January 22, 1979

Docket Nos. 50-321
and 50-366

Distribution

Docket	OI&E (5)
ORB #3	BJones (8)
NRR Reading	BScharf (10)
Local PDR	STSG
NRC PDR	DEisenhut
HDenton	ACRS (16)
VStello	CMiles
BGrimes	DRoss
TIppolito	RDiggs
PKreutzer	TERA
DVerrelli	JRBuchanan
OELD	

Mr. Charles F. Whitmer
Vice President - Engineering
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Whitmer:

The Commission has issued the enclosed Amendments Nos. 62 and 3 to Facility Operating License Nos. DPR-57 and NPF-5 for the Edwin I. Hatch Nuclear Plant Units Nos. 1 and 2. The amendments revise the surveillance requirements for the Reactor Protection System Power Supplies and is in response to your requests dated January 18 and 19, 1979. Copies of the Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
1. Amendment No. 62 to DPR-57
2. Amendment No. 3 to NPF-5
3. Safety Evaluation
4. Notice

cc w/enclosures:
See next page

cp 1

*SEE PREVIOUS YELLOW FOR CONCURRENCES

7902020347 GP

OFFICE	ORB #3	ORB 3	OELD	PSB	ORB #3	
SURNAME	PKreutzer	*DVerrelli			*GLainas	
DATE	1/22/79	1/22/79	1/ /79	1/22/79	1/ /79	

*DISTRIBUTION:

Docket	DRoss
NRC PDR	RDiggs
Local PDR	TERA
ORB#3 Rdg	JRBuchanan
NRR Rdg	File
HDenton	Xtra Copies
VStello	
BGrimes	
SSheppard	
DVerrelli	
OELD	
OI&E (5)	
BJones (8)	
BScharf (15)	
STSG	
DEisenhut	
ACRS (16)	
CMiles	

Docket Nos. 50-321
and 50-366

Mr. Charles F. Whitmer
Vice President - Engineering
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Whitmer:

The Commission has issued the enclosed Amendments Nos. and to Facility Operating License Nos. DPR-57 and NPF-5 for the Edwin I. Hatch Nuclear Plant Units Nos. 1 and 2. The amendments revise the surveillance requirements for the Reactor Protection System Power Supplies and is in response to your request dated January 18 and 19, 1979. Copies of the Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
1. Amendment No. to DPR-57
2. Amendment No. to NPF-5
3. Safety Evaluation
4. Notice

cc w/enclosure:
see next page

Verbal Concurrence from J. Scinto

OFFICE	ORB#3	ORB#3	OELD	ORB#3	PSB	
SURNAME	PKruetzer	DVerrelli:acr		TIppolito	GLainas	
DATE	1/ /79	1/ 22 /79	1/ 22 /79	1/ 23 /79	1/ 22 /79	

NRC FORM 318 (9-76) NRCM 0240 ★ U.S. GOVERNMENT PRINTING OFFICE: 1978 - 265 - 769

cc:
G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Ruble A. Thomas
Vice President
P. O. Box 2625
Southern Services, Inc.
Birmingham, Alabama 35202

Mr. Harry Majors
Southern Services, Inc.
300 Office Park
Birmingham, Alabama 35202

Charles H. Badger
Office of Planning and Budget
Room 610
270 Washington Street, S. W.
Atlanta, Georgia 30334

Mr. H. B. Lee, Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513

Mr. L. T. Gucwa
Georgia Power Company
Engineering Department
P. O. Box 4545
Atlanta, Georgia 30302

Mr. C. T. Moore
Georgia Power Company
Power Generation Department
P. O. Box 4545
Atlanta, Georgia 30302

Mr. D. P. Shannon
Georgia Power Company
Edwin I. Hatch Plant
P. O. Box 442
Baxley, Georgia 31513

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N. E.
Atlanta, Georgia 30308

Appling County Public Library
Parker Street
Baxley, Georgia 31513

Mr. R. F. Rodgers
U. S. Nuclear Regulatory Commission
P. O. Box 710
Baxley, Georgia 31513

Director, Technical Assessment
Division
Office of Radiation Programs (AW-459)
US EPA
Crystal Mall #2
Arlington, Virginia 20460

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al, (the licensee) dated January 18, as supplemented January 19, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 62, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 22, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 62

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Add page 3.9-6b.

Remove	Insert
3.9-6a	3.9-6a
	3.9-6b

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS

3.9.D Reactor Protection System MG Sets

Applicability:

The limiting conditions for operation apply to the reactor protection system instrumentation.

Objective:

The objective of the limiting condition of operation is to assure that failure of the motor-generator set voltage regulating circuitry will not result in damage to the reactor protection system components with an attendant potential loss of capability to scram the plant.

Specifications:

If in service the RPS instrumentation MG set 1A and/or 1B voltage will be within the range of 108 to 132 VAC.

1. With RPS instrumentation MG set 1A and/or 1B voltage outside the range of 108 to 132 VAC, demonstrate the OPERABILITY of all equipment which could have been subjected to the abnormal voltage for all Class 1E loads connected to the associated bus(es) by performance of a CHANNEL FUNCTIONAL TEST, as required, within 24 hours.

2. With RPS instrumentation MG set 1A and/or 1B inoperable, restore the inoperable MG set(s) to OPERABLE status within 30 minutes or remove the inoperable MG set(s) from service.

4.9.D Reactor Protection System MG Sets

Applicability:

The surveillance requirements apply to the periodic testing requirements of the reactor protection system instrumentation.

Objective:

The objective of the surveillance requirements is to verify that the RPS instrumentation MG sets are within their normal range to preclude sustained over-voltage or under-voltage conditions that might compromise the capability of the RPS from performing its intended safety function.

Specifications:

The specified RPS instrumentation MG sets 1A and 1B shall be determined operable:

a. At least once per 8* hours by verifying:

   1. RPS instrumentation MG sets 1A and 1B voltage to be between 108 and 132 VAC, and

   2. No unexplained change in RPS MG set 1A and/or 1B current and voltage in excess of 5% from its nominal value.

b. At least once per 6 months and prior to resetting the Reactor Protection System trips following a seismic event of Operational Basis Earthquake intensity, by demonstrating the OPERABILITY of RPS instrumentation MG set 1A and 1B over-voltage, under-voltage and under-frequency protective instrumentation by performance of a CHANNEL CALIBRATION including simulated automatic actuation of the protective relays, tripping logic and output circuit breakers and verifying the following setpoints:

   1. Over-voltage $\leq$ 132 VAC,

   2. Under-voltage $\geq$ 108 VAC*, and

   3. Under-frequency $\geq$ 57 Hz.

Order dated August 7, 1978;
Amendment No. 62

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.9.D Reactor Protection System MG Sets	4.9.D Reactor Protection System MG Sets *If the undervoltage trip is <108 VAC increase surveillance frequency to once per hour and the following ACTION statement applies: With RPS instrumentation MG set 1A and/or 1B voltage outside the range of 108-132 vac, immediately transfer the RPS instrumentation affected to the alternate source, if its power is acceptable. Within 1 hour, verify by test that each manual half scram is operable. Within 24 hours demonstrate the Operability of all Class IE loads connected to the affected bus(es) by performance of a Channel Functional Test. If the MG set(s) are not Operable and returned to service within that 24 hours, declare the affected RPS instrumentation inoperative.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al, (the licensee) dated January 18, 1979 as supplemented January 19, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 3, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Appolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 22, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 3

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages are provided for your conviencnce.

Remove	Insert
3/4 8-11	3/4 8-11
3/4 8-12	3/4 8-12

SURVEILLANCE REQUIREMENTS

4.8.2.1.1 The above required A.C. distribution system buses and inverters shall be determined OPERABLE:

a. At least once per 7 days by verifying correct breaker alignment and indicated power availability, and

b. At least once per 31 days by determining that the 250 volt DC/600 volt AC inverters 2R44-S002 and 2R44-S003 are OPERABLE by verifying inverter output voltage of 600 volts ± 5% while supplying their respective buses.

4.8.2.1.2 The above specified RPS instrumentation MG sets 2A and 2B shall be determined OPERABLE:

a. At least once per 8* hours by verifying:

1. RPS instrumentation MG sets 2A and 2B voltage to be between 108 and 132 VAC, and

2. No unexplained change in RPS instrumentation MG set 2A and/or 2B current in excess of 5% from the value observed during the Startup Test Program.

b. At least once per 6 months and prior to resetting the Reactor Protection System trips following a seismic event of Operational Basis Earthquake intensity, by demonstrating the OPERABILITY of RPS instrumentation MG set 2A and 2B over-voltage, under-voltage and under-frequency protective instrumentation by performance of a CHANNEL CALIBRATION including simulated automatic actuation of the protective relays, tripping logic and output circuit breakers and verifying the following setpoints;

1. Over-voltage ≤ 132 VAC,

2. Under- voltage ≥ 108 VAC*, and

3. Under- frequency ≥ 57 Hz.

*If the undervoltage trip is <108 VAC, increase surveillance to once per hour and the following ACTION statement applies to Specification 3.8.2.1 and 3.8.2.2:

With RPS instrumentation MG set 1A and/or 1B voltage outside the range of 108-132 VAC, immediately transfer the RPS instrumentation affected to the alternate source, if its power is acceptable. Within 1 hour, verify by test that each manual half scram is operable. Within 24 hours demonstrate the Operability of all Class IE loads connected to the affected bus(es) by performance of a Channel Functional Test. If the MG set(s) are not Operable and returned to service within that 24 hours, declare the affected RPS instrumentation inoperable.

ELECTRICAL POWER SYSTEMS

A.C. DISTRIBUTION - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.8.2.2 As a minimum, the following A.C. distribution system buses, inverters and motor-generator (MG) sets shall be OPERABLE:

a. Two 4160 volt Essential Buses, 2E, 2F and/or 2G,

b. One 600 volt Essential Bus, 2C or 2D,

c. One 120/208 volt Essential Cabinet, 2A or 2B,

d. One 120/208 volt Instrument Bus, 2A or 2B,

e. A.C. inverters 2R44-S002 and 2R44-S003*, and

f. If in service, Reactor Protection System instrumentation MG sets 2A and 2B.

APPLICABILITY: CONDITIONS 4 and 5.

ACTION:

a. With less than the above required A.C. distribution system buses and inverters OPERABLE, suspend all operations involving CORE ALTERATIONS, irradiated fuel handling, positive reactivity changes or operations that have the potential of draining the reactor vessel. The provisions of Specification 3.0.3 are not applicable.

b. With RPS instrumentation MG set 2A and/or 2B voltage outside the range of 108 to 132 VAC, demonstrate the OPERABILITY of all equipment which could have been subjected to the abnormal voltage for all Class IE loads connected to the associated bus(es) by performance of a CHANNEL FUNCTIONAL TEST or CHANNEL CALIBRATION, as required, within 24 hours.

c. With RPS instrumentation MG set 2A and/or 2B inoperable, restore the inoperable MG set(s) to OPERABLE status within 30 minutes or remove the inoperable MG set(s) from service.

SURVEILLANCE REQUIREMENTS

4.8.2.2 At least the above required A.C. distribution system buses, inverters and MG sets shall be determined OPERABLE per Specifications 4.8.2.1.1 and 4.8.2.1.2.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 62 AND 3 TO FACILITY OPERATING

LICENSES NOS. DPR-57 AND NPF-5

GEORGIA POWER COMPANY

OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT UNITS NOS. 1 AND 2

DOCKETS NOS. 50-321 AND 50-366

I. Introduction

By letters dated January 18 and 19, 1979, Georgia Power Company (licensee) proposed amendments to the Technical Specifications appended to Operating License Nos. DPR-57 and NPF-5 for the Edwin I. Hatch Nuclear Plants Units Nos. 1 and 2. The proposed amendments would revise the surveillance requirements for the Reactor Protection System (RPS) power supply.

II. Background

During the review of the operating license for Hatch Unit No. 2, the staff identified certain specific deficiencies of the power supply for the RPS. Part of the staff's concern was that multiple undetected random failures could occur and could produce power of such poor quality as to preclude automatic or manual scram of the control rods. The concern of multiple random failures was allayed by requiring surveillance of the RPS power supplies which are non-Class IE systems. Included in this surveillance is the requirement for semi-annual verification that the undervoltage trip level is set at $\geq$108 VAC. Subsequently, the licensee's submittals and discussions with the staff indicated that the installed undervoltage relay trip setting operates in the range of 30-60% of operating voltage, i.e., 36-72 VAC. Since the undervoltage protection at this trip level does not satisfy the trip level of 108 VAC called for by the staff conditions, the Technical Specifications require removal of the RPS from service as "inoperable", even though the RPS system itself is operating within its normal range of 108-132 VAC. However, removal of the RPS from service reduces the available sources of power for the reactor protection system. Since RPS circuitry is relied on even in the shutdown condition, the removal of one of the sources of RPS power is undesirable, if adequate protection can be otherwise provided. The licensee's request was submitted to alleviate this situation.

III. Evaluation

The licensee proposes to replace the present undervoltage trip with trips which will function at 108 VAC. The licensee proposed that during that period of time until the present undervoltage trips can be replaced, that surveillance of the power supplies be increased to hourly. If the power supply voltage is outside the range of 108-132 VAC, he will immediately transfer the RPS instrumentation affected to the alternate source, if its power is acceptable. Within 1 hour, he will verify by test that each manual half scram is operable and has not been damaged by sustained undervoltage. Within 24 hours he will demonstrate the Operability of all Class IE loads connected to the affected bus(es) by performance of a Channel Functional Test. If the power supplies are not Operable and returned to service within that 24 hours, he will declare the affected RPS instrumentation inoperable, and follow applicable Technical Specifications.

We have reviewed the licensee's submittals and determined that the proposed revision is acceptable to protect the RPS from sustained undervoltage conditions. The acceptability of increased surveillance is amplified by the design of the Class IE loads connected to the associated bus(es) which include low-voltage alarms in the control room. Since it is very unlikely the sustained undervoltage lasting less than one hour would damage protective systems, the change to manual action rather than an automatic undervoltage trip does not significantly decrease applicable safety margins.

Environmental Considerations

We have determined that the amendments do not involve a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmenal impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusions

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered

by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 22, 1979

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-321 AND 50-366

GEORGIA POWER COMPANY, ET AL

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 62 to Facility Operating License No. DPR-57 and Amendment No. 3 to Facility Operating License No. NPF-5 issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Units Nos. 1 and 2, located in Appling County, Georgia. The amendments are effective as of their date of issuance.

The amendments revise the surveillance requirements for the Reactor Protection System Power Supplies while the licensee replaces an inadequate undervoltage protection trip.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated January 18 and 19, 1979, (2) Amendment No. 62 to License No. DPR-57, (3) Amendment No. 3 to License No. NPF-5, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. A copy of items (2), (3), and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 22nd day of January 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors