

AUG 10 1982

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Mr. J. T. Beckham, Jr.  
 Vice President - Nuclear Generation  
 Georgia Power Company  
 P. O. Box 4545  
 Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendment No. 89 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated December 2, 1981.

This amendment revises the TSs by changing Table 3.7-4, Primary Containment Testable Isolation Valves, to agree with the as-built plant configuration.

Copies of the Safety Evaluation and a related Notice of Issuance are enclosed.

Sincerely,

ORIGINAL SIGNED BY  
 JOHN F. STOLZ  
 John F. Stolz, Chief  
 Operating Reactors Branch #4  
 Division of Licensing

Enclosures:

1. Amendment No. 89 to DPR-57
2. Safety Evaluation
3. Notice

cc w/enclosures:  
 See next page

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*Amend + FR  
 Notice only*

Hatch 1/2  
Georgia Power Company

50-321/366

cc w/enclosure(s):

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cc w/enclosure(s) & incoming dtd.:  
12/2/81

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Room 610  
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Atlanta, Georgia 30334



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

August 10, 1982

Docket No. 50-321

Mr. J. T. Beckham, Jr.  
Vice President - Nuclear Generation  
Georgia Power Company  
P. O. Box 4545  
Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendment No. 89 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated December 2, 1981.

This amendment revises the TSs by changing Table 3.7-4, Primary Containment Testable Isolation Valves, to agree with the as-built plant configuration.

Copies of the Safety Evaluation and a related Notice of Issuance are enclosed.

Sincerely,

A handwritten signature in cursive script that reads "John F. Stolz".

John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 89 to DPR-57
2. Safety Evaluation
3. Notice

cc w/enclosures.  
See next page



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.89  
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated December 2, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

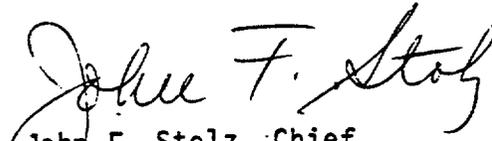
Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 89, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 10, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 89

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove

3.7-26

3.7-27

Insert

3.7-26

3.7-27

Table 3.7-4  
(Concluded)

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>				
		(1)	(2)	(4)	(5)	(9)
X-220	P33-F006 & F014	(1)	(2)	(4)	(5)	(9)
X-220	T48-F318 & F326				"	
X-220	T48-F332A & F333A				"	
X-220	T48-F332B & F333B				"	
X-220	T48-F338 & F339				"	
X-218A	G51-F002	(1)	(2)	(4)	(5)	(10)
X-218A	G51-D001	(1)	(2)	(4)	(5)	(13)
X-218A	G51-F011	(1)	(2)	(4)	(5)	(10)
X-218A	G51-F012	(1)	(2)	(4)	(5)	(10)

Notes For Tables 3.7-2 through 3.7-4

- (1) Test duration for all valves and penetrations listed will generally exceed one hour.
- (2) Test pressures are at least 59 psig for all valves and penetrations except MSIV's which are tested at 28 psig.
- (3) MSIV acceptable leakage limit is 11.5 scfh/valve of air.
- (4) The total acceptable leakage for all valves and penetrations other than the MSIV's is 0.6 L<sub>a</sub>.
- (5) Local leak tests on all testable isolation valves shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.
- (6) Local leak tests on all testable penetrations shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.
- (7) The personnel air lock shall be tested at intervals not to exceed 6 months.
- (8) The personnel air lock door seals are tested at 10 psig after each opening.
- (9) Identifies isolation valves that are tested by applying pressure between the inboard and outboard isolation valves. Inboard valve is not tested in the direction required for isolation but will have equivalent or more conservative leakage results.
- (10) Identifies isolation valves that are tested by applying pressure between the isolation valve and a manually operated valve such that the isolation valve is tested in the direction required for isolation.
- (11) Identifies isolation valves that are tested by applying pressure between the isolation valves and other system valves. Isolation valves not tested in the direction required for isolation will have equivalent or more conservative results.
- (12) The RHR system remains water filled post-LOCA. Isolation valves are tested with water.
- (13) Identifies blind flange that is tested by applying pressure between the flange and a manually operated valve such that the flange is tested in the direction required for isolation.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 89 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1  
DOCKET NO. 50-321

Introduction

By letter dated December 2, 1981, Georgia Power Company (GPC or the licensee) applied for a change to the Technical Specifications (TSs) appended to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The proposed change would update the listing of Primary Containment Testable Isolation Valves.

Background

GPC is installing a Torus Water Drain and Clean-Up System on Hatch Unit 1. During an early 1981 refueling outage, a portion of the system was installed, and at the same time the TSs were amended to include a manual valve (G51-F016) and a blind flange (G51-D004) as the isolation valves for the Clean-Up System. Late in 1981 during a subsequent outage, two air-operated valves were installed between the manual valve and the flange; however the flange was removed as the two new valves were considered as an equivalent.

Evaluation

We agree that the two air-operated valves are equivalent to the flange that has been removed. These two valves also replace the manual valve G51-F016 as a testable isolation valve; as a consequence G51-F016 has been deleted from TS Table 3.7-4. The licensee has verified that the two new valves are closed and have not been connected to the air supply so that the valves cannot be inadvertently opened, remotely. The valves have also been "tagged-out", an administrative precaution, that will prevent the valves from being manually opened until this proposed TS change is approved. Based on the above, we conclude that the substitution of the two valves for the blind flange and the designation of the two valves in place of G51-F016 as testable isolation valves, is acceptable.

### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 10, 1982

This Safety Evaluation was prepared by M. B. Fairtile.

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY, ET AL.

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 89 to Facility Operating License No. DPR-57, issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, which revised Technical Specifications (TSs) for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1 (the facility) located in Appling County, Georgia. The amendment is effective as of the date of issuance.

The amendment revises the TSs by changing Table 3.7-4, Primary Containment Testable Isolation Valves, to agree with the as-built plant configuration.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

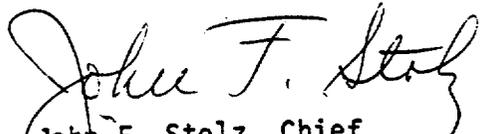
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration

and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated December 2, 1981, (2) Amendment No. 89 to License No. DPR-57, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 10th day of August 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing