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Docket No. 50-321

Georgia Power Company
Oglethorpe Electric Membership Corporation
Municipal Electric Association of Georgia
City of Dalton, Georgia
ATTN: Mr. I. S. Mitchell, III
Vice President and Secretary
Georgia Power Company
Atlanta, Georgia 30302

Gentlemen:

The Commission has issued the enclosed Amendment No. 41 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your applications dated September 3, 1976 (supplemented by letter dated January 6, 1977) and January 26, 1977.

The amendment will revise the Technical Specifications relating to required periodic surveillance on the recirculation pump discharge valves and an administrative change to delete reference to recirculation pump discharge valve bypass line hydraulic shock suppressors.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
1. Amendment No. 41
2. Safety Evaluation
3. Federal Register Notice

cc: See next page

Const. 1
GD

OFFICE	ORB#3	ORB#3	OELD	ORB#3		
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DATE	4/ 5 /77	4/ 5 /77	4/ 8 /77	4/ 19 /77		

NRC FORM 318 (9-76) NRCM 0240 ☆ U.S. GOVERNMENT PRINTING OFFICE: 1976 — 626-624

Georgia Power Company
Oglethorpe Electric Membership Corporation
Municipal Electric Association of Georgia
City of Dalton, Georgia

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 41
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment by Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, (the licensees) dated September 3, 1976 (supplemented by letter dated January 6, 1977) and January 26, 1977, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 41, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 19, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 41

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed page. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove	Replace
3.5-3	3.5-3
3.5-4	3.5-4
3.6-10e	3.6-10e

3.5.B.1 Normal System Availability (Cont.)

One RHR loop with two pumps or two loops with one pump per loop shall be operable in the shutdown cooling mode when irradiated fuel is in the reactor vessel and the reactor pressure is atmospheric except prior to a reactor startup as stated in Specification 3.5.B.1.a.

c. The reactor shall not be started up with the RHR system supplying cooling to the fuel pool.

d. During reactor power operation, the LPCI system discharge cross-tie valve, E11-F010, shall be in the closed position and the associated valve motor starter circuit breaker shall be locked in the off position. In addition, an annunciator which indicates that the cross-tie valve is not in the fully closed position shall be available in the control room.

e. Both recirculation pump discharge valves shall be operable prior to reactor startup (or closed if permitted elsewhere in these specifications).

2. Operation with Inoperable Components

a. One LPCI Pump Inoperable

If one LPCI pump is inoperable, the reactor may remain in operation for a period not to exceed seven (7) days provided that the remaining LPCI pumps, both LPCI subsystem flow paths, the Core Spray system, and the associated diesel generators are operable.

b. One LPCI Subsystem Inoperable

A LPCI subsystem is considered to be inoperable if (1) both of the LPCI pumps within that system are inoperable or (2) the active valves in the subsystem flow path are inoperable.

4.5.B.1 Normal Operational Tests

Item	Frequency
b. Simulated Automatic Actuation Test	Once/Operating Cycle
c. System flow rate: Each RHR pump shall deliver at least 7700 gpm against a system head of at least 20 psig.	Once/3 months
d. Pump Operability	Once/month
e. Motor Operated valve operability	Once/month

f. Both recirculation pump discharge valves shall be tested for operability during any outage exceeding 48 hours, if operability tests have not been performed during the preceding month.

2. Surveillance with Inoperable Components

a. One LPCI Pump Inoperable

When one LPCI pump is inoperable, the remaining LPCI pumps and associated flow paths, the Core Spray system, and the associated diesel generators shall be demonstrated to be operable immediately and daily thereafter, until the inoperable LPCI pump is restored to normal service.

b. One LPCI Subsystem Inoperable

When one LPCI subsystem is inoperable, all active components of the remaining LPCI subsystem, the Core Spray system, and the associated diesel generators shall be demonstrated to be operable, immediately

3.5.B.2. Operation with Inoperable Components (Continued)

b. If one LPCI subsystem is inoperable, the reactor may remain in operation for a period not to exceed seven (7) days provided that all active components of the remaining LPCI subsystem, the Core Spray system, and the associated diesel generators are operable.

4.5.B.2. Surveillance with Inoperable Components (Continued)

and daily thereafter, until the inoperable LPCI subsystem is restored to normal service.

SNUBBER #	SIZE	LOCATION	ELEVATION	*SHUTDOWN ACCESSIBILITY CODE
SS-A1	35	315°	123'	RA
A2	50	315°	123'	RA
A3	50	315°	123'	RA
A4	50	310°	131'	RA
A5	50	320°	131'	RA
A6	50	315°	134'	RA
A7	21	15°	134'	RA
A8	35	10°	134'	RA
A13	35	270°	145'	RA
A14	50	270°	122'	RA
A12	35	270°	145'	RA
SS-B1	21	140°	120'	RA
B2	50	135°	123'	RA
B3	50	135°	123'	RA
B4	50	145°	131'	RA
B5	50	135°	131'	RA
B6	50	135°	137'	DA
B7	21	185°	140'	RA
B8	35	180°	140'	RA
B12	35	90°	145'	RA
B13	35	90°	145'	RA
B14	50	90°	116'	RA

SNUBBERS ACCESSIBLE DURING REACTOR OPERATION

CORE SPRAY SYSTEM

CSH-75	3	10'NR3-7'WRL	125'	RA
71	10	7'NR13-10'WRL	121'	RA
79	10	2'NR9-7'WRH	172'	RA

HPCI SYSTEM

HPCIH-9	20	13'SR1-6'ERG	88'	RA
13	50	7'SR1-2'WRL	94'	RA
12	10	5'NR3-3'ERF	123'	RA
HPSEH-2	10	12'NR2-10'WRL	92'	RA
8	10	6'NR2-4ERG	112'(2)	RA
12	10	5'NR3-3'ERF	123'8"	RA
13	10	4'NR3-3'ERF	123'6"(2)	RA
17	10	5'NR3-14'ERF	123'6"	RA
57	3	1'SR1-18'WRL	99 1/2'	RA
58	3	4'SR1-18'WRL	99'	RA
60	3	4'NR2-4'ERG	120'(2)	RA
61	10	3'NR5-11'ERH	123'	RA
62	10	3'NR5-11'ERH	123'	RA

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NO. DPR-57

GEORGIA POWER COMPANY

OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA

CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1

DOCKET NO. 50-321

Introduction

By letters dated September 3, 1976, January 6, and 26, 1977 Georgia Power Company (GPC) requested amendment changes to Facility Operating License DPR-57 for Edwin I. Hatch Nuclear Plant Unit No. 1. The amendment would modify the Technical Specifications to require periodic surveillance on the recirculating pump discharge valves (RPDV) and would delete reference to hydraulic shock suppressors which will be removed with the 4" recirculation pump discharge valve bypass lines. The amendment application is in response to our letter dated July 28, 1976.

Evaluation

Edwin I. Hatch Nuclear Plant Unit No. 1 is one of several boiling water reactors on which the Low Pressure Coolant Injection System (LPCIS) modification has been completed. An essential element of this modification involves the closure of the RPDVs upon LPCIS initiation following a Loss of Coolant Accident.

The closure of the RPDV is necessary to isolate a pipe rupture occurring in the recirculation loop suction line and thereby ensure that the LPCIS will not discharge makeup water back through the recirculation pump and out of the break. The failure of RPDV to close upon LPCIS initiation has an adverse affect on core cooling similar to the failure of a LPCIS injection valve to open. The failure of a LPCIS injection valve to open is the limiting single failure in the Edwin I. Hatch ECCS analysis.

Currently, the RPDV's are tested only during refueling outages which occur every 12 - 18 months. The standard interval for surveillance testing of motor operated ECCS valves is 31 days. We consider it desirable for surveillance to be performed on the RPDV in a manner, and with a surveillance frequency similar to that which is performed on the LPCIS injection valves. However, unlike the LPCIS injection valves, the RPDV's cannot be tested during power operation. We have considered the known safety effects on the plant associated with a plant shutdown and cooldown, solely for the purpose of testing these valves and have determined that the increase in reliability that might be gained does not justify an interruption in normal plant operations. Therefore, we have required that the RPDV be tested during periods of reactor cold shutdown in excess of 48 hours if they have not been tested in the previous 31 days. For most operating BWR's today this cold shutdown period would occur, because of the necessity for maintenance and other planned operations, every 3 - 4 months. This expected outage frequency is consistent with the 3-month surveillance interval specified in Section XI of the ASME Boiler and Pressure Vessel Code.

On the basis of the foregoing, we consider the proposed changes to be improvements to overall plant safety and reliability; therefore, the changes are acceptable.

The 4" recirculation pump discharge bypass lines, being removed during the Cycle 2 reload outage, have hydraulic shock suppressors attached to prevent damage to the bypass lines. Since the suppressors are attached to the lines and serve no other function than to protect the lines being removed, they also must be removed and reference to the suppressors in the "Safety Related Shock Suppressors" table must be deleted.

Since deletion of reference to the hydraulic shock suppressors is an administrative change only, we consider the proposed change to have no affect on plant safety and reliability and, therefore, the change is acceptable.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 19, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 41 to Facility Operating License No. DPR-57 issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

The amendment consists of changes to the Technical Specifications relating to required periodic surveillance on the recirculation pump discharge valves and an administrative change to delete reference to recirculation pump discharge valve bypass line hydraulic shock suppressors.

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the applications for amendment dated September 3, 1976, (supplemented by letter dated January 6, 1976) and January 26, 1977, (2) Amendment No. 41 to License No. DPR-57 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 19 day of April 1977.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors