

MISCELLANEOUS CORRESPONDENCE

FOR THE QUAD CITIES INITIAL EXAMINATION - AUGUST 2001

Exelon Generation
Quad Cities Generating Station
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August 17, 2001

SVP-01-091

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, IL 60532-4351

Quad Cities Generating Station, Units 1 and 2
Facility Operating License Nos. DPR-29 and DPR-30
NRC Docket Nos. 50-254 and 50-265

Subject: Initial License Training NRC Written Examinations

On August 13, 2001, Initial License Training (ILT) NRC written examinations were administered to four (4) Senior Reactor Operator (SRO) candidates and two (2) Reactor Operator (RO) candidates at Quad Cities Nuclear Power Station. Review of the administered examinations by the Station's Training Department has identified comments on three (3) questions which are detailed in the Attachment to this letter. The information presented in the Attachment is consistent with guidance provided in NUREG 1021 (Revision 8), ES-402, Section E. The Station requests review of the comments, and consideration be given to the recommended disposition for these items.

Should you have any questions concerning this letter, please contact Mr. W. J. Beck at (309) 227-2800.

Respectfully,



Timothy J. Tulon
Site Vice President
Quad Cities Nuclear Power Station

Attachment: Examination Comments

cc: NRC Senior Resident Inspector – Quad Cities Generating Station
David Hills – Region III
Dell McNeil – Region III

AUG 20 2001

ATTACHMENT
EXAMINATION COMMENTS

(1 of 3)

Examination Question Number: RO exam, question # 84 / SRO exam, question # 52

Question: The reactor core is being reloaded during a refueling outage. Prior to the start of the reload all SRMs were verified operable, and all Tech Spec required SRM surveillances are current, showing 4 operable SRMs. During the subsequent fuel loading, who has the responsibility for verifying that the Tech Spec SRM operability requirements for CORE ALTERATIONS continue to be met between surveillances?

Answer: a. The Reactor Operator.

Reference: QCFHP 0100-01
KA # 234000 2.2.30

Discussion: This question asks who is responsible for verifying that the Technical Specification SRM operability requirements for Core Alterations continue to be met between surveillances during a core reload. Per OP-AA-101-111 (Roles and Responsibilities of On-Shift Personnel), step 4.2.5, the Unit Supervisor always has the responsibility of ensuring operations are conducted within the bounds of regulatory requirements in accordance with approved procedures. Therefore, the Unit Supervisor has the responsibility to ensure that core alterations are not taking place unless the Technical Specification requirements are being met. Additionally, per QCFHP 0100-01 (Master Refueling Procedure), step E.2, the Nuclear Station Operator (NSO) is responsible for verifying that the Technical Specification requirements are met for the SRM detectors during Core Alterations. Therefore, the NSO is also responsible for the Technical Specification requirements.

Recommendation: Based on the above information, both "Reactor Operator" (answer "a") and "Unit Supervisor" (answer "c") are correct responses and should be considered as such on the exam.

ATTACHMENT
EXAMINATION COMMENTS
(2 of 3)

Examination Question Number: RO exam, question # 1

Question: The reactor was operating at 25% power when all inboard MSIVs inadvertently shut. The MSIV closure scram signal was NOT generated. Which of the following scram signals would be generated next if there were no operator actions taken?

Answer: b. Reactor Vessel Steam Dome Pressure – High.

Reference: Tech Spec Table 3.3.1.1-1
KA # 295007A201

Discussion: This question asks during a spurious Group One isolation from 25% power with a failure of RPS to scram on MSIV closure, which of the following scram signals would be generated next if there were no operator actions taken. The question lists both “reactor pressure” and “APRM neutron flux” as possible answers. Based on UFSAR 5.2.2.2.3, at 100% power in this situation, the APRM flux scram is what is analyzed to scram the reactor. However, it should be noted that based on the analysis, there is less than a one second difference between the pressure scram and the flux scram. While the UFSAR discussion is from 100% power, the amount of time separating the two scram signals is minimal. Additionally, there are several factors to consider in this matter, including pressure instrument response times, APRM response times, the flow control line, and the drive water flow value.

Recommendation: Based on the above information, both “reactor pressure” (answer “b”) and “APRM neutron flux” (answer “d”) should be considered as correct responses. Alternatively, this question could be considered for removal from the exam.

ATTACHMENT
EXAMINATION COMMENTS
(3 of 3)

Examination Question Number: RO exam, question # 58 / SRO exam, question # 26

Question: Following a refueling the Unit 1 Reactor Head Cavity and Dryer/Separator Storage Pit was being drained using QCOP 1900-12, DRAINING THE REACTOR CAVITY AND THE DRYER/SEPARATOR STORAGE PIT. The level in the Reactor Head Cavity and Dryer/Separator Storage Pit had decreased by 20 feet when the supervisor in charge of the drain down ordered the FPC Reject Flow Control Valve AO-1901-58 closed to terminate the drain down. After the AO-1901-58 valve was closed, but before any other valves were repositioned, both Unit 1 fuel pool cooling water pumps tripped. (Assume all closed valves and check valves are leak tight.)

If no operator action is taken which of the following describes the response of fuel pool level?

Answer: a. Water level in the fuel pool will drop until it reaches the level of the overflow to the Skimmer Surge Tanks.

Reference: QCOP 1900-12
Lesson Plan LNF-1900, pages 40 and 41
KA # 233000K302

Discussion: This question pertains to QCOP 1900-12, Draining the Reactor Cavity and the Dryer/Separator Storage Pit. Specifically, the question asks about system response to a pump trip when partially through the restoration lineup after the dryer/separator pit had been drained. This procedure is an infrequent evolution and is not tied to the Emergency Operating Procedures or any immediate operator action in an Abnormal Operating Procedure. In order to answer the question without the procedure as a reference, the candidates would need to have the procedure valve line-up memorized, the procedure steps memorized, and then be able to determine system response based on this partially returned lineup.

Recommendation: Based on the above information, this procedure does not warrant questions where partial valve line-ups have been made without the procedure being given to the candidates as a reference. Therefore, it is recommended that the question be removed from the exam.

April 5, 2001

Mr. Oliver D. Kingsley, President
Exelon Nuclear
Exelon Generation Company, LLC
1400 Opus Place, Suite 500
Downers Grove, IL 60515

Dear Mr. Kingsley:

In a telephone conversation on April 3, 2001, between Mr. Dan Snook and Mr. Dell McNeil, arrangements were made for the administration of licensing examinations at the Quad Cities Nuclear Power Station the week of August 6, 2001. In addition, the NRC will make an examination validation visit to your facility the week of July 16, 2001.

The NRC will prepare the examinations based on the guidelines in Revision 8 of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors." Your staff will be given the opportunity to review the examinations the week of July 16, 2001.

To meet this schedule, it was necessary for your staff to furnish the reference materials identified in Attachment 2 of ES-201 by January 29, 2001. Your staff provided the required reference material on schedule.

In order to conduct the requested written examinations and operating tests, it will be necessary for your staff to provide adequate space and accommodations in accordance with ES-402, and to make the simulation facility available on the dates noted above. In accordance with ES-302, your staff should retain the original simulator performance data (e.g., system pressures, temperatures, and levels) generated during the dynamic operating tests until the examination results are final.

Appendix E of NUREG-1021 contains a number of NRC policies and guidelines that will be in effect while the written examinations and operating tests are being administered.

To permit timely NRC review and evaluation, your staff should submit preliminary reactor operator and senior reactor operator license applications (Office of Management and Budget (OMB) approval number 3150-0090), medical certifications (OMB approval number 3150-0024), and waiver requests (if any) (OMB approval number 3150-0090) at least 30 days before the first examination date. If the applications are not received at least 30 days before the examination date, a postponement may be necessary. Signed applications certifying that all training has been completed should be submitted at least 14 days before the first examination date.

ML010960388

This letter contains information collections that are subject to the *Paperwork Reduction Act of 1995* (44 U.S.C. 3501 et seq.). These information collections were approved by the Office of Management and Budget, approval number 3150-0018, which expires on April 30, 2003.

The public reporting burden for this collection is estimated to average 50 hours per response, including the time for reviewing instructions, gathering and maintaining the data needed, and completing and reviewing the collection of information. Send comments on any aspect of this collection of information, including suggestions for reducing the burden, to the Information and Records Management Branch (T-6 F33), U.S. Nuclear Regulatory Commission, Washington, D.C. 20555-0001, or by Internet electronic mail at BJS1@NRC.GOV; and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0018), Office of Management and Budget, Washington, D.C. 20503.

The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.

Thank you for your cooperation in this matter. Mr. Snook has been advised of the policies and guidelines referenced in this letter. If you have any questions regarding the NRC's examination procedures and guidelines, please contact Dell McNeil at 630-829-9737, or me at 630-829-9733.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,



David E. Hills, Chief
Operations Branch
Division of Reactor Safety

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

See Attached Distribution

DOCUMENT NAME: G:\DRS\qua04_01drs.wpd

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cc: W. Bohlke, Senior Vice President, Nuclear Services
C. Crane, Senior Vice President - Mid-West Regional
J. Cotton, Senior Vice President - Operations Support
J. Benjamin, Vice President - Licensing and Regulatory Affairs
R. Krich, Director - Licensing
H. Stanley, Operations Vice President
J. Skolds, Chief Operating Officer
R. Helfrich, Senior Counsel, Nuclear
DCD - Licensing
T. J. Tulon, Site Vice President
G. Barnes, Quad Cities Station Manager
W. Beck, Regulatory Affairs Manager
W. Leach, Manager - Nuclear
Vice President - Law and Regulatory Affairs
Mid American Energy Company
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Illinois Department of Nuclear Safety
State Liaison Officer, State of Illinois
State Liaison Officer, State of Iowa
Chairman, Illinois Commerce Commission
C. Peterson, Training Department

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December 7, 2000

Mr. Oliver D. Kingsley
President, Nuclear Generation Group
Commonwealth Edison Company
ATTN: Regulatory Services
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

Dear Mr. Kingsley:

In a telephone conversation on November 30, 2000, between Mr. Dan Snook and Mr. Dell McNeil, arrangements were made for the administration of licensing examinations at the Quad Cities Nuclear Power Station the week of August 6, 2001. In addition, the NRC will make an examination validation visit to your facility the week of July 16, 2001.

Mr. Snook has agreed with our request to have NRC examiners prepare the written and operating tests. This request was made to facilitate certification of a new NRC examiner. The NRC will prepare the examinations based on the guidelines of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors," Revision 8, Supplement 1, Addendum 1. Your staff will be given an opportunity to review the examinations during the examination validation week of July 16, 2001.

To meet the above schedule, it will be necessary for your staff to furnish the reference materials identified in Attachment 2 of ES-201 by January 29, 2001. Any delay in receiving the required reference materials, or the submission of inadequate or incomplete materials, may cause the examinations to be rescheduled.

A supplementary letter will be sent to the training department approximately 120 days prior to the examination to reconfirm the examination dates and to reconfirm the number of candidates you have in the training program. If you have any questions concerning this information, please contact Dell McNeil of my staff at (630) 829-9737 or me at (630) 829-9733.

Sincerely,

/RA by Jay Hopkins Acting for/

David E. Hills, Chief
Operations Branch
Division of Reactor Safety

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

See Attached Distribution

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06

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cc: D. Helwig, Senior Vice President, Nuclear Services
C. Crane, Senior Vice President, Nuclear Operations
H. Stanley, Vice President, Nuclear Operations
R. Krich, Vice President, Regulatory Services
DCD - Licensing
J. Dimmette, Jr., Site Vice President
G. Barnes, Quad Cities Station Manager
C. Peterson, Regulatory Affairs Manager
M. Aguilar, Assistant Attorney General
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State Liaison Officer, State of Iowa
Chairman, Illinois Commerce Commission
W. Leech, Manager of Nuclear
MidAmerican Energy Company

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October 20, 2000

Mr. Oliver D. Kingsley
President, Nuclear Generation Group
Commonwealth Edison Company
ATTN: Regulatory Services
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

Dear Mr. Kingsley:

This is to notify you of a change from our letter dated March 13, 2000, in which we informed you that we had tentatively scheduled an initial licensing examination for calendar year 2001 for your operator license applicants at the Quad Cities Nuclear Power Station. In response to a verbal request from your staff and confirmed in a telephone conversation on September 28, 2000, between Mr. D. Snook and Mr. H. Peterson, we have changed the examination dates. The examination is now tentatively scheduled only for the week of August 6, 2001. In addition, the NRC will make an examination validation visit to your facility the week of July 16, 2001.

A supplementary letter will be sent to the training department approximately 120 days prior to the examination outlining examination security expectations, listing the materials required the NRC to conduct the examination, reconfirming the examination dates, and reconfirming the number of candidates you have in the training program. If you have any questions concerning this information, please contact Mary Ann Bies of my staff at 630-829-9711.

Sincerely,

David E. Hills

David E. Hills, Chief
Operations Branch
Division of Reactor Safety

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

See Attached Distribution

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G. Barnes, Quad Cities Station Manager
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March 13, 2000

Mr. Oliver D. Kingsley
President, Nuclear Generation Group
Commonwealth Edison Company
ATTN: Regulatory Services
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

Dear Mr. Kingsley:

In response to R. M. Krich's letter dated October 4, 1999, we have tentatively scheduled an initial licensing examination for your operator license applicants at the Quad Cities Nuclear Power Station during the weeks of July 30, 2001 and August 6, 2001. Validation of the examination will occur at the station during the week of July 9, 2001. In the unlikely event that we are unable to support the examination during the scheduled weeks, we will inform you immediately upon discovery of such conditions and make arrangements to administer the examination at a mutually acceptable date.

As stated in the enclosure to Mr. Krich's letter and confirmed in a telephone conversation with Mr. D. Snook on March 6, 2000, your staff will develop the examination.

Mr. Krich's letter indicated you are training approximately 13 candidates for the examination. Please inform us if the number of candidates declines below 10 as this will impact the examination schedule. Please also inform us at your earliest opportunity if you discover you are unable to support the examination on the scheduled dates.

A supplementary letter will be sent to the training department approximately 120 days prior to the examination outlining examination security expectations, listing the materials required by the NRC to conduct the examination, reconfirming the examination dates, and reconfirming the number of candidates you have in the training program. If you have any questions concerning this information, please contact Mary Ann Bies of my staff at 630-829-9711.

Sincerely,



David E. Hills, Chief
Operations Branch

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

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DCD - Licensing
J. Dimmette, Jr., Site Vice President
G. Barnes, Quad Cities Station Manager
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ComEd

*cc: D. Hill
M. Bielby
H. Peterson*

October 4, 1999

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77
NRC Docket Nos. STN 50-456 and STN 50-457

Byron Station, Units 1 and 2
Facility Operating License Nos. NPF-37 and NPF-66
NRC Docket Nos. STN 50-454 and STN 50-455

Dresden Nuclear Power Station, Units 2 and 3
Facility Operating License Nos. DPR-19 and DPR-25
NRC Docket Nos. 50-237 and 50-249

LaSalle County Station, Units 1 and 2
Facility Operating License Nos. NPF-11 and NPF-18
NRC Docket Nos. STN 50-373 and STN 50-374

Quad Cities Nuclear Power Station, Units 1 and 2
Facility Operating License Nos. DPR-29 and DPR-30
NRC Docket Nos. 50-254 and 50-265

Subject: Response to Administrative Letter 99-03, "Preparation and Scheduling of Operator Licensing Examinations"

Reference: NRC Administrative Letter 99-03, "Preparation and Scheduling of Operator Licensing Examinations," dated August 20, 1999

This letter provides our voluntary response to the request for information contained in the referenced Administrative Letter (AL). The NRC requested a voluntary response to the AL be submitted within 30 days of receipt of the AL.

**OPERATOR LICENSING
EXAMINATION DATA**

Estimated burden per response to comply with this voluntary information collection request: 1 hour. This information collection is used to plan budgets and resources for operator examinations. Send comments regarding burden estimate to the Records Management Branch (T-6 E6) U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001 or by internet email to bjs1@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0131), Office of Management and Budget Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY

Quad Cities Nuclear Power Station, Units 1 and 2
Facility Operating License Nos. DPR-29 and DPR-30
NRC Docket Nos. 50-254 and 50-265

NRC REGION

III

A. PROPOSED EXAMINATION PREPARATION SCHEDULE

PROPOSED NUMBER	CY <u>00</u>	CY <u>01</u>	CY <u>02</u>	CY <u>03</u>
ESTIMATED NUMBER OF LICENSEE-PREPARED EXAMINATIONS	*	1	1	1

B. INITIAL OPERATOR LICENSING EXAMINATIONS

PROPOSED NUMBER	CY <u>00</u>	CY <u>01</u>	CY <u>02</u>	CY <u>03</u>
NUMBER OF REACTOR OPERATORS	*	5	6	5
NUMBER OF SENIOR REACTOR OPERATORS-INSTANT	*	5	3	3
NUMBER OF SENIOR REACTOR OPERATORS-UPGRADE	*	3	3	3
NUMBER OF SENIOR REACTOR OPERATORS-LIMITED	*	0	0	0
PROPOSED DATES				
PRIMARY DATE	*	07/16/01 – 07/27/01	11/04/02 – 11/15/02	11/03/03 – 11/14/03
ALTERNATE DATE	*	07/23/01 – 08/03/01	10/28/02 – 11/08/02	10/27/03 – 11/07/03

C. PROPOSED GENERIC FUNDAMENTALS EXAMINATION (GFE) SCHEDULE

PROPOSED NUMBER	CY <u>00</u>		CY <u>01</u>	
	FIRST	SECOND	FIRST	SECOND
ESTIMATED NUMBER OF CANDIDATES	0	13	0	11

* Already scheduled in NRC letter dated September 8, 1999.