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 Docket No. 50-321 AEOD ASLAB
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Mr. J. T. Beckham, Jr.
 Vice President - Nuclear Generation
 Georgia Power Company
 P. O. Box 4545
 Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendment No. 95 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of a one-time change to the Technical Specifications (TSs) in response to your telecopied request dated February 22, 1983, as confirmed by your application dated March 2, 1983.

On February 22, 1983, you received oral authorization from the NRC for this one-time change in TS 3.7.A.5.b, "Oxygen Concentration". The change permitted an extension of the time interval from 24 hours to 72 hours, before the oxygen concentration in the drywell need be reduced to 4%, after the reactor is placed in the Run Mode. By letter dated February 23, 1983, we confirmed the oral authorization.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

"ORIGINAL SIGNED BY:"

George W. Rivenbark, Sr. Project Manager
 Operating Reactors Branch #4
 Division of Licensing

Enclosures:

1. Amendment No. 95 to DPR-57
2. Safety Evaluation
3. Notice

cc w/enclosures:
 See next page

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*Concur Audit
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DATE	3/7/83	3/14/83	3/14/83	3/14/83	3/15/83		

Hatch 1/2
Georgia Power Company

50-321/366

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 22, 1983

Docket No. 50-321

Mr. J. T. Beckham, Jr.
Vice President - Nuclear Generation
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendment No. 95 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of a one-time change to the Technical Specifications (TSs) in response to your telecopied request dated February 22, 1983, as confirmed by your application dated March 2, 1983.

On February 22, 1983, you received oral authorization from the NRC for this one-time change in TS 3.7.A.5.b, "Oxygen Concentration". The change permitted an extension of the time interval from 24-hours to 72 hours, before the oxygen concentration in the drywell need be reduced to 4%, after the reactor is placed in the Run Mode. By letter dated February 23, 1983, we confirmed the oral authorization.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

George W. Rivenbark, Sr. Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 95 to DPR-57
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95
License No. DPR-57

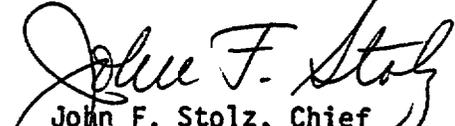
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) telecopied February 22, 1983, as confirmed March 2, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment was effective February 22, 1983, and expired February 25, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: MAR 22 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 95

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove

3.7-9

3.7-10

Insert

3.7-9

3.7-10

3.7.A.5 Oxygen Concentration

- a. After completion of the startup test program and demonstration of plant electrical output, the primary containment atmosphere shall be reduced to less than 4% oxygen with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as stated in Specification 3.7.A.5.b.
- b. Within the 24-hour* period subsequent to placing the reactor in the Run Mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition. De-inerting may commence 24 hours prior to a shutdown.

6. Containment Atmosphere Dilution (CAD)a. Operability Requirements

After completion of the startup test program and demonstration of plant electrical output and thereafter whenever the reactor is in power operation, the post-LOCA containment Atmosphere Dilution (CAD) System must be operable and capable of supplying nitrogen to the primary containment for dilution if required by post-LOCA conditions. If this specification cannot be met, the system must be restored to an operable condition within seven days or the reactor must be taken out of power operation.

b. Seven-Day Nitrogen Supply

After completion of the startup test program and demonstration of plant electric output and thereafter whenever the reactor is in power operation, the CAD System shall contain a minimum of 2000 gallons of liquid nitrogen. If this specification cannot be met, the minimum volume will be restored within seven days or the reactor must be taken out of power operation.

4.7.A.5 Oxygen Concentration

The primary containment oxygen concentration shall be measured and recorded daily in the main control room.

*A 72-hour period is allowed for the startup in progress on February 22, 1983.

6. Containment Atmosphere Dilution (CAD)a. Functional Test

The post-LOCA Containment Atmosphere Dilution (CAD) System shall be functionally tested once per operating cycle.

Seven-Day Nitrogen Supply

The level in the liquid nitrogen storage tanks shall be recorded twice weekly.

3.7.A.6.c. H₂ and O₂ Analyzer

Whenever the reactor is in power operation, there shall be at least one CAD System H₂ and O₂ analyzer serving the primary containment. If one H₂ and O₂ analyzer is inoperable, the reactor may remain in operation for a period not to exceed seven days.

d. Post-LOCA Repressurization Limit

The maximum post-LOCA primary containment repressurization limit allowable using the CAD System shall be 30 psig. Venting via the SGTS to the main stack must be initiated at 30 psig following the initial post-LOCA pressure peak.

7. Drywell-Suppression Chamber Differential Pressure

Differential pressure between the drywell and suppression chamber shall be maintained equal to or greater than 1.5 psid except as specified in (1) and (2) below: If this specification cannot be met, and the differential pressure cannot be restored within the subsequent six (6) hour period, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.

- 1) This differential pressure shall be established within 24 hours* after having placed the Mode Switch in the RUN mode. The differential pressure may be removed within 24 hours prior to achieving a shutdown.
- 2) This differential pressure may be decreased to less than 1.5 psid for a maximum of four hours during required operability testing of the HPCI system pump, the RCIC system pump, and the drywell-pressure suppression chamber vacuum breakers.

4.7.A.6.c. I and O₂ Analyzer

Instrumentation surveillance is listed in Table 4.2-11.

7. Drywell-Suppression Chamber Differential Pressure

The pressure differential between the drywell and suppression chamber shall be recorded once each shift.

*A 72-hour period is allowed for the start-up in progress on February 22, 1983.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 95 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-321

Introduction

By application dated February 22, 1983, as confirmed on March 2, 1983, Georgia Power Company requested changes to the Technical Specifications (TSs) appended to Facility Operating License No. DPR-57. The proposed change would permit an extension in time interval, from 24 hours to 72 hours, before the oxygen concentration in the drywell shall be reduced to 4%, after the reactor is placed in the Run Mode. This change was requested on an expedited basis; oral authorization was issued February 22, 1983, and was confirmed by letter dated February 23, 1983.

Discussion

Technical Specification 3.7.A.5.b requires "within the 24-hour period subsequent to placing the reactor in the RUN mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition". However, during the return to operation of February 22, 1983, it became apparent that this Limiting Condition for Operation (LCO) could not be met. Inability to meet the 24-hour LCO was due to time limits imposed on the use of the 18-inch containment purge and vent valves. This necessitated the use of 2-inch lines (bypass lines for the 18-inch valves) for containment purge and vent functions. These 2-inch lines do not permit timely inerting of the containment. No other lines are available for containment purge and vent functions. Inerting commenced at about 7:00 p.m. (CST) on February 20, 1983, following last personnel entry into primary containment. The licensee, therefore, requested a one-time extension of the 24-hour LCO to 72 hours. The 24-hour LCO interval began at about 9:00 a.m. (CST) on February 22, 1983.

Evaluation

The basis for the present 24-hour requirement on attaining a 4% oxygen concentration following entry into the RUN mode is to minimize the possibility of hydrogen combustion following a loss-of-coolant accident (LOCA). The 24-hour period was established, as stated in the bases for Specification 3.7.A.5, as being sufficient to perform necessary drywell inspections and establish the required oxygen concentration. The control is administrative and supplements the Containment Atmosphere Dilution (CAD) system, which can be used to control oxygen concentrations.

The CAD system provides the basis for assuring containment integrity post LOCA. At the time Unit 1 was licensed, the CAD system was installed to meet the applicable regulations addressing hydrogen evolution.

Because the 24-hour limit is an administrative control, as opposed to providing a safety function, margins of safety, as defined in the Technical Specifications, are not reduced by allowing a longer period of time to complete drywell inerting. The safety margin is provided by the CAD system.

Additionally, due to plant factors, maximum thermal power will not exceed fifty percent (50%) while oxygen concentrations exceed 4%.

Extension of the drywell inerting LCO also necessitates an extension of the LCO for Technical Specification 3.7.A.7.(1), Drywell-Suppression Chamber Differential Pressure, for a like time period. Required differential pressure cannot be established until inerting is completed as the containment vents are open during inerting.

Hatch Unit 1 has previously used the 18" purge and vent lines for inerting purposes while starting up. Due to the NRC's requirements to limit operation of large purge systems with unqualified purge valves to 90 hours a year, Hatch Unit 1 is now attempting to inert by using the 2" purge bypass valve, which does not have a time restriction. This is the first plant startup using only the smaller purge line. Currently containment atmospheric monitors measure the oxygen concentration at approximately 6%.

The licensee has determined that permanent Technical Specification changes are necessary because 24 hours do not allow sufficient time for inerting. This is because the licensee has decided not to use the 18" purge system for the purpose of inerting, thus preserving the 90 hours of use for other situations. We will pursue permanent Technical Specification changes with the licensee in the future. However, for the present plant startup situation, we will address the licensee's proposed one-time only change.

We believe that a basis exists to permit acceptance of the proposed changes. They are as follows:

1. The containment atmosphere is presently at 6% oxygen concentration. This in itself greatly reduces the probability of hydrogen combustion in the event of a postulated LOCA and subsequent hydrogen generation.

2. The maximum thermal power will not exceed 50% during the 72-hour interval. This will limit fission product and decay heat buildup. Thus, the worst case scenario will be less than the design basis event.
3. There is a very low likelihood of an accident involving large amounts of hydrogen generation in the 72-hour interval.
4. The proposed change for Unit 1 is nearly identical to the 72 LCO which has previously been found acceptable for Hatch Unit 2.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: **MAR 22 1983**

The following NRC personnel have contributed to this SER:
D. Pickett and G. Rivenbark.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-321GEORGIA POWER COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 95 to Facility Operating License No. DPR-57, issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, which revised Technical Specifications (TSs) for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1 (the facility) located in Appling County, Georgia.

This amendment was authorized by phone on February 22, 1983. It revised the TSs for a one-time change to permit an extension in the time interval, from 24 hours to 72 hours, before the oxygen concentration in the drywell need be reduced to 4%, after the reactor is placed in the Run Mode. The amendment was issued on an expedited basis to avoid termination of an already initiated startup.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration

and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the request for amendment dated February 22, 1983, as confirmed by application dated March 2, 1983, (2) the Commission's letter to Georgia Power Company dated February 23, 1983, (3) Amendment No. 95 to License No. DPR-57, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 22nd day of March 1983.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing