

October 18, 2001

Mr. J. A. Stall  
Senior Vice President, Nuclear and  
Chief Nuclear Officer  
Florida Power and Light Company  
P.O. Box 14000  
Juno Beach, Florida 33408-0420

SUBJECT: ST. LUCIE UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS REGARDING  
MINOR CHANGES AND CORRECTIONS (TAC NOS. MB1886 AND MB1904)

Dear Mr. Stall:

The Commission has issued the enclosed Amendment Nos. 177 and 119 to Facility Operating License Nos. DPR-67 and NPF-16 for the St. Lucie Plant, Units Nos. 1 and 2. These amendments consist of changes to the Unit 1 Operating License and the Unit 1 and 2 Technical Specifications (TS) in response to your application dated April 17, 2001.

These amendments change the St. Lucie Unit 1 Operating License and the Unit 1 and 2 TS to correct various minor errors and to incorporate conforming changes necessitated by the revision to the rule in Title 10, *Code of Federal Regulations*, Section 50.59.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Brendan T. Moroney, Project Manager, Section 2  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-335  
and 50-389

Enclosures:

1. Amendment No. 177 to DPR-67
2. Amendment No. 119 to NPF-16
3. Safety Evaluation

cc w/enclosures: See next page

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DATE	10/17/01	10/17/01	09/21/01	10/12/01	10/18/01

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FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 177  
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power & Light Company (the licensee), dated April 17, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Operating License and Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-67 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 177 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

**/RA/**

Richard P. Correia, Chief, Section 2  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications and Operating License

Date of Issuance: October 18, 2001

ATTACHMENT TO LICENSE AMENDMENT NO. 177

TO FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace page 3 of Operating License No. DPR-67 with the attached page.

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove Pages

3/4 3-15  
3/4 3-21  
3/4 3-42  
3/4 5-3  
3/4 5-7  
6-2  
6-4  
6-7  
6-8  
6-10  
6-14  
6-23

Insert Pages

3/4 3-15  
3/4 3-21  
3/4 3-42  
3/4 5-3  
3/4 5-7  
6-2  
6-4  
6-7  
6-8  
6-10  
6-14  
6-23

FLORIDA POWER & LIGHT COMPANY  
ORLANDO UTILITIES COMMISSION OF  
THE CITY OF ORLANDO, FLORIDA  
AND  
FLORIDA MUNICIPAL POWER AGENCY  
DOCKET NO. 50-389  
ST. LUCIE PLANT UNIT NO. 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119  
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated April 17, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

**/RA/**

Richard P. Correia, Chief, Section 2  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 18, 2001

ATTACHMENT TO LICENSE AMENDMENT NO. 119

TO FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove Pages

3/4 3-43  
3/4 5-3  
3/4 5-7  
6-8  
6-9  
6-11  
6-14

Insert Pages

3/4 3-43  
3/4 5-3  
3/4 5-7  
6-8  
6-9  
6-11  
6-14



SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 177 AND 119

TO FACILITY OPERATING LICENSES NOS. DPR-67 AND NPF-16

FLORIDA POWER AND LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNITS NOS. 1 AND 2

DOCKET NOS. 50-335 AND 50-389

1.0 INTRODUCTION

By letter dated April 17, 2001, Florida Power and Light Company, et al., (FPL, the licensee) requested amendments to Operating Licenses DPR-67 and NPF-16 for St. Lucie Unit 1 and 2, respectively. The proposed amendments would revise the Technical Specifications (TS) to correct minor errors and to incorporate conforming changes to reflect the revision to 10 CFR 50.59, and correct a minor error in one condition of the St. Lucie Unit 1 operating license.

2.0 EVALUATION

2.1 Operating License DPR-67, St. Lucie Unit 1

Paragraph 2.C(3) of Operating License DPR-67 contains a listing of several submittals regarding the fire protection program. The date of FPL letter L-83-227 is listed as "April 22, 1983". The licensee indicated that this date is incorrect and proposes to change the date to "April 12, 1983."

Amendment 115 dated June 11, 1992, revised Paragraph 2.C(3). The date of FPL letter L-83-227 was apparently listed incorrectly. The letter was reviewed in the U.S. Nuclear Regulatory Commission (NRC) document system (Accession No. 8305040563) and the date was verified to be April 12, 1983. Therefore, the proposed change represents only a correction of an administrative error and is acceptable.

Paragraph 2.C(3) of Operating License DPR-67 also contains a listing of NRC letters approving the fire protection program. The licensee stated that the date of the October 4, 1988, letter was incorrectly listed as October 4, 1998. A review of Amendment 115 verified that the 1988 date was listed correctly. This was discussed with the licensee by telephone on August 28, 2001. The licensee determined that its copy contained a typographical error, which apparently was introduced when the licensee's copy was reformatted following a subsequent amendment. Thus, there is no need for a revision to Paragraph 2.C(3). The licensee initiated a corrective action program Condition Report to identify the error and the need to correct its copy of the Operating License.

## 2.2 TS page 3/4 3-15, St. Lucie Unit 1

The listing of previous, superseded revisions at the bottom of TS page 3/4 3-15 includes Amendment 74. The licensee indicates this is incorrect and proposes to change the list to show that this page was previously revised by Amendment 72, and not Amendment 74.

TS page 3/4 3-15 contains Table 3.3-4, "Engineered Safety Feature Actuation System Instrumentation Trip Values." This table was revised by Amendment 72 on January 15, 1986. Amendment 74 dated June 10, 1986, addressed different parts of the TS and did not include a change to page 3/4 3-15. Therefore, the proposed change represents only a correction of an administrative error and is acceptable.

## 2.3 TS 3.3.3.1, St. Lucie Unit 1

The Limiting Condition for Operation (LCO) for TS 3.3.3.1, "Monitoring Instrumentation Radiation Monitoring," states "The radiation monitoring instrumentation channels shown in Table 3.3-6 shall be OPERABLE\* with their alarm setpoints within the specified limits." The asterisk with the word OPERABLE refers to a footnote that reads "\*The emergency power source may be inoperable in Modes 5 and 6." It may be inferred from this statement that a radiation monitor that is capable of being powered from an Emergency Diesel Generator (EDG) must have an operable EDG to be considered operable in Modes 1 through 4. The licensee proposes to delete the asterisk and its associated statement from TS 3.3.3.1.

The asterisk statement was added to the TS by Amendment 40 (May 28, 1981), which also changed the definition of OPERABILITY such that both normal and emergency electrical power was required to consider a component OPERABLE. (It should be noted that the definition of OPERABILITY was revised again by TS Amendment 69 (December 6, 1985). The current definition only requires that "electrical power" be available, which is also consistent with the provisions of NUREG-1432, "Standard Technical Specifications Combustion Engineering Plants," to have "normal or emergency electrical power capable of performing its support function"). The licensee submitted the request for Amendment 40 in response to an NRC letter to all power reactor licensees dated April 10, 1980, which requested that all plant TS be changed to incorporate the Standard TS definition of operability. The April 10, 1980, letter stated that "...Standard Technical Specifications (STS) were formulated to preserve the single failure criterion for systems that are relied upon in the safety analysis report. By and large, the single failure criterion is preserved by specifying Limiting Conditions for Operation (LCOs) that require all redundant components of safety related systems to be OPERABLE."

With the exception of the containment radiation monitors, the radiation detectors listed in Table 3.3-6 are single train quality-related systems, and redundant capability is not provided by design. Although many of the quality-related, single train radiation monitors are capable of being powered from an emergency power source, they do not provide critical functions requiring an operable emergency power backup. Based on the standard set forth in the April 10, 1980, letter, there is no reason to require an operable EDG in Modes 1 through 4 for any radiation monitors other than the containment radiation monitors. The containment radiation monitors are part of a safety-related, redundant system used for initiating the containment isolation signal. The need for emergency power and/or redundant component operability applies to these instruments, and is assured by following the requirements of TS 3.8.1.1, "AC Sources, Operating, Limiting Conditions for Operation."

The original reason for adding the asterisked statement is unclear. The April 10, 1980, letter from the NRC asked licensees to conform their TS to the Standard TS, which did not contain this statement. The asterisked statement was not specifically addressed in the amendment request or the safety evaluation associated with Amendment 40. The statement does not appear in the comparable section of the Unit 2 TS. It may have been an attempt to clarify the requirement, based on the assumption that both normal and emergency power sources were needed for operability. The licensee states that its submittal for TS Amendment 103 should have deleted the asterisk and the associated statement regarding radiation monitor operability. TS Amendment 103 (June 7, 1990) consolidated the EDG operability and support system requirements into TS 3.8.1.1, and it would have been acceptable to delete the asterisked statement at that time. The statement could also have been deleted by Amendment 69, when the definition of OPERABILITY was changed. Notwithstanding these missed opportunities, based on the design of the instruments, the current definition of OPERABILITY, and TS 3.8.1.1, the statement is not needed and, in fact, imposes an unnecessary requirement on most of the radiation monitors. Deleting the asterisked statement will not change the operability requirements for the safety-related containment radiation monitors. Therefore, the proposed change is acceptable.

#### 2.4 TS Table 3.3-11, St. Lucie Unit 1

TS 3.3.3.8 provides Limiting Conditions for Operation of Accident Monitoring Instrumentation, as listed in Table 3.3-11. TS Table 3.3-11, "Accident Monitoring Instrumentation," lists three channels of pressurizer-water-level instrumentation under the "Total No. of Channels" column. The licensee proposes to change this to indicate that there are only two channels. Pressurizer level channels LI-1110X and LI-1110Y are environmentally qualified and credited to satisfy Regulatory Guide 1.97 instrumentation requirements. The third channel, LI-1103, is not environmentally qualified, is not powered by Class 1E power, and is not credited to satisfy Regulatory Guide 1.97 requirements.

The existence of two safety-related pressurizer-level instruments is confirmed in the system description in the Updated Final Safety Analysis Report (UFSAR), Section 7.5, "Safety Related Display Instrumentation." The requirement for only two channels is consistent with NUREG-1432 and with the comparable section of the St. Lucie Unit 2 TS. The dependence on the two qualified channels is consistent with the BASES for TS 3.3.3.8, which provide that instrumentation be available during and after an accident to assess plant conditions. The proposed change is not a result of a modification. It corrects an inaccuracy in the number of safety-related pressurizer-level instruments listed in TS Table 3.3-11. Therefore, since the change makes TS Table 3.3-11 consistent with the UFSAR and accurately reflects installed instrumentation, it is administrative in nature and the staff finds this change acceptable.

#### 2.5 TS 3.5.2 and 3.5.3, St. Lucie Units 1 and 2

Currently, the heading for TS 3.5.2 reads, "ECCS Subsystems -  $T_{ave} \geq 325^{\circ}\text{F}$ " and the heading for TS 3.5.3 reads, "ECCS Subsystems -  $T_{ave} < 325^{\circ}\text{F}$ ." Both TS 3.5.2 and TS 3.5.3 contain requirements for ECCS components in Operational Mode 3. Since the definition of Mode 3 includes  $T_{ave} \geq 325^{\circ}\text{F}$ , the heading for TS 3.5.3 is not correct. The correct breakpoint for the requirements in TS 3.5.2 and TS 3.5.3 is based on pressurizer pressure, not  $T_{ave}$ . The dependence on pressurizer pressure is clearly indicated in each section by an asterisk on Mode 3 and a footnote. The licensee proposes to change the headings for TS 3.5.2 and 3.5.3

to "ECCS Subsystems-Operating" and "ECCS Subsystems-Shutdown," respectively. These changes are consistent with the terminology used in NUREG-1432.

The proposed changes do not change any requirements regarding operability of ECCS components. They provide a more accurate description of the plant conditions to which the limits apply, and are administrative in nature. Therefore, the staff finds the proposed changes acceptable.

## 2.6 TS 6.2.2, St. Lucie Unit 1

TS 6.2.2 lists a number of requirements regarding the unit staff organization. TS Amendment 115 dated June 11, 1992, deleted subparagraph 6.2.2.e, which contained requirements regarding the Fire Brigade that were relocated out of the TS. The amended page has a blank space where subparagraph 6.2.2.e was located and a sidebar indicating a revision. The licensee proposes to add "e. Deleted." in the space following 6.2.2.d to clearly indicate the deleted section. Since the blank space is followed by subparagraphs 6.2.2.f and 6.2.2.g, the TS 6.2.2 subparagraphs were incorrectly updated when the amendment was implemented. The proposed change will add clarity and eliminate potential confusion. Therefore, the proposed change represents only a correction of an administrative error and is acceptable.

## 2.7 TS Page 6-4, St. Lucie Unit 1

The listing of previous, superseded revisions at the bottom of TS page 6-4 includes Amendment 57. The licensee indicates this is incorrect and proposes to change the list to show that this page was previously revised by Amendment 51 not Amendment 57.

TS page 6-4 contains Table 6.2-1, "Minimum Shift Crew Composition, Two Units With Two Separate Control Rooms." This table was revised by Amendment 51 on July 29, 1982. Amendment 57 dated May 3, 1983, addressed different parts of the TS and did not include a change to page 6-4. Therefore, the proposed change represents only a correction of an administrative error and is acceptable.

## 2.8 TS Page 6-14, St. Lucie Unit 1

The listing of previous, superseded revisions at the bottom of TS page 6-14 includes Amendment 125. The licensee indicates this is incorrect and proposes to change the list to show that this page was previously revised by Amendment 126 not Amendment 125.

TS page 6-14 contains administrative requirements. This page was revised by Amendment 126 on May 2, 1994. Amendment 125 dated December 16, 1993, addressed different parts of the TS and did not include a change to page 6-14. Therefore, the proposed change represents only a correction of an administrative error and is acceptable.

## 2.9 TS Page 6-15c, St. Lucie Units 1 and 2

These revisions updated an amendment request in FPL letter L-2001-13 dated January 17, 2001, which was previously submitted and dealt with establishment of a TS Bases Control Program. They were treated as a supplemental submittal and reviewed and approved in

conjunction with Amendments 176 and 117 to the licenses of St. Lucie Unit 1 and 2, respectively, which were issued July 12, 2001.

#### 2.10 TS 6.14, St. Lucie Unit 1

TS page 6-23 contains TS 6.14, "Offsite Dose Calculation Manual (ODCM)." The first sentence of TS 6.14 incorrectly states the abbreviation as "OCDM" and the licensee proposes to change this obvious typographical error to "ODCM." Therefore, the proposed change represents only a correction of an administrative error and is acceptable.

#### 2.11 TS Table 4.3-7, St. Lucie Unit 2

TS 3.3.3.6 provides LCO of accident monitoring instrumentation listed on TS Table 3.3-10. TS 4.3.3.6 prescribes surveillance requirements for these same instruments as delineated in TS Table 4.3-7. On October 4, 2000, the NRC issued Amendment 109 to the St. Lucie Unit 2 TS. This changed the Reactor Coolant Outlet Temperature instrument listed on Table 3.3-10 from the "Narrow Range" to the "Wide Range" instrument. However, Amendment 109 did not make a corresponding change to TS Table 4.3-7. The licensee proposes to change TS Table 4.3-7 to indicate that the surveillance requirement applies to Reactor Coolant Outlet Temperature Wide Range instrument.

The purpose of TS Table 4.3-7 is to provide surveillance requirements for the instruments listed on TS Table 3.3-10 to ensure that accident monitoring instrumentation remains operable. With the exception of Reactor Coolant Outlet Temperature, all instruments on each table are the same. TS Table 4.3-7 should have been changed by Amendment 109 to ensure that surveillance is performed on the Reactor Coolant Outlet Temperature instrument corresponding to the instrument listed in TS Table 3.3-10. Failure to do so was an administrative oversight, and should be corrected. Therefore, the proposed change is acceptable.

#### 2.12 Conforming Changes for Title 10, *Code of Federal Regulations* (10 CFR), Section 50.59

Section 6 of each unit's TS addresses Administrative Controls. Included therein are responsibilities of the Facility Review Group and the Company Nuclear Review Board regarding review of activities covered by 10 CFR 50.59, "Changes, Tests, and Experiments." Currently, these review activities refer to "safety evaluation" and "unreviewed safety question," which reflects terminology used in the previous version of 10 CFR 50.59. A revision to 10 CFR 50.59, which became effective on March 13, 2001, eliminated these terms. The licensee proposes to revise appropriate sections of TS Section 6 to replace the terms "safety evaluation" and "unreviewed safety question" with language that conforms to that in the current version of 10 CFR 50.59. The changes in wording do not change the functional responsibilities of the Facility Review Group and the Company Nuclear Review Board, as described in the TS. Also, the proposed wording changes are consistent with previously approved amendments establishing a TS Bases Control Program (see 2.9 above). Those amendments adopted the wording of NUREG-1432. The licensee's changes to its TS Bases Control Program incorporate the review requirements of the revised 10 CFR 50.59. Therefore, the proposed changes only remove obsolete terminology, update the TS to reflect the current version of 10 CFR 50.59, are administrative in nature, and are acceptable.

### 3.0 STATE CONSULTATION

Based upon a letter dated March 8, 1991, from Mary E. Clark of the State of Florida, Department of Health and Rehabilitative Services, to Deborah A. Miller, Licensing Assistant, U.S. Nuclear Regulatory Commission, the State of Florida does not desire notification of issuance of license amendments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment to TS 3.3.3.1 for St. Lucie Unit 1 changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The remaining amendments relate to changes in record keeping, administrative procedures or requirements. The NRC staff has determined that the amendments involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (66 FR 29357, dated May 30, 2001). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Brendan Moroney

Date: October 18, 2001

Mr. J. A. Stall  
Florida Power and Light Company

**ST. LUCIE PLANT**

cc:  
Senior Resident Inspector  
St. Lucie Plant  
U.S. Nuclear Regulatory Commission  
P.O. Box 6090  
Jensen Beach, Florida 34957

Mr. R. G. West  
Plant General Manager  
St. Lucie Nuclear Plant  
6351 South Ocean Drive  
Jensen Beach, Florida 34957

Joe Myers, Director  
Division of Emergency  
Preparedness  
Department of Community Affairs  
2740 Centerview Drive  
Tallahassee, Florida 32399-2100

E. J. Weinkam  
Licensing Manager  
St. Lucie Nuclear Plant  
6351 South Ocean Drive  
Jensen Beach, Florida 34957

M. S. Ross, Attorney  
Florida Power & Light Company  
P.O. Box 14000  
Juno Beach, FL 33408-0420

Mr. Don Mothena  
Manager, Nuclear Plant Support Services  
Florida Power & Light Company  
P.O. Box 14000  
Juno Beach, FL 33408-0420

Mr. Douglas Anderson  
County Administrator  
St. Lucie County  
2300 Virginia Avenue  
Fort Pierce, Florida 34982

Mr. Rajiv S. Kundalkar  
Vice President - Nuclear Engineering  
Florida Power & Light Company  
P.O. Box 14000  
Juno Beach, FL 33408-0420

Mr. William A. Passetti, Chief  
Department of Health  
Bureau of Radiation Control  
2020 Capital Circle, SE, Bin #C21  
Tallahassee, Florida 32399-1741

Mr. J. Kammel  
Radiological Emergency  
Planning Administrator  
Department of Public Safety  
6000 SE. Tower Drive  
Stuart, Florida 34997

Mr. Donald E. Jernigan, Site Vice President  
St. Lucie Nuclear Plant  
6501 South Ocean Drive  
Jensen Beach, Florida 34957