valves
2T48-F321
2T48-F322
2T48-F325
2T48-F327
11. Demineralized water isolation valves
2P21-F032
2P21-F034
12. Chilled water supply and return isolation valves
2P64-F045
2P64-F047
13. Chemical pump discharge isolation valves
2G11-F852
2G11-F853

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER

B. MANUAL ISOLATION VALVES[e] (Continued)

14. Nitrogen vent isolation valves
    2T48-F332 A, B
    2T48-F333 A, B
    2T48-F334 A, B
    2T48-F335 A, B

15. Nitrogen inlet isolation valves
    2T48-F113
    2T48-F114

16. RCIC pump suction isolation valves
    2E51-F003
    2E51-F031

17. RHR pump suction isolation valves
    2E11-F004A, B, C, D

18. Vacuum relief isolation valves
    2T48-F310
    2T48-F311

19. Vacuum relief instrumentation line isolation valve
    2T48-F364A, B

20. Torus water level instrumentation line isolation valves
    2T48-361 A, B
    2T48-362 A, B

21. HPCI pump suction isolation valve
    2E41-F051

22. Core spray pump suction isolation valves
    2E21-F001 A, B

23. Fire protection isolation valve
    2T43-F160

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER
B. MANUAL ISOLATION VALVES[e] (Continued)
24. FPM sample isolation valves
2D11-F058
2D11-F061
25. Torus purification suction isolation valves
2G51-F002
26. RHR relief valve discharge isolation valve
2E11-F103 A, B
27. Nitrogen makeup isolation valves
2T48-F115
2T48-F116
28. Core spray test line isolation valves
2E11-F007 A, B

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER
C. OTHER ISOLATION VALVES
1. Primary feedwater isolation valves
2B21-F010 A, B
2B21-F077 A,(f) B(f)
2. Drywell pneumatic return isolation valves
2P70-F004
2P70-F005
2P70-F066
2P70-F067
3. Recirculation line flow instrumentation line isolation valves(g)
2B31-F009 A, B, C, D
2B31-F010 A, B, C, D
2B31-F011 A, B, C, D
2B31-F012 A, B, C, D
4. Recirculation pump seal purge isolation valves
2B31-F013 A, B
2B31-F017 A, B
5. Recirculation line pressure instrumentation line isolation valves(g)
2B31-F057 A, B
6. Recirculation pump discharge pressure instrumentation line isolation valves(g)
2B31-F040 A, D

(f) Air assist check valve
(g) Excess flow check valve

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 120 AND 59 TO

FACILITY OPERATING LICENSES NOS. DPR-57 AND NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNITS NOS. 1 AND 2

DOCKETS NOS. 50-321 AND 50-366

1.0 INTRODUCTION AND EVALUATION

By letter dated August 23, 1985, Georgia Power Company (GPC) requested changes to the Hatch Units 1 and 2 Technical Specifications to reflect the design modifications of the drywell pneumatic system.

GPC stated that the modifications have been completed for Hatch Unit 2 and will be implemented for Hatch Unit 1 during the Fall of 1985 refueling outage. The modification changes the original single drywell pneumatic header to two separate headers which penetrate the drywell at different locations. Each penetration is provided with two isolation valves powered from two separate divisions. A third valve that has now become unnecessary has been removed from the Unit 2 header. The drywell pneumatic system is designated as an essential system and therefore would automatically isolate only in the case of a break within the system. This design modification will assure a long-term pneumatic supply to some safety relief valves. The proposed Technical Specification changes would modify the tables of containment isolation valves to add the two new isolation valves for each unit and to delete the valve that was removed from the Unit 2 header.

Based on our review of the design change and proposed supporting Technical Specification changes, we conclude that they enhance the safety of the plants, are in compliance with all regulations and appropriate NRC Standard Review Plan sections and are acceptable.

2.0 ENVIRONMENTAL CONSIDERATION

The amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure.

The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

3.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 26, 1985

Principal Contributors: F. Eltawila and D. Katze