August 27, 1982

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Docket No. 50-321

Mr. J. T. Beckham, Jr.
Vice President - Nuclear Generation
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendment No. 90 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of a one-time change to the Technical Specifications (TSs) in response to your telecopied request dated February 8, 1982, as confirmed by your application dated April 5, 1982.

On February 8, 1982, you received oral authorization from the NRC for this one-time change in TS 3.7.A.5.b, "Oxygen Concentration." The change permitted an extension of the time interval from 24 hours to 72 hours, before the oxygen concentration in the drywell need be reduced to 4%, after the reactor is placed in the Run Mode. By letter dated February 10, 1982, we confirmed the oral authorization.

T.S. 3.7.A.5.b provides for administrative control over the oxygen (O_2) content of the drywell during reactor operation. The containment atmosphere dilution (CAD) system is the safety system required to protect against excessive hydrogen flammability in the drywell atmosphere in the event of an accident. The CAD system remained operable throughout the 72-hour interval that the drywell O_2 concentration was above 4%. In addition, the O_2 concentration had been reduced to 7%, less than one-half of normal air oxygen content, prior to the start of the subject 48-hour extension; the reactor never exceeded 50% power during the extension; and the probability of an accident during the 72-hour interval was acceptably low. As an extension in drywell inerting necessitates an extension of time in the interval needed to implement differential pressure between the drywell and suppression chamber as required by Section 3.7.A.7(1) of the Technical Specifications, we likewise extended the time interval to establish the differential pressure to 3:00 p.m. EST, February 10, 1982. The differential pressure cannot be established until inerting is completed as the containment vents are open during inerting. Based on the above, we conclude that the one-time extension of 48 hours to operate the drywell with O_2 concentration above 4% was acceptable.

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We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

A copy of a related Notice of Issuance is also enclosed.

Sincerely,

"ORIGINAL SIGNED BY
JOHN F. STOLZ"
John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:
1. Amendment No. 90 to DPR-57
2. Notice

cc w/enclosures:
See next page

Others agreed + FRN only

OFFICE	ORB#4:DL	ORB#4:DL	C-ORB#4:DL	AD:OR:DL	OELD		
SURNAME	RIngram	MFairtile:cf	JStolz	GLainas			
DATE	8/19/82	8/19/82	8/19/82	8/20/82	8/24/82		

cc w/enclosure(s):

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Chairman
Appling County Commissioners
County Courthouse
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Mr. R. F. Rogers
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Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
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cc w/enclosure(s) & incoming dtd.:
2/8/82, 4/5/82

Charles H. Badger
Office of Planning and Budget
Room 610
270 Washington Street, S.W.
Atlanta, Georgia 30334

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

August 27, 1982

DISTRIBUTION
Docket File
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RIngram

Docket No. 50-321

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).

☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.

☐ Notice of Availability of Applicant's Environmental Report.

☐ Notice of Proposed Issuance of Amendment to Facility Operating License.

☐ Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.

☐ Notice of Availability of NRC Draft/Final Environmental Statement.

☐ Notice of Limited Work Authorization.

☐ Notice of Availability of Safety Evaluation Report.

☐ Notice of Issuance of Construction Permit(s).

☒ Notice of Issuance of Facility Operating License(s) or Amendment(s).

☒ Other: Amendment No. 90.
Referenced documents have been provided PDR.

Division of Licensing, ORB#4
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE→	ORB#4:DL					
SURNAME→	RIngram:cf					
DATE→	8/30/82					

NRC FORM 102 7—79

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al., (the licensee) telecopied February 8, 1982, as confirmed April 5, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 90, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment was effective February 8, 1982 and expired February 10, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 27, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove	Insert
3.7-9	3.7-9
3.7-10	3.7-10

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.7.A.5 Oxygen Concentration	4.7.A.5 Oxygen Concentration
a. After completion of the startup test program and demonstration of plant electrical output, the primary containment atmosphere shall be reduced to less than 4% oxygen with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as stated in Specification 3.7.A.5.b.	The primary containment oxygen concentration shall be measured and recorded daily in the main control room.
b. Within the 24-hour* period subsequent to placing the reactor in the Run Mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition. Deinerting may commence 24 hours prior to a shutdown.	*A 72-hour period is allowed for the startup in progress on February 7, 1982.
6. Containment Atmosphere Dilution (CAD)	6. Containment Atmosphere Dilution (CAD)
a. Operability Requirements	a. Functional Test
After completion of the startup test program and demonstration of plant electrical output and thereafter whenever the reactor is in power operation, the post-LOCA containment Atmosphere Dilution (CAD) System must be operable and capable of supplying nitrogen to the primary containment for dilution if required by post-LOCA conditions. If this specification cannot be met, the system must be restored to an operable condition within seven days or the reactor must be taken out of power operation.	The post-LOCA Containment Atmosphere Dilution (CAD) System shall be functionally tested once per operating cycle.
b. Seven-Day Nitrogen Supply	Seven-Day Nitrogen Supply
After completion of the startup test program and demonstration of plant electric output and thereafter whenever the reactor is in power operation, the CAD System shall contain a minimum of 2000 gallons of liquid nitrogen. If this specification cannot be met, the minimum volume will be restored within seven days or the reactor must be taken out of power operation.	The level in the liquid nitrogen storage tanks shall be recorded twice weekly.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.7.A.6.c. H_2 and O_2 Analyzer	4.7.A.6.c. H_2 and O_2 Analyzer
Whenever the reactor is in power operation, there shall be at least one CAD System H_2 and O_2 analyzer serving the primary containment. If one H_2 and O_2 analyzer is inoperable, the reactor may remain in operation for a period not to exceed seven days.	Instrumentation surveillance is listed in Table 4.2-11.
d. Post-LOCA Repressurization Limit	
The maximum post-LOCA primary containment repressurization limit allowable using the CAD System shall be 30 psig. Venting via the SGTS to the main stack must be initiated at 30 psig following the initial post-LOCA pressure peak.	
7. Drywell-Suppression Chamber Differential Pressure	7. Drywell-Suppression Chamber Differential Pressure
Differential pressure between the drywell and suppression chamber shall be maintained equal to or greater than 1.5 psid except as specified in (1) and (2) below: If this specification cannot be met, and the differential pressure cannot be restored within the subsequent six (6) hour period, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.	The pressure differential between the drywell and suppression chamber shall be recorded once each shift.
1) This differential pressure shall be established within 24 hours* after having placed the Mode Switch in the RUN mode. The differential pressure may be removed within 24 hours prior to achieving a shutdown.	*A 72-hour period is allowed for the start-up in progress on February 7, 1982.
2) This differential pressure may be decreased to less than 1.5 psid for a maximum of four hours during required operability testing of the HPCI system pump, the RCIC system pump, and the drywell-pressure suppression chamber vacuum breakers.	

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY, ET AL.

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 90 to Facility Operating License No. DPR-57, issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, which revised Technical Specifications (TSs) for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1 (the facility) located in Appling County, Georgia.

This amendment was authorized by phone on February 8, 1982. It revised the TSs for a one-time change to permit an extension in the time interval, from 24 hours to 72 hours, before the oxygen concentration in the drywell need be reduced to 4%, after the reactor is placed in the Run Mode. The amendment was issued on an expedited basis to avoid termination of an already initiated startup.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the request for amendment dated February 8, 1982, as confirmed by application dated April 5, 1982, (2) the Commission's letters to the licensee dated February 10, 1982 and August 27, 1982, and (3) Amendment No. 90 to License No. DPR-57. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 27th day of August 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing