DRC 016

February 22, 1984

Docket Nos. 50-321
and 50-366

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EJordan
JTaylor

Mr. J. T. Beckham, Jr.
Vice President, Nuclear Division
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendment Nos. 98 and 35 to Facility Operating Licenses Nos. DPR-57 and NPF-5, respectively for the Edwin I. Hatch Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your applications dated April 5, 1982; June 11, 1982 as corrected October 15, 1982; November 10, 1982 and March 31, 1983.

The amendments revise the TSs for Hatch Unit 2 to: 1) extend the time allowed to restore thermal power scram and control rod block trip setpoints to within allowable limits, and 2) correct an erroneous reference to a section of the TSs.

The amendments also revise the TSs for both Hatch Unit 1 and Unit 2 to: 1) increase the minimum shift crew composition and 2) increase the frequency of independent audits of both the emergency and security plans.

Your application dated April 5, 1982, also requested changes for Unit 1 which have been completed and an additional change for Unit 2 which is being handled separately.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next Monthly Notice.

Sincerely,

original signed by:
George Rivenbark, Project Manager
Operating Reactors Branch No. 4
Division of Licensing

Enclosures:
1. Amendments Nos. 98 and 35
2. Safety Evaluation

cc: w/enclosures
See next page

ORB#4:DL	ORB#4:DL	ORB#4:DL	OELD	AD/OR:DL
RIngram	GRivenbark;ef	JStolz	Goddard	GLainas
02/9/84	02/9/84	02/9/84	02/14/84	02/21/84

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 98
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment by Georgia Power Company, et al., (the licensee) dated June 11, 1982, as corrected October 15, 1982; and March 31, 1983, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 98, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 22, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 98

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove	Insert
6-4	6-4
6-11	6-11

TABLE 6.2.2-1

MINIMUM SHIFT CREW COMPOSITION#

Condition of Unit 1 - Unit 2 in Reactor Power Operation, Hot Standby or Hot Shutdown Condition

LICENSE CATEGORY	APPLICABLE OPERATIONAL CONDITIONS	
	1, 2, 3	4 & 5
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1	1

Condition of Unit 1 - Unit 2 in Cold Shutdown Condition or Refuel Mode

LICENSE CATEGORY	APPLICABLE OPERATIONAL CONDITIONS	
	1, 2, 3	4 & 5
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1	None

Condition of Unit 1 - No Fuel in Unit 2

LICENSE CATEGORY	APPLICABLE OPERATIONAL CONDITIONS	
	1, 2, 3	4 & 5
SOL	2	1*
OL	3	2
Non-Licensed	3	3
Shift Technical Advisor	1	None

*Does not include the Licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS.

**Assumes each individual is licensed on both units.

#Shift crew composition, including an individual qualified in radiation protection procedures, may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2.2-1.

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the SRB. Each inspection or audit shall be performed within the specified time interval with:

1. A maximum allowable extension not to exceed 25% of the inspection or audit interval.

2. A total maximum combined interval time for any 3 consecutive inspection or audit intervals not to exceed 3.25 times the specified inspection or audit interval.

These audits shall encompass:

a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.

b. The performance, training and qualifications of the entire unit staff at least once per 12 months.

c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.

d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.

e. The Emergency Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.

f. The Security Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.

g. Any other area of unit operation considered appropriate by the SRB or the Senior Vice President Power Supply.

h. The Fire Protection Program and implementing procedures at least once per 24 months.

i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.

j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years. During the year in which the inspection or audit occurs, the requirements of 6.5.2.8i can be affected concurrently.

AUTHORITY

6.5.2.9 The SRB shall report to and advise the Executive Vice President - Power Supply on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment by Georgia Power Company, et al., (the licensee) dated April 5, 1982; June 11, 1982, as corrected October 15, 1982; November 10, 1982; and March 31, 1983, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amenament can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 35, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 22, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 35

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove	Insert
3/4 2-5	3/4 2-5
3/4 5-9	3/4 5-9
6-4	6-4
6-10	6-10

3/4.2.2 APRM SETPOINTS

LIMITING CONDITION FOR OPERATION

3.2.2 The APRM flow referenced simulated thermal power scram trip setpoint (S) and control rod block trip setpoint (S_{RB}) shall be established* according to the following relationships:

$$S \leq (0.66W + 51\%)$$

$$S_{RB} \leq (0.66W + 42\%)$$

where: S and S_{RB} are in percent of RATED THERMAL POWER, and
W = Loop recirculation flow in percent of rated flow.

APPLICABILITY: CONDITION 1, when THERMAL POWER $\geq$ 25% of RATED THERMAL POWER.

ACTION:

With S or S_{RB} exceeding the allowable value, initiate corrective action within 15 minutes and continue corrective action so that S and S_{RB} are within the required limits* within 6 hours or reduce THERMAL POWER to less than 25% of RATED THERMAL POWER within the next 4 hours.

SURVEILLANCE REQUIREMENTS

4.2.2 The CMFLPD shall be determined and the APRM flow referenced simulated thermal power scram and control rod block trip setpoints or APRM readings adjusted, as required:

a. At least once per 24 hours,

b. Whenever THERMAL POWER has been increased by at least 15% of RATED THERMAL POWER and steady state operating conditions have been established, and

c. Initially and at least once per 12 hours when the reactor is operating with a CMFLPD $\geq$ FRTP.

*With CORE MAXIMUM FRACTION OF LIMITING POWER DENSITY (CMFLPD) greater than the fraction of RATED THERMAL POWER (FRTP), $\frac{\text{THERMAL POWER}}{\text{RATED THERMAL POWER}}$, up to 95% of RATED THERMAL POWER, rather than adjusting the APRM setpoints, the APRM gain may be adjusted such that APRM readings are greater than or equal to 100% times CMFLPD, provided that the adjusted APRM reading does not exceed 100% of RATED THERMAL POWER and the required gain adjustment increment does not exceed 10% of RATED THERMAL POWER.

3/4.2.3 MINIMUM CRITICAL POWER RATIO

LIMITING CONDITION FOR OPERATION

3.2.3 The MINIMUM CRITICAL POWER RATIO (MCPR), as a function of average scram time and core flow, shall be equal to or greater than shown in Figure 3.2.3-1, Figure 3.2.3-2 or Figure 3.2.3-3 multiplied by the K_f shown in Figure 3.2.3-4, where:

$$\tau = 0 \text{ or } \left(\frac{\tau_{ave} - \tau_B}{\tau_A - \tau_B}\right), \text{ whichever is greater,}$$

τ_A = 1.096 sec (Specification 3.1.3.3. scram time limit to notch 36),

$$\tau_B = 0.834 + 1.65\left[\frac{N_1}{\sum_{i=1}^{n} N_i}\right]^{\frac{1}{2}}(0.059),$$

$$\tau_{ave} = \frac{\sum_{i=1}^{n} N_i \tau_i}{\sum_{i=1}^{n} N_i}$$

n = number of surveillance tests performed to date in cycle,
N_i = number of active control rods measured in the i^{th} surveillance test,
Ti = average scram time to notch 36 of all rods measured in the i^{th} surveillance test, and
N_1 = total number of active rods measured in 4.1.3.2.a.

APPLICABILITY: CONDITION 1, when THERMAL POWER ≥ 25% RATED THERMAL POWER

ACTION:

With MCPR less than the applicable limit determined from Figure 3.2.3-1, Figure 3.2.3-2 or Figure 3.2.3-3, initiate corrective action within 15 minutes and continue corrective action so that MCPR is equal to or greater than the applicable limit within 2 hours or reduce THERMAL POWER to less than 25% of RATED THERMAL POWER within the next 4 hours.

SURVEILLANCE REQUIREMENTS

4.2.3 The MCPR limit at rated flow shall be determined for each type of fuel (8X8R, P8X8R, and 7X7) from Figures 3.2.3-1, 3.2.3-2, and 3.2.3-3 using:

a. τ = 1.0 prior to the initial scram time measurements for the cycle performed in accordance with Specification 4.1.3.2.a, or

3/4.5.4 SUPPRESSION CHAMBER

LIMITING CONDITION FOR OPERATION

3.5.4 The suppression chamber shall be OPERABLE with a minimum contained water volume of 653,000 gallons, equivalent to a level of 12'2", and the water level instrumentation channels alarms adjusted to actuate at a low water level $\geq$ 12'2", except that the suppression chamber may be drained:

a. In OPERATIONAL CONDITION 4, provided that;

1. No work is performed which has a potential for draining the reactor vessel,

2. The reactor mode switch is locked in the Shutdown position, and

3. The core spray system is OPERABLE per Specification 3.5.3.1 with an OPERABLE flow path capable of taking suction from the OPERABLE condensate storage tank and transferring the water through the spray sparger to the reactor vessel.

b. In OPERATIONAL CONDITION 5, provided that the reactor mode switch is locked in the Refuel position, and:

1. The core spray system is OPERABLE per Specification 3.5.3.1 with an OPERABLE flow path capable of taking suction from the OPERABLE condensate storage tank and transferring the water through the spray sparger to the reactor vessel, or

2. The reactor vessel head is removed and the cavity is flooded, the spent fuel pool gates are removed, and the water level is maintained within the limits of Specifications 3.9.9 and 3.9.10

APPLICABILITY: CONDITION 1, 2, 3, 4 and 5.

ACTION:

a. In CONDITION 1, 2 or 3 with the suppression chamber water level less than the above limit, restore the water level to within the limit within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. In CONDITION 4 or 5 with the suppression chamber drained and the conditions of Specification 3.5.4.a or 3.5.4.b, as applicable, not satisfied, suspend all operations in the reactor vessel and all positive reactivity changes. The provisions of Specification 3.0.3 are not applicable.

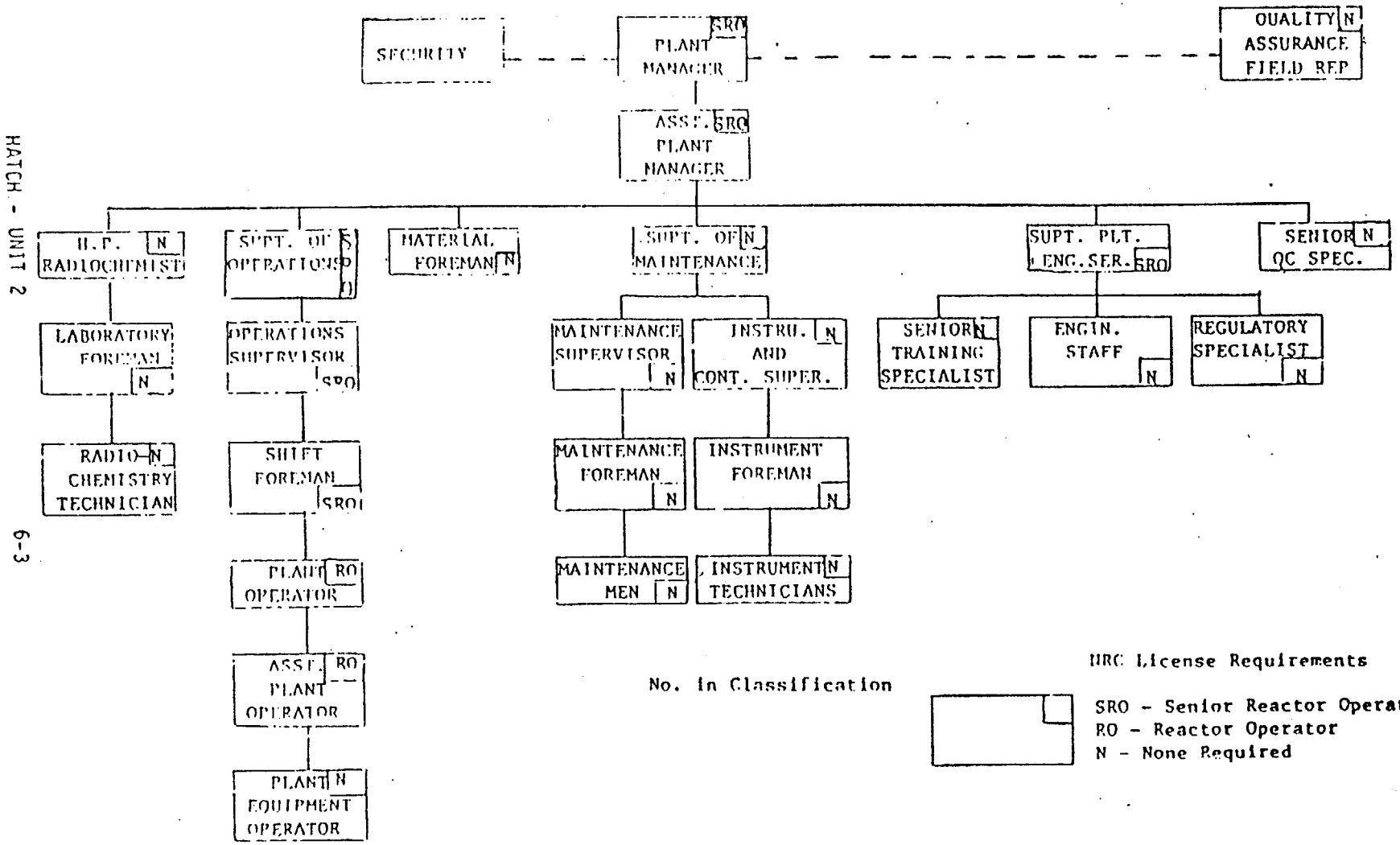

Figure 6.2.2-1

UNIT ORGANIZATION

TABLE 6.2.2-1

MINIMUM SHIFT CREW COMPOSITION #

Condition of Unit 2 - Unit 1 in Reactor Power Operation, Hot Standby or Hot Shutdown Condition

LICENSE CATEGORY	APPLICABLE OPERATIONAL CONDITIONS	
	1, 2, 3	4 & 5
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1	1

Condition of Unit 2 - Unit 1 in Cold Shutdown Condition or Refuel Mode

LICENSE CATEGORY	APPLICABLE OPERATIONAL CONDITIONS	
	1, 2, 3	4 & 5
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1	None

Condition of Unit 2 - No Fuel in Unit 1

LICENSE CATEGORY	APPLICABLE OPERATIONAL CONDITIONS	
	1, 2, 3	4 & 5
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1	None

* Does not include the Licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS.

** Assumes each individual is licensed on both units.

Shift crew composition, including an individual qualified in radiation protection procedures, may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2.2-1.

QUORUM

6.5.2.6 The minimum quorum of the SRB necessary for the performance of the SRB review and audit functions of these Technical Specifications shall consist of the Chairman or Vice Chairman and at least 3 SRB members. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7 The SRB shall review:

a. The safety evaluations for (1) changes to procedures, equipment or systems and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.

b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.

c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.

d. Proposed changes to Technical Specifications or this Operating License.

e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.

f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.

g. Events requiring 24 hour written notification to the Commission.

h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.

i. Reports and meetings minutes of the Plant Review Board.

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the SRB. Each inspection or audit shall be performed within the specified time interval with:

1. A maximum allowable extension not to exceed 25% of the inspection or audit interval.

2. A total maximum combined interval time for any 3 consecutive inspection or audit intervals not to exceed 3.25 times the specified inspection or audit interval.

These audits shall encompass:

a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.

b. The performance, training and qualifications of the entire unit staff at least once per 12 months.

c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.

d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.

e. The Emergency Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.

f. The Security Plan and implementing procedures at least once per 12 months shall be performed by individuals who have no direct responsibility for implementation of this plan.

g. Any other area of unit operation considered appropriate by the SRB or the Senior Vice President Power Supply.

h. The Fire Protection Program and implementing procedures at least once per 24 months.

i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.

j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years. During the year in which the inspection or audit occurs, the requirements of 6.5.2.8i can be affected concurrently.

AUTHORITY

6.5.2.9 The SRB shall report to and advise the Executive Vice President - Power Supply on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 98 TO FACILITY OPERATING LICENSE NO. DPR-57

AND AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNITS NOS. 1 & 2
DOCKETS NOS. 50-321 AND 50-366

INTRODUCTION

By letters, as noted below, Georgia Power Company (GPC) proposed revisions to the Edwin I. Hatch Nuclear Plant (Hatch) Unit 2 Technical Specifications (TSs) as follows:

1. Letter dated April 5, 1982, proposed to extend the time allowed to restore thermal power scram and control rod block trip setpoints to within allowable values; and

2. Letter dated November 10, 1982, proposed to correct an erroneous reference to a section of the TSs.

By letters, as noted below, GPC proposed revisions to the TSs for both Hatch Unit 1 and Unit 2 as follows:

1. Letter dated June 11, 1982, as corrected by letter dated October 15, 1982, proposed to increase the minimum shift crew composition to conform with the requirements of NUREG-0737, Item I.A.1.3.2; and

2. Letter dated March 31, 1983, proposed to increase the frequency of independent audits of both the emergency and security plans.

Our evaluation of these proposals and our conclusions are discussed below.

EVALUATION

Restoration of Thermal Power Scram and Control Rod Block Trip Setpoints.

By letter dated April 5, 1982, GPC requested a change to the Hatch Unit 2 TSs to extend the time allowed for corrective actions to restore the Average Power Range Monitor (APRM), flow referenced, simulated thermal power scram and control rod block trip setpoints to within allowable values.

Present Technical Specifications require a reduction in thermal power if APRM, flow referenced, simulated thermal power or control rod block setpoints that exceed allowable values are not brought back within these allowable values within two hours. The corrections are made by adjusting the control rod patterns and determining the effects of the adjustments. This is an iterative process which is difficult to complete within two hours. The time limit specified in the NRC Standard Technical Specifications for making this correction is six hours. GPC proposes to extend the current two hour limit to six hours, making it consistent with the Standard Technical Specification limit. We conclude that the proposed change is acceptable.

Correction of Error

By letter dated November 10, 1982, GPC requested a change to the Hatch Unit 2 TSs to correct an error where Section 3.9.8 is incorrectly referenced in Section 3.5.4.b.2 as containing limits on refueling water level, while in fact, the referenced section actually regulates crane travel over the spent fuel pool. The correct reference, Section 3.9.10, "Water Level-Spent Fuel Storage Pool", will be provided by the change.

Minimum Shift Crew Composition

By letters dated June 11, 1982, and October 15, 1982, GPC proposed to modify Table 6.2.2-1 of both the Hatch Unit 1 and Unit 2 TSs to increase the minimum shift composition requirements. GPC has subsequently informed us that it inadvertently listed two instead of three Non-Licensed personnel in the Table for the situation where one reactor is in Condition 1, 2 or 3 and there is no fuel in the other reactor. GPC has orally informed us that this number should be three rather than two, and we have made this correction.

We have reviewed the proposed increases and found that the proposed minimum staffing numbers are consistent with the requirements of 10 CFR 50.54(m) and with the numbers specified by NUREG-0737, Item I.A.1.3.2. On this basis, we conclude that the proposed changes are acceptable.

Frequency of Audit of Emergency and Security Plans

By letter dated March 31, 1983, GPC proposed to modify Section 6.5.2.8 of the TSs for both Hatch Unit 1 and Unit 2 to change the required frequency of independent audits of both the emergency and security plans from "at least once per 24 months" to "at least once per 12 months". These proposed changes make these TSs consistent with the frequency of independent audits of the emergency and security plans as required by 10 CFR 50.54(t) and 10 CFR 73.40(d) and are acceptable.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have

further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 22, 1984

PRINCIPAL CONTRIBUTORS:
George Rivenbark
Margaret Chatterton
Peter Kang