



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

MAY 26 1981

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Docket No. 50-321

Mr. William Widner
Vice President - Engineering
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

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Dear Mr. Widner:

The Commission has issued the enclosed Amendment No. 85 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated May 4, 1981.

This amendment revises the TSs by (1) adding two valves and two flanges to the list of containment isolation valves; and by (2) changing the setpoint for the Safety Relief Valve tail pipe pressure switches.

Copies of the Safety Evaluation and a related Notice of Issuance are enclosed.

Sincerely,

"ORIGINAL SIGNED BY

JOHN F. STOLTZ"

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

- 1. Amendment No 85
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures:
See next page

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AMENDMENT 1
+
FR NOTICE

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|---------|---------------------|-------------------------|----------------------|-----------------------|---------------------|--|
| OFFICE | ORB#4:DL RINGram | ORB#4:DL MFairtile;c | C-ORB#4:DL JStolz | AD-ORB#4:DL TNovak | OELD [Signature] | |
| SURNAME | | | | | | |
| DATE | 5/20/81 | 5/19/81 | 5/20/81 | 5/20/81 | 5/21/81 | |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
May 26, 1981

Docket No. 50-321

Mr. William Widner
Vice President - Engineering
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Widner:

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Copies of the Safety Evaluation and a related Notice of Issuance are enclosed.

Sincerely,

A handwritten signature in cursive script that reads "John F. Stolz".

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 85
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

Hatch 1/2
Georgia Power Company

50-321/366

cc w/enclosure(s):

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Vice President
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Birmingham, Alabama 35202

cc w/enclosure(s) & incoming dtd.:
5/4/81

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P. O. Box 2625
Southern Services, Inc.
Birmingham, Alabama 35202

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Office of Planning and Budget
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270 Washington Street, S.W.
Atlanta, Georgia 30334

Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513

Mr. L. T. Gucwa
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Georgia Power Company
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P. O. Box 442
Baxley, Georgia 31513

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N.E.
Atlanta, Georgia 30308

Appling County Public Library
301 City Hall Drive
Baxley, Georgia 31513

Mr. R. F. Rodgers
U.S. Nuclear Regulatory Commission
Route 1, P. O. Box 279
Baxley, Georgia 31513



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated May 4, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

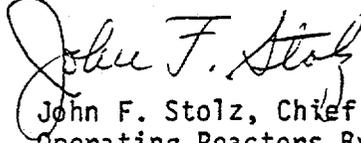
(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 85, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 26, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 85

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

| <u>Remove</u> | <u>Insert</u> |
|---------------|---------------|
| 3.6-9a | 3.6-9a |
| 3.7-26 | 3.7-26 |
| 3.7-27 | 3.7-27 |

4.6.H. Relief/Safety Valves (Continued)

5. Operability of Tail Pipe Pressure Switches

a. Functional Test:

2. At each scheduled outage greater than 72 hours during which entry is made into the primary containment, if not performed within the previous 31 days.

- b. Calibration and verifying the setpoint to be 30 ± 5 psig at least once per 18 months.

.I. Jet Pumps

Whenever the reactor is in the Start & Hot Standby or Run Mode with both recirculating pumps operating, all jet pumps shall be operable. If it is determined that a jet pump is inoperable, an orderly shutdown shall be initiated and the reactor shall be in the Cold Shutdown Condition with 24 hours.

I. Jet Pumps

Whenever both recirculating pumps are operating with the reactor in the Start & Hot Standby or Run Mode, jet pump operability shall be checked daily by verifying that the following conditions do not occur simultaneously.

1. The two recirculation loops have a flow imbalance of 15% or more when the pumps are operated at the same speed.

Table 3.7-4
(Concluded)

Primary Containment Testable Isolation Valves

| <u>Penetration Number</u> | <u>Valve Designation</u> | <u>Notes</u> |
|---------------------------|--------------------------|----------------------|
| X-220 | P33-F006 & F014 | (1) (2) (4) (5) (9) |
| X-220 | T48-F318 & F326 | " |
| X-220 | T48-F332A & F333A | " |
| X-220 | T48-F332B & F333B | " |
| X-220 | T48-F338 & F339 | " |
| X-218A | G51-F002 | (1) (2) (4) (5) (10) |
| X-218A | G51-0001 | (1) (2) (4) (5) (13) |
| X-218A | G51-F016 | (1) (2) (4) (5) (14) |
| X-218A | G51-0004 | (1) (2) (4) (5) (13) |

Notes For Tables 3.7-2 through 3.7-4

- (1) Test duration for all valves and penetrations listed will generally exceed one hour.
- (2) Test pressures are at least 59 psig for all valves and penetrations except MSIV's which are tested at 28 psig.
- (3) MSIV acceptable leakage limit is 11.5 scfh/valve of air.
- (4) The total acceptable leakage for all valves and penetrations other than the MSIV's is 0.6 L_a.
- (5) Local leak tests on all testable isolation valves shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.
- (6) Local leak tests on all testable penetrations shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.
- (7) The personnel air lock shall be tested at intervals not to exceed 6 months.
- (8) The personnel air lock door seals are tested at 10 psig after each opening.
- (9) Identifies isolation valves that are tested by applying pressure between the inboard and outboard isolation valves. Inboard valve is not tested in the direction required for isolation but will have equivalent or more conservative leakage results.
- (10) Identifies isolation valves that are tested by applying pressure between the isolation valve and a manually operated valve such that the isolation valve is tested in the direction required for isolation.
- (11) Identifies isolation valves that are tested by applying pressure between the isolation valves and other system valves. Isolation valves not tested in the direction required for isolation will have equivalent or more conservative results.
- (12) The RHR system remains water filled post-LOCA. Isolation valves are tested with water.
- (13) Identifies blind flange that is tested by applying pressure between the flange and a manually operated valve such that the flange is tested in the direction required for isolation.
- (14) Identifies an isolation valve that is tested by applying pressure between the manually operated isolation valve and a blind flange. The isolation valve is not tested in the direction required for isolation but will have equivalent or more conservative results.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-321

Introduction

By letter dated May 4, 1981, Georgia Power Company (licensee) requested changes to the Technical Specifications (TSs) appended to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The changes involve adding two valves and two flanges to the list of containment isolation valves and deleting surveillance requirements for the Safety Relief Valve tail pipe pressure switches.

The licensee, in the May 4 letter, proposed as an alternate to deletion of the Safety Relief Valve tail pipe pressure switch surveillance program, should it not be acceptable, that we should evaluate a change in setpoint for these switches. After discussions between our staffs, we mutually agreed that NRC should evaluate the setpoint change. This is the evaluation we have prepared, as given below, of the second proposed change.

Evaluation

The licensee, during the current refueling outage of Hatch Unit 1, is adding a Torus Drainage and Purification System. A component of the system penetrates the torus shell, a part of the containment boundary, thus requiring redundant isolation capability per 10 CFR 50, Appendix A, General Design Criterion (GDC) 56. We have reviewed the licensee's submittal and the proposed TS change and conclude the added valves and flanges comply with GDC 56 and are thus acceptable.

The second proposed change results as a consequence of upgrading the pressure switches in the tail pipes on the discharge side of the Pressure Relief Valves. The new switches are being installed during the current outage and are designed per the environmental qualification requirements of electrical equipment of IE Bulletin 79-01B, the NRC Criterion for such components. The previous switches were not qualified. The new

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switches have a pressure setpoint of 30 ± 5 psig and require a TS change as the old switches had a setpoint different from the new switches. We conclude that the change in setpoint is acceptable as it permits use of switches qualified for the intended service.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 26, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-321GEORGIA POWER COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 85 to Facility Operating License No. DPR-57, issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1, (the facility) located in Appling County, Georgia. The amendment is effective as of its date of issuance.

The amendment revises the Technical Specifications by (1) adding two valves and two flanges to the list of containment isolation valves; and by (2) changing the setpoint for the Safety Relief Valve tail pipe pressure switches.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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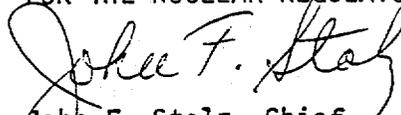
-2-

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated May 4, 1981, (2) Amendment No. 85 to License No. DPR-57, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 26th day of May 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing