

JULY 17 1979

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Docket No. 50-321

Mr. Charles F. Whitmer
Vice President - Engineering
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Whitmer:

REGULATORY DOCKET FILE COPY

The Commission has issued the enclosed Amendment No. 67 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated May 14, 1979.

This amendment revises the Turbine Control Valve Fast Closure setpoint from >1000 psig to >600 psig on low electrohydraulic control oil pressure.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

for
V. Rooney

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 67
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures:
See page 2

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OFFICE	ORB #3	ORB #3	PSB	AD: <i>P.P. W. GARNER</i>	OELD	ORB #3
SURNAME	<i>PKreutzer</i>	DVerrelli:mif		<i>W. GARNER</i>	<i>B. Smith</i>	Ippolito
DATE	7/17/79	6/1/79	7/16/79	7/11/79	7/16/79	7/11/79

Mr. Charles F. Whitmer
Georgia Power Company

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July 17, 1979

cc:

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Washington, D. C. 20036

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Southern Services, Inc.
Birmingham, Alabama 35202

Mr. Harry Majors
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Charles H. Badger
Office of Planning and Budget
Room 610
270 Washington Street, S. W.
Atlanta, Georgia 30334

Mr. H. B. Lee, Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513

Mr. L. T. Gucwa
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Atlanta, Georgia 30302

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Mr. Max Manry
Georgia Power Company
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P. O. Box 442
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U. S. Environmental Protection
Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N. E.
Atlanta, Georgia 30308

Appling County Public Library
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Baxley, Georgia 31513

Mr. R. F. Rodgers
U. S. Nuclear Regulatory Commission
P. O. Box 710
Baxley, Georgia 31513

Director, Technical Assessment
Division
Office of Radiation Programs (AW 459)
US EPA
Crystal Mall #2
Arlington, Virginia 20460



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated May 14, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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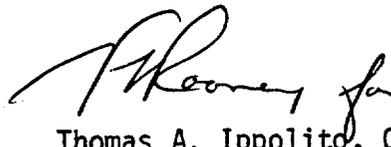
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 67, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 17, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 67

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

Remove

3.1-9

Insert

3.1-9

Notes for Table 4.1-1 (Cont'd)

- d. Calibrations are not required when the systems are not required to be operable or are tripped. However, if calibrations are missed, they shall be performed prior to returning the system to an operable status.
- e. This instrumentation is exempted from the instrument functional test definition. This instrument functional test will consist of injecting a simulated electrical signal into the measurement channels.
- f. Initially once per month or according to Figure 4.1-1 with an interval of not less than one month nor more than three months. The compilation of instrument failure rate data may include data obtained from other BWR's for which the same design instrument operates in an environment similar to that of HNP-1. The failure rate data must be reviewed and approved by the NRC prior to any change in the once-a-month frequency.
- g. The water level in the reactor will be perturbed and the corresponding level indicator changes will be monitored. This perturbation test will be performed every month after completion of the monthly functional test program.
- h. Physical inspection and actuation of these position switches will be performed once per operating cycle.
- i. Standard current source used which provides an instrument channel alignment. Calibration using a radiation source shall be made once per operating cycle.
- j. Measure time interval from EHC pressure switch actuation to RPS relay K14 de-energization.
- k. The electrohydraulic control oil pressure sensors shall be set to trip at >600 psig control oil pressure.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1

DOCKET NO. 50-321

I. INTRODUCTION

By letter dated May 14, 1979, Georgia Power Company (licensee) proposed a change to the Technical Specifications appended to Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit No. 1. The amendment would revise the Turbine Control Valve Fast Closure setpoint from ≥ 1000 psig to ≥ 600 psig on low Electro-Hydraulic Control oil pressure.

II. BACKGROUND

Fast closure of the turbine control valves is initiated to prevent overspeed of the turbine in the event of a load rejection which is greater than the bypass capacity of the facility. Such a fast closure, if otherwise unmitigated by the Reactor Protection System (RPS), would result in a nuclear system pressure increase and a shutdown of the reactor upon reaching either the high-pressure scram setpoint or high-flux scram setpoint. To mitigate the consequences this increase in reactor pressure and accompanying increase in core reactivity due to a reduction in core void fraction, a reactor scram is initiated upon sensing the start of control valve fast closure.

This reactor scram is required by Technical Specifications to insure that thermal limits of the core are not exceeded. The analyses which support the Technical Specification limit is a calculation of the reduction in Minimum Critical Power Ratio from the operating value that would result should a load rejection occur. The assumption used in this analysis (relevant to control valve closure) is that reactor scram is initiated at the start of control valve fast closure.

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It is noted that the nuclear system high-pressure scram in conjunction with the pressure relief system is more than adequate to preclude over pressuring the reactor system even if a turbine control valve closure were to occur. Thus, the reactor scram due to fast closure of the control valves is not required to ensure integrity of the reactor coolant system. Therefore, the evaluation of the acceptability of the licensee's request is limited to ensuring that the core thermal limits are not exceeded.

III. EVALUATION

The licensee proposed to decrease the Control Valve Fast Closure setpoint from > 1000 psig to > 600 psig in order to decrease the probability of inadvertent reactor scrams due to normal EHC oil pressure fluctuations. The elimination of inadvertent scrams would decrease unwarranted thermal cycles on the reactor vessel. The licensee's proposal is identical to that approved for Hatch Unit 2. We have reviewed the licensee's submittal and agree that a reduced setpoint would be acceptable if the conservative assumptions used to ensure that the core thermal limits are not exceeded remain valid, i.e., a reactor scram is initiated upon sensing the start of control valve fast closure.

The EHC system at Hatch is similar to other BWRs. Fast closure of the turbine control valves is initiated when the EHC pressure drops below that pressure required to maintain the disk dump valve closed. At both Hatch 1 and 2, the operating range for opening the disk dump valve is 480-500 psig. A pressure switch senses the pressure of the EHC oil, which causes the disk dump valve to operate. Thus, the proposed trip setting of > 600 psig for the pressure switch is higher than that pressure which would cause the disc dump valve to operate and is, therefore, at least as conservative as those conditions assumed in the analysis.

In the course of staff review, we considered instrumentation accuracy and RPS response times. The current Technical Specifications include the provisions that the RPS response time will not exceed 50 msec and the scram trip signal for turbine control valve fast closure will occur within 30 msec of the start of valve closure i.e., operation of the disk dump valve. Accordingly, instrumentation accuracy and delay times are already accounted for in the specifications and are unaffected by the proposed change. It is noted that the span of adjustment for the pressure switch is 250 psi to 1600 psi.

In view of the above, we find that the proposed change is acceptable.

IV. ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendment does not authorize a change in effluent types or total amounts not an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

V. CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 17, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-321GEORGIA POWER COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 67 to Facility Operating License No. DPR-57, issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1 (the facility) located in Appling County, Georgia. The amendment is effective as of its date of issuance.

The amendment revises the Turbine Control Valve Fast Closure setpoint from ≥ 1000 psig to ≥ 600 psig on low electrohydraulic control oil pressure.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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For further details with respect to this action, see (1) the application for amendment dated May 14, 1979, (2) Amendment No. 67 to License No. DPR-57, and

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(3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 17th day of July 1979.

FOR THE NUCLEAR REGULATORY COMMISSION



Vernon L. Rooney, Acting Chief
Operating Reactors Branch #3
Division of Operating Reactors