

MARCH 2 1978

Docket No. 50-321

Georgia Power Company
Oglethorpe Electric Membership Corporation
Municipal Electric Association of Georgia
City of Dalton, Georgia
ATTN: Mr. Charles F. Whitmer
Vice President - Engineering
Georgia Power Company
Atlanta, Georgia 30302

Gentlemen:

The Commission has issued the enclosed Amendment No. ⁵⁰ to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit No. 1. The amendment consists of changes to the Technical Specifications and is in accordance with your application dated January 24, 1977, as revised by letters dated July 22, 1977 and December 20, 1977 and our letter of November 23, 1977.

The amendment consists of changes in the Technical Specifications that incorporate the existing Fire Protection System into the Limiting Conditions for Operation, Surveillance Requirements and Administrative Controls.

The following changes to the Technical Specifications from those sent to you in our November 23, 1977 letter were made at your request:

- (1) Specification 3.13.2, Action b.1 from 10 day report to 14 day report to be consistent with the Standard Technical Specifications,
- (2) Specification 4.13.2.e from 6 months to 18 months for a system flush since the water quality records indicate that there will not be a significant problem with scaling and corrosion in the system,
- (3) Table 3.13-2 since the original table contained some errors,
- (4) Specification 6.2.2.f by replacing the word "Brigade" by the word "squad" to be consistent with Plant Procedures, (5) Specification 6.4.2 by replacing the words "Fire Brigade" by "fire protection" to include all training and adding the requirement of quarterly training to be consistent with the latest staff guidance, and (6) Figure 6.2-1 modified to correct the titles. Since these revisions are administrative matters, the staff finds them acceptable.

Cond. 1
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OFFICE >						
SURNAME >						
DATE >						

In order to achieve expeditious implementation of the Fire Protection Technical Specifications, Specification 3.13.6 is being issued with the wording you proposed to allow penetration surveillance every four hours with the reactor in the cold shutdown condition and no work is being performed at the penetration. However, we are presently evaluating your justification for the relaxation of the continuous watch requirement and when the evaluation is completed your wording addition will be deleted if we do not agree with your position. Also for expeditious implementation of the Fire Protection Technical Specifications, Specification 6.5.2.8.j is being issued with the wording you proposed to use the Quality Assurance Department for inspection and audit instead of an outside fire consultant. However, we are presently evaluating your justification for this change and when the evaluation is completed the requirement for an outside consultant will be reestablished if we do not agree with your position.

Figure 6.2-2 is not being issued with this amendment. Your figure reflecting the plant organization was not accompanied by an explanation for the changes made nor definition of the Regulatory Specialist who is responsible for plant fire protection procedures and training.

All other Specifications are the same as those transmitted to you on November 23, 1977, and are supported by the Safety Evaluation which was attached to that letter.

Copies of the related FEDERAL REGISTER Notice also are enclosed.

Sincerely,

Original signed by

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

1. Amendment No. to License DPR-57
2. FEDERAL REGISTER Notice

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Georgia Power Company
Oglethorpe Electric Membership Corporation
Municipal Electric Association of Georgia
City of Dalton, Georgia

- 2 -

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, (the licensees) dated January 24, 1977 (as revised by letters dated July 22 and December 20, 1977), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

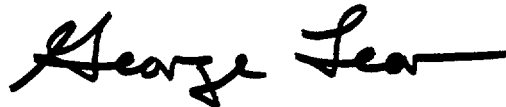
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 50, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: MARCH 2 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 50
TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-57
DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages and add pages 3.13-1 through 3.13-12. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

v
viii
1.0-7
6-1
6-4
6-9
Figure 6.2-1

Replace

v
viii
1.0-7
6-1
6-4
6-9
Figure 6.2-1

LIMITING CONDITIONS	OR OPERATION	SURVEILLANCE	REQUIREMENTS
3.9 AUXILIARY ELECTRICAL SYSTEMS (CONT'D)	4.9 AUXILIARY ELECTRICAL SYSTEMS (CONT'D)		
B. Requirements for Continued Operation With Inoperable Components	B. Requirements for Continued Operation With Inoperable Components	3.9-4	
C. Diesel Generator Requirements (Reactor in the Shutdown or Refuel Mode)			3.9-6
3.10 REFUELING	4.10 REFUELING		3.10-1
A. Refueling Interlocks	A. Refueling Interlocks		3.10-1
B. Fuel Loading			3.10-1
C. Core Monitoring During Core Alterations	C. Core Monitoring During Core Alterations		3.10-2
D. Spent Fuel Pool Water Level	D. Spent Fuel Pool Water Level		3.10-2
E. Control Rod Drive Maintenance	E. Control Rod Drive Maintenance		3.10-2
F. Reactor Building Cranes	F. Reactor Building Cranes		3.10-4
G. Spent Fuel Cask Lifting Trunnions and Yoke	G. Spent Fuel Cask Lifting Trunnions and Yoke		3.10-5
H. Time Limitation			3.10-5
3.11 FUEL RODS	4.11 FUEL RODS		3.11-1
A. Average Planar Linear Heat Generation Rate (APLHGR)	A. Average Planar Linear Heat Generation Rate (APLHGR)		3.11-1
B. Linear Heat Generation Rate (LHGR)	B. Linear Heat Generation Rate (LHGR)		3.11-1
C. Minimum Critical Power Ratio (MCPR)	C. Minimum Critical Power Ratio (MCPR)		3.11-1
3.12 MAIN CONTROL ROOM ENVIRONMENTAL SYSTEM	4.12 MAIN CONTROL ROOM ENVIRONMENTAL SYSTEM		3.12-1
A. Ventilation System Operability	A. Ventilation System Tests		3.12-1
B. Isolation Valve Operability and Closing Time	B. Isolation Valve Testing		3.12-2
C. Radiation Monitors	C. Radiation Monitors		3.12-2
D. Shutdown Requirements			3.12.-3
E. Chlorine Monitors	E. Chlorine Monitors		3.12-3
3.13 FIRE PROTECTION SYSTEMS	4.13 FIRE PROTECTION SYSTEMS		

LIST OF TABLES
(Concluded)

<u>Table</u>	<u>Title</u>	<u>Page</u>
4.2-7	Check, Functional Test, and Calibration Minimum Frequency for Neutron Monitoring Instrumentation Which Initiates Control Rod Blocks	3.2-40
4.2-8	Check, Functional Test, and Calibration Minimum Frequency for Radiation Monitoring Systems Which Limit Radioactivity Release	3.2-42
4.2-9	Check and Calibration Minimum Frequency for Instrumentation Which Initiates Recirculation Pump Trip	3.2-45
4.2-10	Check, Functional Test, and Calibration Minimum Frequency for Instrumentation Which Monitors Leakage into the Drywell	3.2-46
4.2-11	Check and Calibration Minimum Frequency for Instrumentation Which Provides Surveillance Information	3.2-48
3.6-1	Safety Related Shock Suppressors (Snubbers)	3.6-10c
4.6-1	In-Service Inspection Program	3.6-11
3.7-1	Primary Containment Isolation Valves	3.7-16
3.7-2	Testable Penetrations with Double O-Ring Seals	3.7-21
3.7-3	Testable Penetrations with Testable Bellows	3.7-22
3.7-4	Primary Containment Testable Isolation Valves	3.7-23
3.13-1	Fire Detectors	3.13-2
3.13-2	Fire Hose Stations	3.13-9
6.0-1	Shift Manning Chart for Unit Operation	6.0-23
6.0-3	Special Test Report Requirements	6.0-25

- MM. Minimum Critical Power Ratio (MCPR) - Minimum Critical Power Ratio (MCPR) is the value of the critical power ratio associated with the most limiting assembly in the reactor core. Critical Power Ratio (CPR) is the ratio of that power in a fuel assembly, which is calculated to cause some point in the assembly to experience boiling transition, to the actual assembly operating power.
- NN. Trip System - A trip system means an arrangement of instrument channel trip signals and auxiliary equipment required to initiate action to accomplish a protective function. A trip system may require one or more instrument channel trip signals related to one or more plant parameters in order to initiate trip system action. Initiation of protective action may require the tripping of a single trip system or the coincident tripping of two trip systems.
- OO. Cumulative Downtime - The cumulative downtime for those safety components and systems whose downtime is limited to 7 consecutive days prior to requiring reactor shutdown shall be limited to any 7 days in a consecutive 30 day period.
- PP. Fire Suppression Water System - A Fire Suppression Water System shall consist of: a water source(s), or pump(s); and distribution piping with associated sectionalizing control or isolation valves. Such valves shall include yard hydrant curb valves, and the first valve ahead of the water flow alarm device on each sprinkler, hose standpipe or spray system riser.
- QQ. Channel Calibration - A Channel Calibration is the adjustment, as necessary, of the channel output such that it responds with the necessary range and accuracy to known values of the parameter which the channel monitors. The Channel Calibration shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the Channel Functional Test. The Channel Calibration may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.
- RR. Channel Functional Test - A Channel Functional Test shall be:
- a. Analog channels - the injection of a simulated signal into the channel as close to the primary sensor as practicable to verify operability including alarm and/or trip functions.
 - b. Bistable channels - the injection of a simulated signal into the channel sensor to verify operability including alarm and/or trip functions.

3/4.13 FIRE PROTECTION SYSTEMS

3/4.13.1 FIRE DETECTION INSTRUMENTATION

LIMITING CONDITIONS FOR OPERATION

3.13.1 As a minimum, the fire protection instrumentation for each fire detection zone shown in Table 3.13-1 shall be OPERABLE.

APPLICABILITY: At all times when equipment in that fire detection zone is required to be OPERABLE.

ACTION:

With the number of OPERABLE fire detection instruments less than required by Table 3.13-1;

1. Within 1 hour, establish a fire watch patrol to inspect the zone with the inoperable instrument(s) at least once per hour, and
2. Restore the inoperable instrument(s) to OPERABLE status within 14 days or prepare and submit a special report to the Commission within the next 30 days outlining the cause of the malfunction and the plans for restoring the instrument(s) to OPERABLE status.

SURVEILLANCE REQUIREMENTS

4.13.1.1 Each of the above fire detection instruments shall be demonstrated OPERABLE: at least once per 6 months by a CHANNEL FUNCTIONAL TEST.

4.13.1.2 The circuitry supervision associated with the detector alarms shall be demonstrated OPERABLE at least once per six months for all NFPA Code 72D Class A supervised circuits.

TABLE 3.13-1

FIRE DETECTORS

<u>BUILDING</u>	<u>NUMBER OF DETECTORS</u>	<u>MINIMUM OPERABLE DETECTORS REQUIRED</u>
A.1 Control Building El. 112'-0"		
1.1 Station Battery Room 1A	2	1
1.2 Station Battery Room 1B	2	1
A.2 Control Building El. 130'-0"		
2.1 Switchgear Rooms 1A, 1B 1E, 1F, and Transformer Room	8	(1 in each room or 5)
A.3 Control Building El. 147'-0"		
3.1 Switchgear & MCC Area N.W. Corner	7	3
3.2 Switchgear & MCC Area N.E. Corner	4	2
3.3 Cable Spreading Room	13	6
A.4 Control Building El. 164'-0"		
4.1 Control Room Unit 1	21	10
4.2 Control Room Unit 2	15	7
B.1 Diesel Generator Building		
1.1 Switchgear Room 1E	4	2
1.2 Switchgear Room 1F	4	2
1.3 Switchgear Room 1G	4	2
1.4 Battery Room 1A	1	1
1.5 Battery Room 1B	1	1
1.6 Battery Room 1C	1	1
C.1 River Intake Structure		
1.1 Building Area	11	5
D.1 Reactor Building El. 130'-0"		
1.1 North Cable Tray Area and Remote Shutdown Area	4	2
1.2 South Cable Tray Area	3	1

3/4.13 FIRE PROTECTION SYSTEMS

3/4.13.2 FIRE SUPPRESSION WATER SYSTEMS

LIMITING CONDITIONS FOR OPERATION

3.13.2 The FIRE SUPPRESSION WATER SYSTEM shall be OPERABLE with;

- a. (2) high pressure pumps each with a capacity of 2500 gpm, with their discharge aligned to the fire suppression header.
- b. Separate water supplies each containing a minimum of 270,000 gallons.
- c. Automatic initiation logic for each fire pump.

APPLICABILITY: At all times.

ACTION:

- a. With less than the above required equipment, restore the inoperable equipment to OPERABLE status within 7 days or prepare and submit a Special Report to the Commission within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system.
- b. With no FIRE SUPPRESSION WATER SYSTEM OPERABLE, within 24 hours;
 1. Establish a backup FIRE SUPPRESSION WATER SYSTEM, and notify the Commission by telephone and provide a special 14 day report outlining the actions taken and the plans and schedule for restoring the system to operable status, or
 2. An orderly shutdown shall be initiated and the reactor shall be in the cold shutdown condition within the next 24 hours.

3/4.13 . FIRE PROTECTION SYSTEMS

3/4.13.2 FIRE SUPPRESSION WATER SYSTEMS

SURVEILLANCE REQUIREMENTS

- 4.13.2 The FIRE SUPPRESSION WATER SYSTEM shall be demonstrated OPERABLE:
- a. At least once per 7 days by verifying the water supply volume.
 - b. At least once per 31 days by starting each pump and operating it for at least 20 minutes on recirculation flow.
 - c. At least once per 31 days by verifying that each valve in the flow path is in its correct position.
 - d. At least once per 12 months by cycling each testable valve through one complete cycle.
 - e. At least once per 18 months by performance of a system flush.
 - f. At least once per 18 months:
 1. By performing a system functional test which includes simulated automatic actuation of the system throughout its operating sequence and verifying that each automatic valve in the flow path actuates to its correct position.
 2. By verifying that each pump develops at least 2500 gpm at a system head of 120 psi.
 - g. At least once per 3 years by performing a flow test of the system in accordance with Chapter 5, Section 11 of the Fire Protection Handbook, 14th Edition, published by the National Fire Protection Association.
 - h. At least once per 31 days by verifying:

The fuel storage tank contains at least 275 gallons of fuel, and the diesel starts from ambient conditions and operates for at least 20 minutes.

- i. At least once per 18 months, during shutdown by subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for the class of service.

3.14-4a

3/4.13 FIRE PROTECTION SYSTEMS

3/4.13.3 SPRAY AND/OR SPRINKLER SYSTEMS

LIMITING CONDITIONS FOR OPERATION

3.13.3 The following spray and/or sprinkler systems located in the following areas shall be OPERABLE:

- a. Turbine Lube Oil Storage
- b. Turbine Lube Oil Reservoir
- c. RCIC Room
- d. HPCI Room
- e. Recirculation Pump Motor Generator Room
- f. East Cableway

APPLICABILITY: At all times when equipment in the area is required to be OPERABLE.

ACTION: ,

- a. With a spray and/or sprinkler system inoperable establish a continuous fire watch with backup fire suppression equipment for the unprotected area(s), within 1 hour, provided radiation levels permit personnel access.
- b. Restore the system to OPERABLE status within 14 days or prepare and submit a Special Report to the Commission within the next 30 days outlining the cause of inoperability and the plans for restoring the system to OPERABLE status.

SURVEILLANCE REQUIREMENTS

4.13.3 The spray and/or sprinkler systems shall be demonstrated to be OPERABLE:

- a. At least once per 12 months by cycling each automatic valve through one complete cycle.
- b. At least once per 18 months:
 1. By performing a system functional test which includes simulated automatic actuation of the system and verifying that the automatic valves in the flow path actuate to their correct positions.
 2. By inspection of spray headers to verify their integrity.

3/4.13 FIRE PROTECTION SYSTEMS

3/4.13.3 SPRAY AND/OR SPRINKLER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

3. By inspection of each nozzle to verify no blockage.
- c. At least once per 3 years by performing an air flow test through each spray/sprinkler header and verifying each open head spray/sprinkler nozzle is unobstructed.

3.13-6

3/4.13 FIRE PROTECTION SYSTEMS

3/4.13.4 CO₂ SYSTEMS

LIMITING CONDITIONS FOR OPERATION

3.13.4 The following CO₂ systems shall be OPERABLE with a minimum level of 60% and a minimum pressure of 275 psi in the associated storage tank(s).

- a. Control Building CO₂ System
- b. Diesel Generator Building CO₂ System

APPLICABILITY: At all times when the equipment in the area is required to be OPERABLE.

ACTION:

- a. With a CO₂ system inoperable, establish a continuous fire watch with backup fire suppression equipment for the unprotected area(s), within 1 hour.
- b. Restore the system to OPERABLE status within 14 days or prepare and submit a Special Report to the Commission within the next 30 days outlining the cause of inoperability and the plans for restoring the system to OPERABLE status.

SURVEILLANCE REQUIREMENTS

4.13.4 The CO₂ system shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying each CO₂ storage tank level and pressure.
- b. At least once per 18 months by verifying the system valves and associated ventilation dampers actuate automatically and manually to a simulated actuation signal. A brief flow test shall be made to verify flow from each nozzle. ("Puff Test").

3/4.13 FIRE PROTECTION SYSTEMS

3/4.13.5 FIRE HOSE STATIONS

LIMITING CONDITIONS FOR OPERATION

3.13.5 The fire hose stations listed in Table 3.13-2 shall be OPERABLE:

APPLICABILITY: At all times when the equipment in the area is required to be OPERABLE:

ACTION:

With a hose station inoperable, route an additional equivalent capacity hose to the unprotected area from an OPERABLE hose station within 1 hour.

SURVEILLANCE REQUIREMENTS

4.13.5 Each fire hose station shall be verified OPERABLE:

- a. At least once per 31 days by visual inspection of the station to assure all equipment is available and the pressure in the standpipe is within limits.
- b. At least once per 18 months by removing the hose for inspection and re-racking and replacing all gaskets in the couplings that are degraded.
- c. At least once per 3 years, partially open each hose station valve to verify valve operability and no blockage.
- d. At least once per 3 years by conducting a hose hydrostatic test at a pressure at least 50 psig greater than the maximum pressure available at that hose station.

TABLE 3.13-2

FIRE HOSE STATIONSA. Control Building

1.	E1. 112'	Column	T11-TA
2.	E1. 112'		T13-TE
3.	E1. 112'		T11-TH
4.	E1. 130'		T13-TE
5.	E1. 130'	16'-South of	T11-TH
6.	E1. 130'		T11-TA
7.	E1. 147'		T11-TH
8.	E1. 147'		T13-TEa
9.	E1. 147'		T5-TA
10.	E1. 147'		T8-TA
11.	E1. 164'		T11-TH
12.	E1. 164'		T13-TE
13.	E1. 164'		T11-TA

B. Reactor Building

1.	E1. 87'	Column	R13 & RH
2.	E1. 87'		R13 & RB2
3.	E1. 87'		R2 & RE
4.	E1. 87'		R4 & RL
5.	E1. 87'		R13 & RA
6.	E1. 87'		R10 & RL
7.	E1. 87'		R2 & RL
8.	E1. 87'		R2 & RL
9.	E1. 87'		R2 & RA
10.	E1. 130'		R10 & RB
11.	E1. 130'		R13 & RJ
12.	E1. 130'		R4 & RL
13.	E1. 130'		R2 & RB
14.	E1. 158'		R9 & RB
15.	E1. 153'		R5 & RL
16.	E1. 158'		R2 & RF
17.	E1. 185'		R9 & RB
18.	E1. 185'		R2 & RF
19.	E1. 185'		R5 & RL
20.	E1. 185'		R13 & RH
21.	E1. 203'		R13 & RH
22.	E1. 203'		R5 & RL
23.	E1. 228'		R9 & RD
24.	E1. 228'		R2 & RE
25.	E1. 228'		R5 & RK
26.	E1. 228'		R13 & RH

3/4.13 FIRE PROTECTION SYSTEMS

3/4.13.6 FIRE BARRIER PENETRATION FIRE SEALS

LIMITING CONDITIONS FOR OPERATION

3.13.6 All penetration fire barriers protecting safety related areas shall be functional.

APPLICABILITY: At all times

ACTION:

- a. With a penetration fire barrier non-functional, a continuous fire watch, or equivalent monitoring, shall be established on at least one side of the affected penetration within 1 hour unless the reactor is maintained in the cold shutdown condition with no work being performed at the penetration, in which case the penetration shall be surveyed once every four hours for fire hazard detection.

SURVEILLANCE REQUIREMENTS

4.13.6.1 Penetration fire barriers shall be verified to be functional by a visual inspection;

- a. At least once per 18 months, and
- b. Prior to declaring a fire penetration seal functional following repairs or maintenance.

BASES

FIRE DETECTION INSTRUMENTATION

OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment and is an integral element in the overall facility fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of frequent fire patrols in the affected areas is required to provide detection capability until the inoperable instrumentation is returned to service.

FIRE SUPPRESSION SYSTEMS

The OPERABILITY of the fire suppression systems ensures that adequate fire suppression capability is available to confine and extinguish fires occurring in any portion of the facility where safety related equipment is located. The fire suppression system consists of the water system, spray and/or sprinklers, CO₂, and fire hose stations. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment and is a major element in the facility fire protection program.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the affected equipment can be restored to service.

In the event that the fire suppression water system becomes inoperable, immediate corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the nuclear plant.

FIRE BARRIER PENETRATION SEALS

The functional integrity of the fire barrier penetration seals ensures that fires will be confined or adequately retarded from spreading to adjacent portions of the facility. This design features minimizes the possibility of a single fire rapidly involving several areas of the facility prior to detection and extinguishment. The fire barrier penetration seals are a passive element in the facility fire protection program and are subject to periodic inspections.

During periods of time when the seals are not functional, a continuous fire watch is required, to be maintained in the vicinity of the affected seal until the seal is restored to functional status.

3.13-12

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

- 6.1.1 The Plant Superintendent shall be responsible for overall plant operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

Offsite

- 6.2.1 The offsite organization for plant management and technical support is shown in relation to plant supervision on Figure 6.2-1.

Plant Staff

- 6.2.2 The plant organization shall be as shown on Figure 6.2-2, and:
- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
 - b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
 - c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown, and during recovery from reactor trips.
 - d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
 - e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
 - f. A Fire Squad of at least 5 members shall be maintained onsite at all times. The Fire Squad shall not include the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.

6.3.12 Shift Supervisors

The Shift Supervisors shall have a minimum of a high school diploma or equivalent and four years of responsible power plant experience, of which a minimum of one year shall be nuclear power plant experience. A maximum of two years of the remaining three years of power plant experience may be fulfilled by academic or related technical training on a one-for-one time basis. Each shall hold a Senior Reactor Operator License.

6.3.13 Plant Operators and Assistant Plant Operators

The Plant Operators and Assistant Plant Operators shall have a high school diploma or equivalent and two years of power plant experience, of which a minimum of one year shall be nuclear power plant experience. Each shall hold a Reactor Operator License.

6.3.14 Auxiliary Equipment Operators

The Auxiliary Equipment Operators shall have a high school diploma or equivalent, and should possess a high degree of manual dexterity and mature judgment.

6.3.15 Technicians

Technicians in responsible positions shall have a minimum of two years of working experience in their specialty and should have a minimum of one year of related technical training in addition to their experience.

6.3.16 Maintenance Personnel

Maintenance personnel in responsible positions shall have a minimum of three years experience in one or more crafts. Any time maintenance is performed on a safety related system, those maintenance personnel performing the task shall meet this requirement or they shall receive detailed supervision from one who does. They should possess a high degree of manual dexterity and ability and should be capable of learning and applying basic skills in maintenance operations.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the plant staff shall be maintained under the direction of the Training Specialist and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971, and Appendix "A" of 10 CFR, Part 53.

6.4.2 A training program for fire protection shall be maintained under the direction of the Training Specialist and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for Fire Squad Training sessions which will be held at least quarterly.

Audits (continued)

- d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- e. The plant Emergency Plan and implementing procedures at least once per two years.
- f. The plant Security Plan and implementing procedures at least once per two years.
- g. Any other area of plant operation considered appropriate by the SRB or the Senior Vice President - Power Supply.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by the Quality Assurance Department with offsite consultant expertise available as required at least once per 36 months.

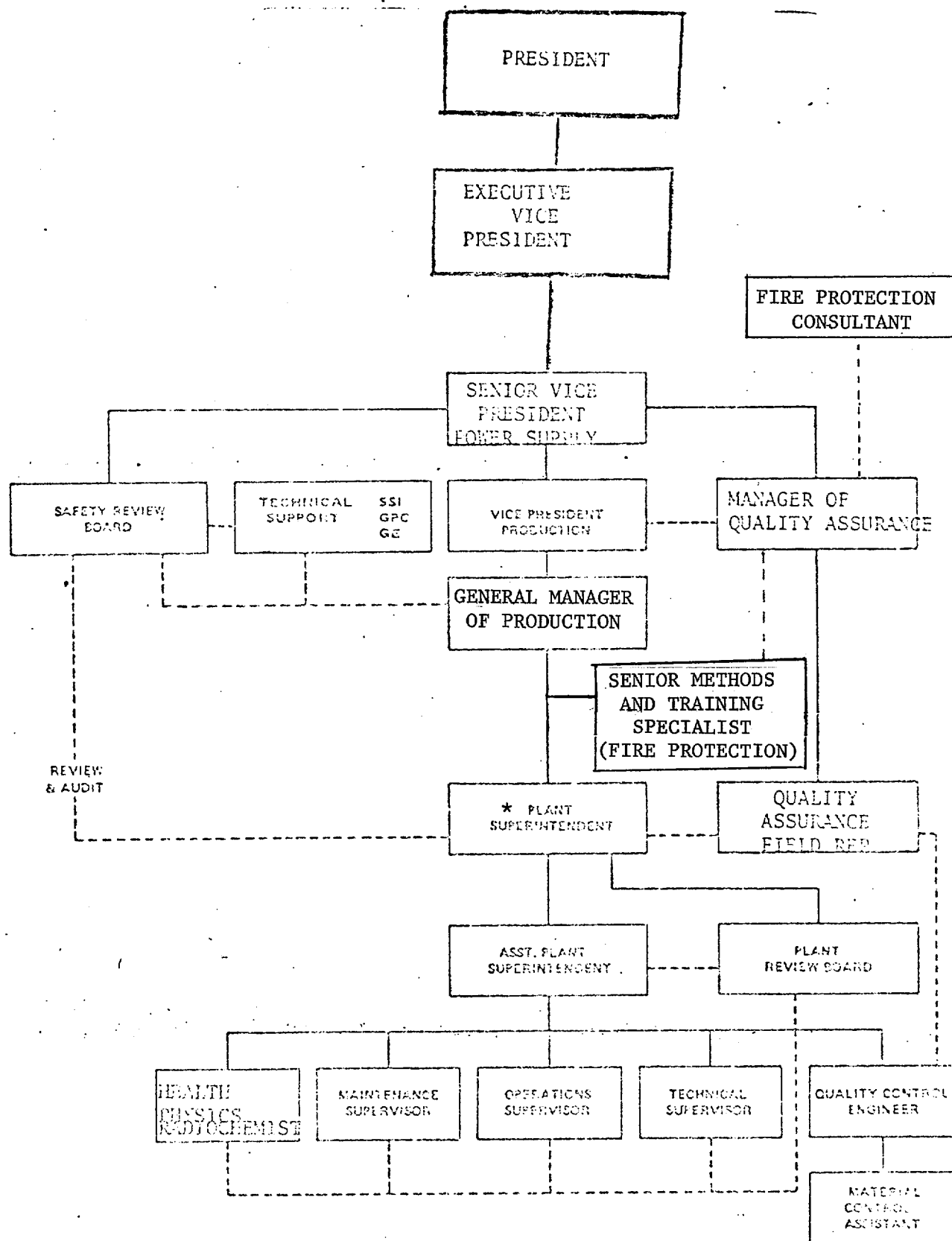
Authority

- 6.5.2.9 The SRB shall report to and advise the Senior Vice President Power Supply on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

Records

- 6.5.2.10 Records of SRB activities shall be prepared, approved, and distributed as indicated below:
- a. Minutes of each SRB meeting shall be prepared, approved, and forwarded to the Senior Vice President - Power Supply within 14 days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7. e, f, g, and h above shall be prepared, approved, and forwarded to the Senior Vice President - Power Supply within 14 days following completion of the review.
 - c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President - Power Supply and to the management positions responsible for the areas audited within 30 days after completion of the audit.

MANAGERIAL ORGANIZATION



* Overall responsibility for fire protection

— LINES OF RESPONSIBILITY

- - - LINES OF COMMUNICATION

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-321GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIANOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 50 to Facility Operating License No. DPR-57 issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 1, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

The amendment incorporates fire protection Technical Specifications on the existing fire protection equipment and adds administrative controls related to fire protection at the facility. This action is being taken pending completion of the Commission's overall fire protection review of the facility.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

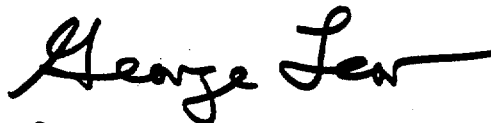
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated January 24, 1977 (as revised by letters dated July 22 and December 20, 1977), (2) the Commission's November 23, 1977 letter and related Safety Evaluation Report, (3) Amendment No. 50 to License No. DPR-57 and (4) the Commission's letter dated All of these items are available for public inspection at the the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. A copy of itmes (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 2nd day of March 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors