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MAR 26 1979

Docket No. 50-321

Mr. Charles F. Whitmer
 Vice President - Engineering
 Georgia Power Company
 P. O. Box 4545
 Atlanta, Georgia 30302

Dear Mr. Whitmer:

The Commission has issued the enclosed Amendment No. ⁶⁵ to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit No. 1. The amendment revises the Administrative Controls Section of the Technical Specifications by converting the entire Section 6 to a Standard Technical Specification format and content. The amendment is in partial response to your applications dated January 24, 1977 and December 21, 1978. Other requested changes to the administrative controls section related to radiological environmental monitoring are not included in this amendment; they will be evaluated as part of our review of the radiological effluent Technical Specifications proposed in your December 21, 1978 letter.

The changes approved by the amendment include: addition to the Unit staff of a Regulatory Specialist; change in titles of select members of the organization; and editorial changes which provide identical wording between Units Nos. 1 and 2 Technical Specifications.

By Amendment No. 50 to DPR-57 we issued on March 2, 1978 interim Technical Specifications to incorporate the existing fire protection system at Hatch 1 into the Limiting Conditions for Operation, Surveillance Requirements and Administrative Controls. As indicated in the transmittal letter, the amendment was not totally responsive to your January 24, 1977 request pending the completion of our review of administrative controls, specifically the addition to the staff of the Regulatory Specialist. The individual's responsibility includes fire protection procedures and training. We completed our review of the Hatch Units 1 and 2 Fire Protection Program and our evaluation including Administrative Controls was set forth in the Safety Evaluation supporting Amendment No. 60 to DPR-57 dated October 4, 1978. Our acceptance of the addition of a Regulatory Specialist with responsibility for fire protection training is specifically set forth in Specification 6.4.2 of the Hatch Unit No. 2 Technical Specifications (NUREG 0395). Accordingly, this change is an administrative action which implements a previously reviewed and approved change. The change

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DATE					

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Mr. Charles F. Whitmer

- 2 -

of titles (e.g., Plant Superintendent to Plant Manager, and Shift Supervisor to Shift Foreman) does not involve changes to qualifications or functions of individuals and is acceptable. None of the editorial changes affect the current requirements related to organization and management, procedures, recordkeeping, review and audit, and reporting requirements. They are pro forma in nature and have no safety or environmental significance.

We have determined that the amendment does not involve a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(f)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Since the amendment applies only to administrative details, it does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of a related Notice of Issuance is also enclosed.

Sincerely,

Original signed by

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 65 to DPR-57
- 2. Notice

cc w/enclosures:
See next page

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DATE	1/26/79	1/12/79	1/28/79	3/26/79	1/15/79

RFB
1/15/79

Mr. Charles F. Whitmer

- 3 -

March 26, 1979

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 65
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Georgia Power Company et al. (the licensee) dated January 24, 1977 and December 21, 1978, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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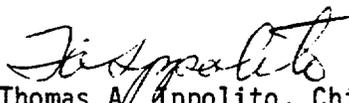
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 65, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance:- March 26, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 65

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

vi
viii
ix
6-1 through 6-26 (to
include Figures 6.2-1
and 6.2.2)

Insert

vi
viii
ix
6-1 through 6-21

<u>Section</u>	<u>Section</u>	<u>Page</u>
	LIMITING CONDITIONS FOR OPERATION	
	SURVEILLANCE REQUIREMENTS	
5.0	MAJOR DESIGN FEATURES	5.0-1
	A. Site	5.0-1
	B. Reactor Core	5.0-1
	C. Reactor Vessel	5.0-1
	D. Containment	5.0-1
	E. Fuel Storage	5.0-1
	F. Seismic Design	5.0-2
6.0	ADMINISTRATIVE CONTROLS	6-1
	6.1 Responsibility	6-1
	6.2 Organization	6-1
	6.3 Unit Staff Qualifications	6-6
	6.4 Training	6-6
	6.5 Review and Audit	6-6
	6.6 Reportable Occurrence Action	6-12
	6.7 Safety Limit Violation	6-12
	6.8 Procedures	6-13
	6.9 Reporting Requirements	6-14
	6.10 Record Retention	6-18
	6.11 Radiation Protection Program	6-20
	6.12 High Radiation Area	6-20

LIST OF TABLES
(Concluded)

<u>Table</u>	<u>Title</u>	<u>Page</u>
4.2-7	Check, Functional Test, and Calibration Minimum Frequency For Neutron Monitoring Instrumentation Which Initiates Control Rod Blocks	3.2-40
4.2-8	Check, Functional Test, and Calibration Minimum Frequency for Radiation Monitoring Systems Which Limit Radioactivity Release	3.2-42
4.2-9	Check and Calibration Minimum Frequency for Instrumentation Which Initiates Recirculation Pump Trip	3.2-45
4.2-10	Check, Functional Test, and Calibration Minimum Frequency for Instrumentation Which Monitors Leakage into the Drywell	3.2-46
4.2-11	Check and Calibration Minimum Frequency for Instrumentation Which Provides Surveillance Information	3.2-48
3.6.1	Safety Related Shock Suppressors (Snubbers)	3.6-10c
4.6-1	In-Service Inspection Program	3.6-11
3.7-1	Primary Containment Isolation Valves	3.7-16
3.7-2	Testable Penetrations with Double O-Ring Seals	3.7-21
3.7-3	Testable Penetrations with Testable Bellows	3.7-22
3.7-4	Primary Containment Testable Isolation Valves	3.7-23
3.13-1	Fire Detectors	3.13-2
3.13-2	Fire Hose Stations	3.13-9
6.2.2-1	Minimum Shift Crew Composition	6-3
6.9-1	Special Reporting Requirements	6-19

LIST OF FIGURES

<u>Figure</u>	<u>Title</u>
1.1-1	Core Thermal Power Safety Limit Versus Core Flow Rate
2.1-1	Reactor Vessel Water Levels
4.1-1	Graphical Aid for the Selection of an Adequate Interval Between Tests
4.2-1	System Unavailability
3.4-1	Sodium Pentaborate Solution Volume Versus Concentration Requirements
3.4-2	Sodium Pentaborate Solution Temperature Versus Concentration Requirements
3.6-1	Change in Charpy V Transition Temperature Versus Neutron Exposure
3.6-2	Minimum Temperature for Inservice Hydrostatic and Leak Tests
3.6-3	Minimum Temperature for Mechanical Heatup or Cooldown Following Nuclear Shutdown
3.6-4	Minimum Temperature for Core Operation (Criticality)
3.11-1	(Sheet 1) Limiting Value for APLHGR (Fuel Type 3)
3.11-1	(Sheet 2) Limiting Value for APLHGR (Fuel Types 1 and 2)
3.11-2	Limiting Value for LHGR
3.11-3	K_f Factor
6.2.1-1	Offsite Organization
6.2.2-1	Unit Organization

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

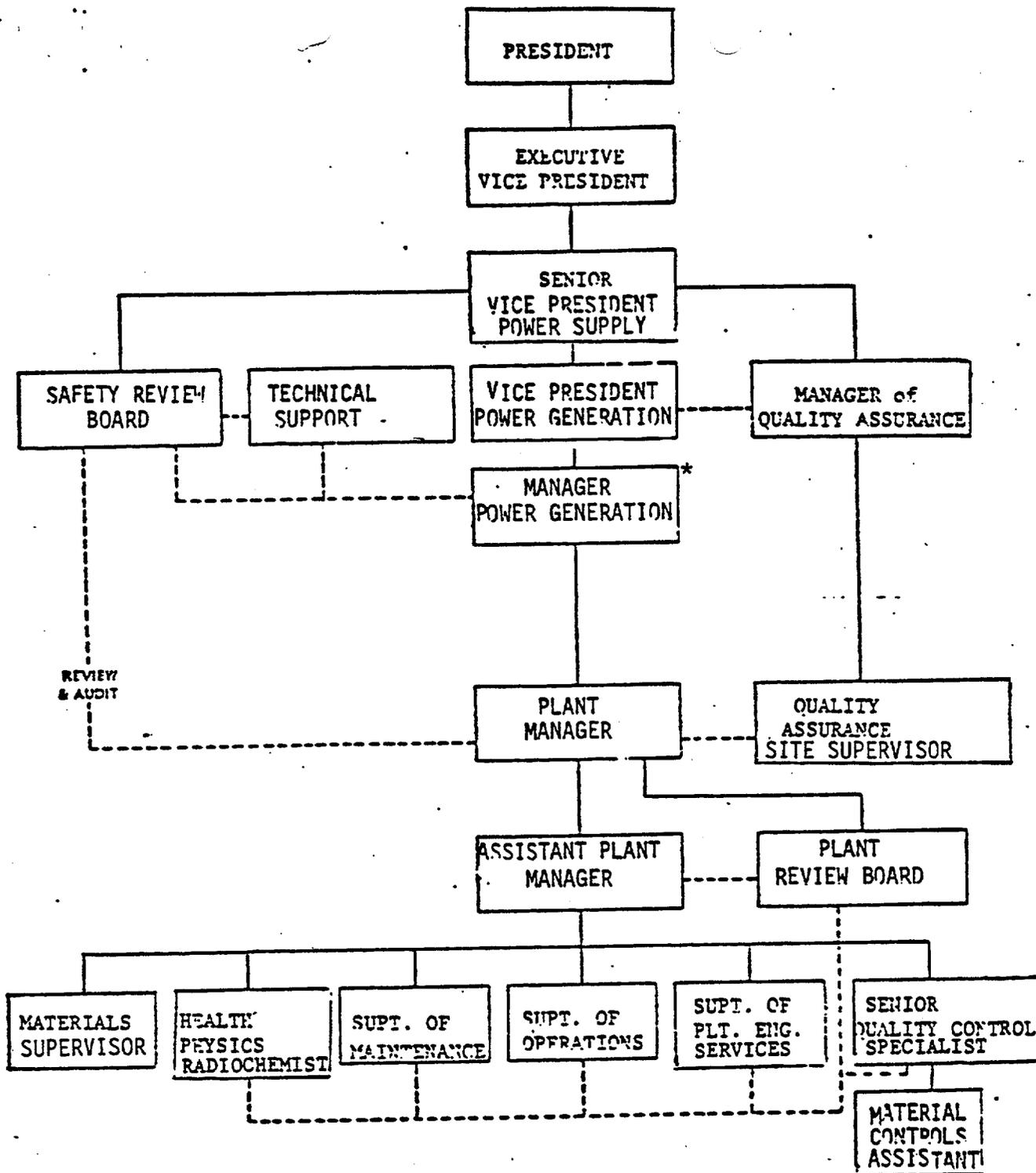
OFFSITE

6.2.1 The offsite organization for unit management and technical support shall be as shown on Figure 6.2.1-1.

UNIT STAFF

6.2.2 The unit organization shall be as shown on Figure 6.2.2-1 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.2-1.
- b. At least one licensed Operator shall be in the control room for each reactor containing fuel.
- c. At least two licensed Operators shall be present in the control room for each reactor in the process of start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified to implement radiation protection procedures shall be on site when fuel is in either reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A Fire Team of at least five members shall be maintained onsite at all times. The Fire Team shall not include the minimum shift crew necessary for safe shutdown of Units 1 and 2 or any personnel required for other essential functions during a fire emergency.



* Corporate responsibility for Fire Protection Program.

— LINES OF RESPONSIBILITY
 - - - LINES OF COMMUNICATION

Figure 6.2.1-1
 OFFSITE ORGANIZATION

TABLE 6.2.2-1

MINIMUM SHIFT CREW COMPOSITION#

Condition of Unit 1 - Unit 2 in Reactor Power Operation,
Hot Standby or Hot Shutdown Condition

LICENSE CATEGORY	APPLICABLE OPERATIONAL CONDITIONS	
	1, 2, 3	4 & 5
SOL**	2	2*
OL**	3	2
Non-Licensed	3	3

Condition of Unit 1 - Unit 2 in Cold Shutdown Condition
or Refuel Mode

LICENSE CATEGORY	APPLICABLE OPERATIONAL CONDITIONS	
	1, 2, 3	4 & 5
SOL**	2	1*
OL**	2	2
Non-Licensed	3	3

Condition of Unit 1 - No Fuel in Unit 2

LICENSE CATEGORY	APPLICABLE OPERATIONAL CONDITIONS	
	1, 2, 3	4 & 5
SOL	1	1*
OL	2	1
Non-Licensed	2	1

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS.

**Assumes each individual is licensed on both units.

#Shift crew composition, including an individual qualified in radiation protection procedures, may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2.2-1.

TABLE 6.2.2-1 (Continued)
MINIMUM SHIFT CREW COMPOSITION

TABLE NOTATION

OPERATIONAL
CONDITION

DEFINITION

1	Run mode
2	Start and Hot Standby Mode
3	Shutdown Mode, Hot Shutdown Condition
4	Shutdown Mode, Cold Shutdown Condition
5	Refuel Mode

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Health Physicist-Radiochemist who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Senior Methods and Training Specialist and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for fire protection shall be maintained under the direction of the Senior Regulatory Specialist and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for fire protection training sessions which shall be held at least once per 92 days.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1 The PRB shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Review Board shall be composed of the:

Chairman:	Plant Manager
Vice Chairman:	Assistant Plant Manager
Member:	Operations Superintendent
Member:	Superintendent Plant Engineer Services
Member:	Maintenance Superintendent
Member:	Senior Quality Control Specialist
Member:	Health Physicist-Radiochemist

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or Vice Chairman and three members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Review Board shall be responsible for:

- a. Review of (1) all procedures required by Specification 6.8 and changes thereto, (2) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager of Power Generation and to the Safety Review Board (SRB).
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of unit operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the SRB.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- i. Review of the Security Plan and implementing procedures and shall submit recommended changes to the SRB.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the SRB.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Recommend in writing to the Plant Manager approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Manager of Power Generation and the Safety Review Board of disagreement between the PRB and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Review Board shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Manager of Power Generation and the Safety Review Board.

6.5.2 SAFETY REVIEW BOARD (SRB)

FUNCTION

6.5.2.1 The SRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,

ADMINISTRATIVE CONTROLS

FUNCTION (Continued)

- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering,
- h. Quality assurance practices.

COMPOSITION

6.5.2.2 The SRB shall be composed of a minimum of seven persons who as a group provide the expertise to review and audit the operation of a nuclear power plant. The Chairman and Vice Chairman and other members shall be appointed by the Company Senior Vice President, Power Supply or such other person as he shall designate. The composition of the SRB shall meet the requirements of ANSI N18.7-1976.

ALTERNATES

6.5.2.3 All alternate representatives shall be appointed in writing by the absent member to serve on a temporary basis; however, no more than two alternates shall participate in SRB activities at any one time and alternate representatives shall not have voting privileges.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRB Chairman or Vice-Chairman to provide expert advice to the SRB.

MEETING FREQUENCY

6.5.2.5 The SRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.2.6 The minimum quorum of the SRB necessary for the performance of the SRB review and audit functions of these Technical Specifications shall consist of the Chairman or Vice Chairman and at least 3 SRB members. No more than a minority of the quorum shall have line responsibility for operation of the unit.

REVIEW

6.5.2.7 The SRB shall review:

- a. The safety evaluations for (1) changes to procedures, equipment or systems and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the Plant Review Board.

ADMINISTRATIVE CONTROLS

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the SRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B" 10 CFR 50, at least once per 24 months.
- e. The Emergency Plan and implementing procedures at least once per 24 months.
- f. The Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of unit operation considered appropriate by the SRB or the Senior Vice President Power Supply.
- h. The Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

AUTHORITY

6.5.2.9 The SRB shall report to and advise the Senior Vice President-Power Supply on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.2.10 Records of SRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRB meeting shall be prepared, approved and forwarded to the Senior Vice President-Power Supply within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Senior Vice President-Power Supply within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President-Power Supply and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PRB and submitted to the SRB and the Manager of Power Generation.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT SHUTDOWN within two hours.
- b. The Safety Limit violation shall be reported to the Commission, the Manager of Power Generation and to the SRB within 24 hours.

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PRB. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the SRB and the Manager of Power Generation within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the PRB and approved by the Plant Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PRB and approved by the Plant Manager within 14 days of implementation.

ADMINISTRATIVE CONTROLS

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

ANNUAL REPORTS^{1/}

6.9.1.4 Annual reports covering the activities of the unit as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

^{1/} A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

ADMINISTRATIVE CONTROLS

ANNUAL REPORTS (Continued)

6.9.1.5 Reports required on an annual basis shall include:

- a. A tabulation on an annual basis of the number of station, utility and other personnel, including contractors, receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,^{2/} e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- b. Any other unit unique reports required on an annual basis.

MONTHLY OPERATING REPORT

6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Management and Program Analysis, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office of Inspection and Enforcement no later than the 15th of each month following the calendar month covered by the report.

REPORTABLE OCCURRENCES

6.9.1.7 The REPORTABLE OCCURRENCES of Specifications 6.9.1.8 and 6.9.1.9 below, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

^{2/} This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

ADMINISTRATIVE CONTROLS

PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP

6.9.1.8 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate no later than the first working day following the event, with a written followup report within 14 days. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed; by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.
- b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
- c. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.
- d. Reactivity anomalies involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation greater than or equal to 1% $\Delta k/k$; a calculated reactivity balance indicating a SHUTDOWN MARGIN less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if subcritical, an unplanned reactivity insertion of more than 0.5% $\Delta k/k$; or occurrence of any unplanned criticality.
- e. Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.
- f. Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

ADMINISTRATIVE CONTROL

PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP (Continued)

- g. Conditions arising from natural or man-made events that, as a direct result of the event, require unit shutdown, operation of safety systems, or other protective measures required by technical specifications.
- h. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.
- i. Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during unit life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

THIRTY DAY WRITTEN REPORTS

6.9.1.9 The types of events listed below shall be the subject of written reports to the Director of the Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- b. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.
- c. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

ADMINISTRATIVE CONTROL

- d. Abnormal degradation of systems other than those specified in 6.9.1.8.c above designed to contain radioactive material resulting from the fission process.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified and for each activity shown in Table 6.9.2-1.

6.10 RECORD RETENTION

In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of unit operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to the procedures required by Specification 6.8.1.
- f. Records of radioactive shipments.
- g. Records of sealed source and fission detector leak tests and results.
- h. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the unit Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.

TABLE 6.9.2-1
SPECIAL REPORTING REQUIREMENTS

	<u>Area</u>	<u>Tech Spec Reference</u>	<u>Submittal Date</u>
a.	Primary Containment Leak Rate Tests (1)	4.7.A	Within 3 months following conduct of test
b.	Secondary Containment Leak Rate Tests (2)	4.7.C	Within 3 months following conduct of test
c.	Primary Coolant Leakage to Drywell	4.6.C	5 years (3)
d.	In-Service Inspection Evaluation	4.6.K	5 years (3)
e.	Reactor Coolant Radioactivity in excess of specified limits	4.6.F	Within 30 days of the occurrence

Notes:

1. Each integrated leak rate test of the primary containment shall be the subject of a summary technical report including results of the local leak rate tests since the last report. The report as described in the 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors," shall include data, analysis, and interpretations of the results which demonstrate compliance in meeting the specified leak rate limits.
2. Each integrated leak rate test of the secondary containment shall be the subject of a summary technical report. This report should include data on the wind speed, wind direction, outside and inside temperatures during the test, concurrent reactor building pressure, and emergency ventilation flow rate. The report shall also include analyses and interpretations of those data which demonstrate compliance with the specified leak rate limits.
3. The report shall be submitted within the period of time listed based on the commercial service date as the starting period.

ADMINISTRATIVE CONTROL

RECORD RETENTION (Continued)

- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient or operational cycles for those unit components identified in Table 5.7.1-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the unit staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PRB and the SRB.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit*. Any individual or group of individuals permitted

*Health Physics personnel, or personnel escorted by Health Physics personnel in accordance with approved emergency procedures, shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

ADMINISTRATIVE CONTROL

to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the facility Health Physicist in the Radiation Work Permit.

6.12.2 The requirements of 6.12.1, above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Foreman on duty and/or the Plant Health Physicist.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-321GEORGIA POWER COMPANY, ET ALNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 65 to Facility Operating License No. DPR-57 issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia, and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1, (the facility) located in Appling County, Georgia. The amendment is effective as of the date of issuance.

The amendment is administrative in nature in that it revises the Administrative Controls section of the Technical Specifications by converting the entire section to a Standard format and content.

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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For further details with respect to this action, see (1) the applications for amendment dated January 24, 1977 and December 21, 1978, (2) Amendment No. 65 to License No. DPR-57, and (3) the Commission's letter dated March 26, 1979 . All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Library, Parker Street, Baxley, Georgia 31513. A copy of item (2) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 26th day of March 1979.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors