

Docket No. 50-321

APRIL 12 1978

Georgia Power Company
Oglethorpe Electric Membership Corporation
Municipal Electric Association of Georgia
City of Dalton, Georgia
ATTN: Mr. Charles F. Whitmer
Vice President - Engineering
Georgia Power Company
Atlanta, Georgia 30302

Gentlemen:

The Commission has issued the enclosed Amendment No. 53 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated November 16, 1977.

The amendment consists of Technical Specification changes to incorporate approved exemptions from certain requirements of 10 CFR Part 50 Appendix J. Appendix J requires that isolation valve leakage tests shall be conducted with air or nitrogen unless the valves are normally pressurized from a seal system. The exemption granted by this amendment permits the testing of the RHR suction line valves, with water, based on our determination that the test condition would be representative of the post-accident environment.

The Commission has further concluded that the granting of the exemption to Appendix J, 10 CFR Part 50, is authorized by law, will not endanger life or property or the common defense and security, and is otherwise in the public interest.

Your letter dated November 16, 1977 included a proposal to revise the reporting requirements for leakage from the RHR suction line valves and certain other valves. We have not yet completed our review of this item.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original signed by
Victor Stello, Jr., Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

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B. Jones
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A/5/78

OFFICE	Enclosures and dcs: See page 2	ORB #3 SSheppard 4/4/78	ORB #3 DVerrelli 4/5/78	OELD B. Jones 4/10/78	ORB #3 GLear 4/11/78	ORB VStello 4/12/78
SURNAME						
DATE						

Distribution

- ✓ Docket
- ORB #3
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- Attorney, OELD
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 12, 1978

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Municipal Electric Association of Georgia
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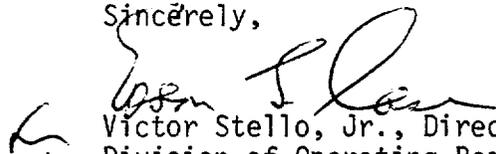
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Sincerely,


Victor Stello, Jr., Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

Enclosures and ccs:
See page 2

Georgia Power Company
Oglethorpe Electric Membership Corporation
Municipal Electric Association of Georgia
City of Dalton, Georgia

- 2 -

Enclosures:

1. Amendment No. 53
2. Safety Evaluation
3. Notice

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 53
License No. DPR-57

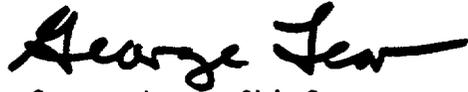
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al, (the licensee) dated November 16, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility requires an exemption to 10 CFR Part 50, Appendix J, Section III.C.2. The exemption is authorized by law, will not endanger life of property or the common defense and security, and is otherwise in the public interest. Therefore, the exemption is granted and now the facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 53, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 12, 1978

ATTACHMENT TO LICENSE AMENDMENT NO.53

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3.7-24
3.7-25
3.7-26*
3.7-27

Replace

3.7-24
3.7-25
3.7-26*
3.7-27

*Overleaf page

Table 3.7-4
(Continued)

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>
X-21	P51-F513 & F514	(1) (2) (4) (5) (9)
X-22	P70-F004	(1) (2) (4) (5) (10)
X-22	P70-F005	(1) (2) (4) (5) (10)
X-25	T48-F307, F308, F309, F103 & F324	(1) (2) (4) (5) (9)
X-25	T48-F113 & F114	(1) (2) (4) (5) (9)
X-25	T48-F321 & F322	"
X-25	T48-F104, F118A, F118B	(1) (2) (4) (5) (9)
X-26	T48-F319 & F320	(1) (2) (4) (5) (9)
X-26	T48-F334A & F335A	"
X-26	T48-F334B & F335B	"
X-26	T48-F340 & F341	"
X-26	P33-F002 & F010	
X-28	P33-F003 & F011	(1) (2) (4) (5) (9)
X-31	P33-F004 & F012	"
X-36	C11-F086	(1) (2) (4) (5) (10)
X-36	C11-F083	"
X-39A	E11-F016A & F021A	(1) (2) (4) (5) (9)
X-39A	E11-F016B & F021B	"
X-40	P70-F002 & F003	"
X-41	B21-F019 & F020	"
X-42	C41-F006	(1) (2) (4) (5) (10)
X-42	C41-F007	"
X-46	P21-F353 & F406	(1) (2) (4) (5) (9)
X-203	E-51-F003 & F031	"
X-204A	E11-F065A & F004A	(1) (2) (5) (9) (12)

Table 3.7-4
(Continued)

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>
X-204B	E11-F065B & F004B	(1) (2) (5) (9) (12)
X-204C	E11-F065C & F004C	"
X-204D	E11-F065D & F004D	"
X-205	T48-F310 & F328A	(1) (2) (4) (5) (9)
X-205	T48-F311 & F328B	"
X-207	E41-F042 & F051	"
X-208A	E21-F019A & F001A	"
X-208B	E21-F019B & F001B	"
X-210A	E11-F007A	(1) (2) (4) (5) (11)
X-210A	E21-F044A	"
X-210A	E51-F019 & F021	(1) (2) (4) (5) (9)
X-210A	E21-F031A	(1) (2) (4) (5) (11)
X-210A	E11-F024A, F027A & F028A	(1) (2) (4) (5) (9)
X-210B	E11-F007B	(1) (2) (4) (5) (11)
X-210B	E21-F044B	"
X-210B	G11-F007A & F008A	(1) (2) (4) (5) (9)
X-210B	G11-F007B & F008B	"
X-210B	E41-F012 & F046	"
X-210B	E21-F031B	(1) (2) (4) (5) (11)
X-210B	E11-F024B, F027B & F028B	(1) (2) (4) (5) (9)
X-212	E51-F001 & F040	(1) (2) (4) (5) (9)
X-213	E51-F002 & F028	"
X-214	E41-F021 & F049	"
X-215	E41-F022 & F040	"
X-217	P33-F007 & F015	"

Table 3.7-4
(Concluded)

Primary Containment Testable Isolation Valves

<u>Penetration Number</u>	<u>Valve Designation</u>	<u>Notes</u>				
X-220	P33-F006 & F014	(1)	(2)	(4)	(5)	(9)
X-220	T48-F318 & F326					"
X-220	T48-F332A & F333A					"
X-220	T-48-F332B & F333B					"
X-220	T-48-F338 & F339					"

Notes For Tables 3.7-2 through 3.7-4

- (1) Test duration for all valves and penetrations listed will generally exceed one hour.
- (2) Test pressures are at least 59 psig for all valves and penetrations except MSIV's which are tested at 28 psig.
- (3) MSIV acceptable leakage limit is 11.5 scfh/valve of air.
- (4) The total acceptable leakage for all valves and penetrations other than the MSIV's is $0.6 L_a$
- (5) Local leak tests on all testable isolation valves shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.
- (6) Local leak tests on all testable penetrations shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.
- (7) The personnel air lock shall be tested at intervals not to exceed 6 months.
- (8) The personnel air lock door seals are tested at 10 psig after each opening.
- (9) Identifies isolation valves that are tested by applying pressure between the inboard and outboard isolation valves. Inboard valve is not tested in the direction required for isolation but will have equivalent or more conservative leakage results.
- (10) Identifies isolation valves that are tested by applying pressure between the isolation valve and a manually operated valve such that the isolation valve is tested in the direction required for isolation.
- (11) Identifies isolation valves that are tested by applying pressure between the isolation valves and other system valves. Isolation valves not tested in the direction required for isolation will have equivalent or more conservative results.
- (12) The RHR system remains water filled post-LOCA. Isolation valves are tested with water.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1

DOCKET NO. 50-321

Introduction

By letter dated November 16, 1977, Georgia Power Company proposed changes to Facility Operating License DPR-57 for the Edwin I. Hatch Nuclear Plant Unit No. 1. Included in this request was the proposal to hydrostatically test the valves in the Residual Heat Removal (RHR) suction lines. This change constitutes an exemption from the requirements of Appendix J to 10 CFR 50.

Evaluation

Section III.C.2 of Appendix J to 10 CFR 50 requires that containment isolation valves subject to Type C testing be locally leakage rate tested by pressurization with air or nitrogen, unless a valve is pressurized with a fluid from a seal system. The licensee requested an exemption from this requirement to permit the containment isolation valves in the Residual Heat Removal (RHR) suction lines (i.e., valves E11-F065A through F065D and E11-F004A through F004D) to be locally leakage rate tested with water rather than air.

The containment isolation valves in the RHR suction lines are normally closed and open following a postulated loss-of-coolant accident to utilize the suppression pool as a source for reactor core cooling. In the event that one of the Emergency Core Cooling Systems fails to operate or becomes disabled, the isolation valve would close to isolate the inoperative system train.

Appendix J requires that leakage rate tests be performed with air or nitrogen to provide a conservative simulation of the post-accident environment. However, the isolation valves in the RHR suction lines will be covered by water from the suppression pool in the event that the valves must be closed. In addition, the suppression pool provides a sufficient source of water to assure that these valves will be covered with water for the duration of the accident. Therefore, testing these valves with water is representative of the post-accident environment. On this basis we conclude that the licensee's proposed exemption to permit hydrostatic (i.e., water testing) of RHR suction line isolation valves is acceptable.

The licensee also requested an exemption to eliminate the requirement to add the leakage rates for the RHR suction line isolation valves to the summation of the other local leakage rates, for the purpose of determining the acceptability of the local leakage test results. This request will be reviewed in conjunction with the evaluation of the leakage testing program for Hatch Unit No. 2. In the interim, we will require that the licensee add the leakage rates (i.e., volumetric flow) for the RHR suction line isolation valves to the summation of the other local leakage rates to determine the acceptability of the test results.

By using this acceptance criterion, we have added a degree of conservatism to the test results since by adding the total volumetric leakage of water, no credit for the partitioning of fission products in the water is taken.

We have determined that the provisions of 10 CFR, Appendix J do not specifically account for those situations in which isolation valves from a non-seal system would be covered with water under accident conditions. Thus, in order to comply with the current provisions of Appendix J, the licensee would be required to modify the current system to provide the capability of isolating and draining the RHR suction lines prior to a leakage test with air or nitrogen. The expenditure of time, manpower and funds required to affect this modification is not justified in view of a more conservative, alternate testing technique as discussed above. Therefore, in view of the discussion above, we have determined that it is in the best interest of the public to grant this exemption.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

Based on the foregoing, we have determined that, pursuant to 10 CFR Section 50.12, specific exemption for hydrostatic testing of the RHR system suction valves, as discussed above, can be granted without endangering life or property, or the common defense and security, and are otherwise in the public interest.

We have also concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does

not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 12, 1978

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-321GEORGIA POWER COMPANY, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSENOTICE OF GRANTING OF AN EXEMPTION FROM
REGULATIONS IN 10 CFR PART 50 APPENDIX J

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 53 to Facility Operating License No. DPR-57 issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1, located in Appling County, Georgia. The amendment is effective as of its date of issuance. The Commission has also granted an exemption from the regulations in 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing For Water-Cooled Power Reactors".

The amendment incorporated an approved exemption from the requirements of 10 CFR Part 50 Appendix J which requires that isolation valves be tested with air or nitrogen, to permit the testing of the RHR suction line valves with water based on the determination that the test condition would be representative of the post-accident environment.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amend-

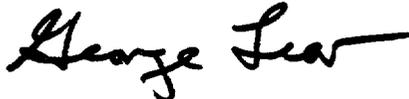
ment and related letter to the licensee. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 16, 1977, (2) Amendment No. 53 to License No. DPR-57, and (3) the Commission's Safety Evaluation and letter to the licensee dated April 12, 1978. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 12th day of April 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors