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 Mr. J. T. Beckham, Jr. SECY w/NRC 102
 Vice President - Nuclear Generation LSchneider
 Georgia Power Company LJHarmon-2
 P. O. Box 4545 DPickett
 Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued the enclosed Amendment No. 91 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 6, 1982. We have made certain changes to your submittal. These changes were discussed with and agreed to by your staff.

This amendment provides a temporary revision to the secondary containment TSs during the time period of the 1982 refueling outage in order to permit conducting certain major modification work concurrent with refueling activities.

Copies of the Safety Evaluation and a related Notice of Issuance are enclosed.

Sincerely,
 *ORIGINAL SIGNED BY
 JOHN F. STOLZ*

John F. Stolz, Chief
 Operating Reactors Branch #4
 Division of Licensing

Enclosures:

1. Amendment No. 91 to DPR-57
2. Safety Evaluation
3. Notice

cc w/enclosures:
 See next page

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Hatch 1/2
Georgia Power Company

50-321/366

cc w/enclosure(s):

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated October 6, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

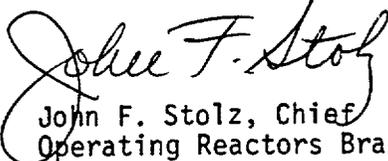
(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 91, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 22, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Remove

3.7-12

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3.7-13

5.0-1

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Insert

3.7-12

3.7-12a

3.7-13

5.0-1

5.0-1a

C. Secondary Containment*1. Secondary Containment Integrity

- a. Integrity of the secondary containment shall be maintained during all modes of Unit 1 plant operation except when all of the following conditions are met:
- (1) The reactor is subcritical and Specification 3.3.A is met.
 - (2) The reactor water temperature is below 212°F and the reactor coolant system is vented.
 - (3) No activity is being performed which can reduce the shutdown margin below that stated in Specification 3.3.A.
 - (4) The fuel cask or irradiated fuel is not being moved in the reactor building.
 - (5) All hatches between Unit 1 secondary containment and Unit 2 secondary containment are closed and sealed.
 - (6) At least one door in each access path between Unit 1 secondary containment and Unit 2 secondary containment is closed.
- b. Integrity of the Unit 1 secondary containment shall be maintained during all modes of Unit 2 plant operations except Operational Condition 4 as defined in the Unit 2 Technical Specifications.

C. Secondary Containment1. Surveillance While Integrity Maintained

Secondary containment surveillance shall be performed as indicated below:

- a. A preoperational secondary containment capability test shall be conducted after isolating the secondary containment and placing the standby gas treatment system filter trains in operation. Such tests shall demonstrate the capability to maintain a minimum 1/4-inch of water vacuum under calm wind (<5 mph) conditions with each filter train flow rate not more than 4000 cfm.
- b. Secondary containment capability to maintain a minimum 1/4-inch of water vacuum under calm wind (<5 mph) conditions with each filter train flow rate not more than 4000 cfm shall be demonstrated at each refueling outage, prior to refueling.

*For secondary containment during 1982 refueling outage, see page 3.7-12a.

2. Secondary Containment Integrity During 1982 Refueling Outage Only

- a. Unit 1 secondary containment below the refueling floor is not required provided all of the following conditions are met:
- (1) The reactor is subcritical and Specification 3.3.A is met.
 - (2) The reactor water temperature is below 212°F and the reactor coolant system is vented.
 - (3) All hatches between Unit 1 secondary containment and Unit 2 secondary containment are closed and sealed.
 - (4) At least one door in each access path between Unit 1 secondary containment and Unit 2 secondary containment is closed.
 - (5) All hatches separating Unit 1 secondary containment above the refueling floor from Unit 1 secondary containment below the refueling floor are closed and sealed.
 - (6) At least one door in each access path separating Unit 1 secondary containment above the refueling floor from Unit 1 secondary containment below the refueling floor is closed.
- b. Integrity of the Unit 1 secondary containment above the refueling floor shall be maintained during all modes of Unit 2 plant operations except Operational Condition 4 as defined in the Unit 2 Technical Specifications.
- c. Refueling operations may continue in Unit 1 secondary containment (above the refueling floor) provided all conditions in Specification 3.7.C.2.a are met.

2. Surveillance While Integrity Maintained

Secondary containment surveillance shall be performed as indicated below:

- a. A preoperational secondary containment capability test shall be performed after isolating the Unit 1 secondary containment above the refueling floor and placing the standby gas treatment system filter trains in operation. Such tests shall demonstrate the capability to maintain a minimum 1/4-inch of water vacuum under calm wind (<5 mph) conditions with each filter train flow rate not more than 4000 cfm.
- b. If secondary containment integrity should be required as stated in Specification 3.7.C.1, perform surveillance as stated in Specification 4.7.C.1.a. If secondary containment is subsequently required as stated in Specification 3.7.C.2, perform surveillance as stated in Specification 4.7.C.2.a.

3.7.C.3 Violation of Secondary Containment Integrity

If Specification 3.7.C.1 cannot be met, procedures shall be initiated to establish conditions listed in Specification 3.7.C.1.a through 3.7.C.1.d.

D. Primary Containment Isolation Valves

1. Valves Required to be Operable

During reactor power operation, all primary containment isolation valves listed in Table 3.7-1, and all reactor coolant system instrument line excess flow check valves shall be operable except as stated in Specification 3.7.D.2.

4.7.C.3 Surveillance After Integrity Violated

After a secondary containment violation is determined the standby gas treatment system will be operated immediately after the affected zones are isolated from the remainder of the secondary containment. The ability to maintain the remainder of the secondary containment at 1/4-inch of water vacuum pressure under calm (≤ 5 mph) wind conditions shall be confirmed.

D. Primary Containment Isolation Valves

1. Surveillance of Operable Valves

Surveillance of the primary containment isolation valves shall be performed as follows:

- a. At least once per operating cycle the operable isolation valves that are power operated and automatically initiated shall be tested for simulated automatic initiation and the closure times specified in Table 3.7-1.

5.0 MAJOR DESIGN FEATURES

A. Site

Edwin I. Hatch Nuclear Plant Unit No. 1 is located on a site of about 2244 acres, which is owned by Georgia Power Company, on the south side of the Altamaha River in Appling County near Baxley, Georgia. The Universal Transverse Mercator Coordinates of the center of the reactor building are: Zone 17R LF 372,935.2m E and 3,533,765.2m N.

B. Reactor Core

1. Fuel Assemblies

The core shall consist of not more than 560 fuel assemblies of the licensed combination of 7x7 bundles which contain 49 fuel rods and 8x8 fuel bundles which contain 62 or 63 fuel rods each.

2. Control Rods

The reactor shall contain 137 cruciform-shaped control rods. The control material shall be boron carbide powder (B_4C) compacted to approximately 70% of its theoretical density.

C. Reactor Vessel

The reactor vessel is described in Table 4.2-2 of the FSAR. The applicable design specifications shall be as listed in Table 4.2-1 of the FSAR.

D. Containment

1. Primary Containment

The principal design parameters and characteristics of the primary containment shall be as given in Table 5.2-1 of the FSAR.

2. Secondary Containment* (See Page 5.0-1a)

The secondary containment shall be as described in Section 5.3.3.1 of the FSAR and the applicable codes shall be as given in Section 12.4.4 of the FSAR.

3. Primary Containment Penetrations

Penetrations to the primary containment and piping passing through such penetrations shall be designed in accordance with standards set forth in Section 5.2.3.4 of the FSAR.

E. Fuel Storage

1. Spent Fuel

All arrangements of fuel in the spent fuel storage racks shall be maintained in a subcritical configuration having a k_{eff} not greater than 0.95.

2. New Fuel

The new fuel storage vault shall be such that the k_{eff} dry shall not be greater than 0.90 and the k_{eff} flooded shall not be greater than 0.95.

* 2. Secondary Containment

During the refueling outage beginning October 9, 1982 the Unit 1 secondary containment shall consist of the main stack, the Standby Gas Treatment System, and the portion of the reactor building above the common Unit 1 and 2 refueling floor provided the conditions of Section 3.7.C.2 are met.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1
DOCKET NO. 50-321

Introduction

By letter dated October 6, 1982, Georgia Power Company (GPC or the licensee) applied for a change to the Technical Specifications (TSs) appended to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The proposed change would provide for a temporary revision to secondary containment TSs during the time period of the 1982 refueling outage.

Background

During the current refueling outage for Cycle 6 operation, the licensee plans to install a substantial portion of the Mark I containment Long-Term Program modifications. This will require the transfer of large amounts of material and manpower through a "railroad" airlock door in the reactor building. Under current TSs the door cannot be opened concurrent with fuel handling activities since TSs require that secondary containment integrity be maintained whenever fuel handling activities take place in order to mitigate the consequences of a fuel handling accident.

In order to efficiently conduct the outage and minimize its length, the licensee has proposed a temporary TS change to permit refueling activities to proceed concurrent with the opening of the reactor building railroad airlock door. This would be accomplished by modifying the current TSs pertaining to secondary containment.

The TSs currently define secondary containment to be the reactor building, the standby gas treatment system (SGTS), and the main stack. The modification would consist of a temporary revision to the secondary containment boundary such that secondary containment would then consist of the SGTS, the main stack, and that portion of the reactor building above the refueling floor. Since only that portion of the reactor building above the refueling floor is necessary to conduct the refueling, all hatches and openings between the refueling floor and the rest of the reactor building would be closed and sealed. Periodic surveillance of the seals used to establish the temporary boundary would be conducted. Access to the floor would be by means of airlocks. In addition, the licensee would

realign the SGTS in order to demonstrate and maintain the integrity of the revised secondary containment. In addition, the SGTS pressure sensor which compares the reactor building atmospheric pressure with outside atmospheric pressure will be isolated during this period to avoid automatic starts of the SGTS; the remaining accident sensing instrumentation is unaffected by this change. Thus, the functional capability of the secondary containment to mitigate the consequences of a fuel handling accident would not be affected.

This change would result in the reactor building below the refueling floor being sealed off from the refueling floor and thus independent of the secondary containment requirements. Therefore, it would permit opening of the railroad airlock doors below the refueling floor level in the reactor building concurrent with fuel handling activities.

Evaluation

We have reviewed the licensee's submittal dated October 6, 1982, which proposes to modify the TSs pertaining to secondary containment. We have also reviewed the appropriate analyses for Hatch Unit 1 pertaining to fuel handling accidents and functional capabilities of the secondary containment. Specifically, we have reviewed Section 14.4.4, Refueling Accidents, of the Hatch Unit 1 Final Safety Analysis Report (FSAR) Update; Section 15.3, Design Basis Accidents, of our Safety Evaluation Report (SER) for Hatch Unit 1; and Regulatory Guide 1.25, Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors, in order to evaluate the impact of the licensee's proposed change on analyzed fuel handling accidents. We have also reviewed FSAR Section 5.3, Secondary Containment, and SER Section 6.2.1.2, Secondary Containment, in order to evaluate the impact of the licensee's proposed change on the functional capability of secondary containment.

We have determined, based on this review, that the proposed temporary modification to the secondary containment boundary does not alter nor negate any assumptions or conditions used in the fuel handling accident analyses. Further, we have determined that there is no degradation of the functional capability of the modified secondary containment to mitigate the consequences of a fuel handling accident. We have also determined that the licensee's proposed surveillance will adequately demonstrate and maintain the integrity of the modified secondary containment through appropriate tests and surveillance procedures. We, therefore, conclude that the licensee's proposed modification and supporting changes to the TSs are acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment

involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 22, 1982

The following NRC personnel have contributed to this Safety Evaluation:
J. Hegner.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-321GEORGIA POWER COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 91 to Facility Operating License No. DPR-57, issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, which revised Technical Specifications (TSs) for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1 (the facility) located in Appling County, Georgia. The amendment is effective as of the date of issuance.

The amendment temporarily revises the secondary containment TSs during the time period of the 1982 refueling outage in order to improve outage efficiency by permitting major modification activities to proceed concurrent with refueling activities.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

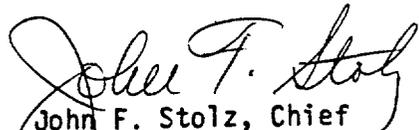
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration

and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 6, 1982, (2) Amendment No. 91 to License No. DPR-57, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 22nd day of October 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing