

Docket No. 50-321

MAR 5 1976

Georgia Power Company
Oglethorpe Electric Membership Corporation
ATTN: Mr. I. S. Mitchell, III
Vice President and Secretary
Georgia Power Company
Atlanta, Georgia 30302

Gentlemen:

In response to your request dated March 2, 1976, the Commission has issued the enclosed Amendment No. 29 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit 1.

The amendment consists of changes in the Technical Specifications to make the requirements on the operability of Pressure Suppression Chamber (TORUS) to Drywell Vacuum Breakers more consistent with standard Technical Specifications.

Certain changes to the original application were made with the mutual approval of the NRC staff and your licensing representatives.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

1. Amendment No. 29 to License DPR-57
2. Safety Evaluation
3. Federal Register Notice

cc: See next page

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Georgia Power Company & Oglethorpe
Electric Membership Corporation

cc:

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Mr. G. Wyman Lamb, Chairman
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company and Oglethorpe Electric Membership Corporation (the licensees) dated March 2, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter A. Paulson
for George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Date of Issuance: MAR 5 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 29

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace page 3.7-8 with the attached revised page.
No change has been made on page 3.7-7.

4.7.A.2.i. Continuous Leak Rate Monitor

- (1) When the primary containment is inerted, the containment shall be continuously monitored for gross leakage by review of the inerting system makeup requirements.
- (2) This monitoring system may be taken out of service for the purpose of maintenance or testing but shall be returned to service as soon as practical.

j. Interior Drywell Surfaces

The interior surfaces of the drywell shall be visually inspected each operating cycle for evidence of deterioration.

.A.3. Reactor Building to Pressure Suppression Chamber-Vacuum Relief System

- a. Except as specified in 3.7.A.3.b below, two reactor building to pressure suppression chamber vacuum relief lines consisting of an air-operated valve and a check valve in series shall be operable at all times when the primary containment integrity is required. The setpoint of the differential pressure instrumentation which actuates the reactor building to pressure suppression chamber vacuum relief valves shall be 0.5 psid, or less.
- b. From and after the date that one of the reactor building to pressure suppression chamber vacuum relief valves are made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such vacuum relief valve is sooner made operable, provided that the procedure does not violate primary containment integrity.

3. Reactor Building to Pressure Suppression Chamber - Vacuum Relief System

The reactor building to pressure suppression chamber vacuum relief valves and associated instrumentation, including setpoint, shall be checked for proper operation every three months.

3.7.A.4 Pressure Suppression Chamber to Drywell Vacuum Breakers

- a. When primary containment is required, all pressure suppression chamber to drywell vacuum breakers shall be operable and positioned in the fully closed position (except during testing) except that up to three vacuum breakers may be inoperable for opening provided that they are known to be in the closed position.
- b. If either of the closed position indicating lights for a pressure suppression chamber to drywell vacuum breaker is inoperable, continued reactor operation is permissible only if: (1) the operability of the redundant closed position indicating circuit is verified, and (2) a leakage test of the pressure suppression chamber to drywell vacuum breaker system is satisfactorily performed within 24 hours.

If either of these requirements cannot be met, the reactor must be in the cold shutdown condition within 36 hours.

- c. The differential pressure which actuates the pressure suppression chamber to drywell vacuum breakers shall be 0.5 psid or less.
- d. The total leakage between the drywell and pressure suppression chamber shall be less than the equivalent leakage through a one-inch diameter orifice at a differential pressure of one psi.

4.7.A.4 Pressure Suppression Chamber to Drywell Vacuum Breakers

- a. The pressure suppression chamber drywell vacuum breakers shall be visually inspected each refueling outage and checked for operability monthly.
- b. Closed position is indicated by redundant lights in the main control room which are operated by two separate closed position switches and circuits for each vacuum breaker. If either redundant position indicating light is inoperable or shows that the vacuum breaker is stuck open, the affected vacuum breaker shall be exercised within two hours to demonstrate operability of the remaining position indicating circuit, and every 15 days thereafter until the redundant circuit is repaired.
- c. Each pressure suppression chamber to drywell vacuum breaker shall be tested for proper opening differential pressure each refueling outage.
- d. A leak test of the pressure suppression chamber to drywell vacuum breaker system shall be conducted at the end of each refueling outage and every 15 days when reactor operation is continued under the requirements of 3.7.A.4.b.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. DPR-57
GEORGIA POWER COMPANY AND OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
EDWIN I. HATCH NUCLEAR PLANT UNIT 1
DOCKET NO. 50-321

Introduction

By letter dated March 2, 1976, Georgia Power Company (GPC) requested a license amendment for the Edwin I. Hatch Nuclear Plant Unit 1 to modify the Technical Specifications associated with the operability and surveillance requirements for the Pressure Suppression Chamber (TORUS) to Drywell Vacuum Breakers. Certain changes to the original application were made with mutual approval of the NRC staff and GPC.

Discussion

The proposed change would allow extended reactor operation with one of two redundant position indicating circuits inoperable on a torus to drywell vacuum breaker. GPC has proposed additional surveillance requirements during the period that the redundant position indicating circuit is inoperable which would require: (1) performance of a leakage test of the torus to drywell vacuum breaker system and (2) testing the operability of the remaining position indicating circuit. Both of these requirements would be completed immediately upon verifying that the suspect vacuum breaker is closed and every 15 days thereafter until the redundant position indicating circuit is repaired.

Evaluation

The objective of the existing Technical Specification on the operability of the torus to drywell vacuum breakers is to insure that: (1) there are a sufficient number of vacuum breakers operable to limit the differential pressure between the torus and the drywell during a LOCA and (2) that no vacuum breaker can be stuck in a partially open condition such that during a LOCA, steam emanating from the break could enter the torus air space directly and bypass the water volume.

The proposed changes will allow extended reactor operation with a redundant position indicating circuit inoperable, whereas the existing Technical Specifications allow continued reactor operation under these conditions only during the succeeding seven days. The NRC staff considers extended operation of Hatch Unit 1 with a vacuum breaker position indicating circuit inoperable to be acceptable provided that: (1) the total leakage between the drywell and pressure suppression chamber is less than the equivalent leakage through a one-inch diameter orifice as determined by the leak test of the torus to drywell vacuum breaker system, repeated on a 15 day interval and (2) the

operability of the remaining position indicating circuit is verified immediately and every 15 days thereafter. The leakage test requirement will verify that the suspect vacuum breaker is actually closed or will confirm in conjunction with the failed position indication that the vacuum breaker is open. Further, in the event that the leakage test results are unsatisfactory the reactor would be required to be in cold shutdown within 36 hours. Thus the proposed additional surveillance requirements will provide a better indication of the actual position of the valve and will require the reactor to be shutdown within 36 hours if the leakage test results are unsatisfactory.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: MAR 5 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 29 to Facility Operating License No. DPR-57 issued to Georgia Power Company and Oglethorpe Electric Membership Corporation, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit 1, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

The amendment modifies the Technical Specifications associated with the operability and surveillance requirements for the Pressure Suppression Chamber (TORUS) to Drywell Vacuum Breakers.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated March 2, 1976, (2) Amendment No. 29 to License No. DPR-57, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 5th day of March 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter A. Paulson

Walter A. Paulson, Acting Chief
Operating Reactors Branch #3
Division of Operating Reactors

For further details with respect to this action, see (1) the application for amendment dated March 2, 1976, (2) Amendment No. 29 to License No. DPR-57, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513.

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Dated at Bethesda, Maryland, this 5th day of March, 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

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Walter A. Paulson, Acting Chief
Operating Reactors Branch #3
Division of Operating Reactors

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UNITED STATES NUCLEAR REGULATORY COMMISSION

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