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Docket No., 50-321

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Georgia Power Company
Oglethorpe Electric Membership Corporation
ATTN: Mr. I. S. Mitchell, III
Vice President and Secretary
Georgia Power Company
Atlanta, Georgia 30302

Gentlemen:

The Commission has issued the enclosed Amendment No. 20 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit 1. The amendment also incorporates Change No. 20 in the Technical Specifications in accordance with your application dated September 11, 1975.

This amendment modifies the Technical Specifications to revise the high steam flow trip setting associated with the High Pressure Coolant Injection (HPCI) system.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

131

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Enclosures:

1. Amendment No. 20
2. Safety Evaluation
3. Federal Register Notice

cc w/encls:
See next page

OFFICE ➤	RL:ORB-3	RL:ORB-3	OELD	RL:ORB-3		
x7872 SURNAME ➤	CParrish <i>cp</i>	JGuibert:esp <i>mullman</i>	<i>mullman</i>	GLear <i>WAL</i>		
DATE ➤	10/15/75	10/16/75	10/20/75	10/21/75		

Georgia Power Company &
Oglethorpe Electric Membership Corporation

cc: w/enclosures

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Edwin I. Hatch Plant
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Mr. G. Wyman Lamb, Chairman
Appling County Commissioners
County Courthouse
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Mr. John Robins
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Atlanta, Georgia 30334

Mr. Dave Hopkins
U. S. Environmental Protection Agency
Region IV Office
1421 Peachtree Street, N. E.
Atlanta, Georgia 30309

Mrs. Fleets Taylor, Librarian
Appling County Public Library
Parker Street
Baxley, Georgia 31513

. For further details with respect to this action, see (1) the application for amendment dated September 11, 1975, (2) Amendment No. 20 to License No. DPR-57, with Change No. 20 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this *21st* day of *October*, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

151

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

OFFICE ➤	RL:ORB-3	RL:ORB-3	OELD	RL:ORB-3		
x7872 SURNAME ➤	CParrish <i>CP</i>	JGuibert:esp <i>JG</i>	<i>mulman</i>	Glear <i>Glear</i>		
DATE ➤	10/15/75	10/15/75	10/20/75	10/ /75		

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 20 to Facility Operating License No. DPR-57 issued to Georgia Power Company and Oglethorpe Electric Membership Corporation, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit 1, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

The amendment modifies the Technical Specifications to revise the high steam flow trip setting associated with the High Pressure Coolant Injection (HPCI) system.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 20
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company and Oglethorpe Electric Membership Corporation (the licensees) dated September 11, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility License No. DPR-57 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 20".

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for *Walter A. Paulson*
George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Attachment:
Change No. 20 to the
Technical Specifications

Date of Issuance: OCT 24 1975

ATTACHMENT TO LICENSE AMENDMENT NO. 20
CHANGE NO. 20 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-57
DOCKET NO. 50-321

Replace page 3.2-6 with the attached revised pages. No change has been made on page 3.2-5.

Table 3.2-2

INSTRUMENTATION WHICH INITIATES OR CONTROLS HPCI

Ref. No. (a)	Instrument	Trip Condition Nomenclature	Required Operable Channels per Trip System(b)	Trip Setting	Remarks
1	Reactor Water Level (Yarway)	Low Low (LL2)	2	≥ -38 inches	Initiates HPCI; also initiates RCIC and selects unbroken recirculation loop for LPCI.
2	Drywell Pressure	High	2	≤ 2 psig	Initiates HPCI; also initiates LPCI and Core Spray and provides a permissive signal to ADS.
3	HPCI Turbine Overspeed	Mechanical	1	≤ 5000 rpm	Trips HPCI turbine
4	HPCI Turbine Exhaust Pressure	High	1	≤ 150 psig	Trips HPCI turbine
5	HPCI Pump Suction Pressure	Low	1	≤ 15 " Hg vacuum	Trips HPCI turbine
6	Reactor Water Level (Narrow Range)	High	2	$\leq +58$ inches	Trips HPCI turbine
7	HPCI System Flow (Flow Switch)	High	1	> 800 gpm	Closes HPCI minimum flow bypass line to suppression chamber.
		Low	1	≤ 500 gpm	Opens HPCI minimum flow bypass line if pressure permissive is present.
8	HPCI Equipment Room Temperature	High	1	$\leq 90^\circ\text{F} + \text{ambient}$	Closes isolation valves in HPCI system, trips HPCI turbine.
9	HPCI Equipment Room Differential Temperature	High	1	$\leq 50^\circ\text{F}$	Closes isolation valves in HPCI system, trips HPCI turbine.

3.2-5

Table 3.2-2 (Cont.)

Ref. No. (a)	Instrument	Trip Condition Nomenclature	Required Operable Channels per Trip Systems(b)	Trip Setting	Remarks
10	HPCI Steam Line Pressure	Low	2	≥ 100 psig	Closes isolation valves in HPCI system, trips HPCI turbine.
11	HPCI Steam Line ΔP (Flow)	High	1	$< 216''$ water (300% Flow)	Close isolation valves in HPCI system, trips HPCI turbine
12	HPCI Turbine Exhaust Diaphragm Pressure	High	1	≤ 10 psig	Close isolation valves in HPCI system, trips HPCI turbine
13	Suppression Chamber Area Air Temperature	High	1	$\leq 90^\circ\text{F} + \text{ambient}$	Close isolation valves in HPCI system, trips HPCI turbine.
14	Suppression Chamber Area Differential Air Temperature	High	1	To be determined during startup	Close isolation valves in HPCI system, trips HPCI turbine.
15.	Condensate Storage Tank Level	Low	2	≥ 0 inches	Automatic interlock switches suction from CST to suppression chamber.
16	Suppression Chamber Water Level	High	2	≤ 0 inches	Automatic interlock switches suction from CST to suppression chamber.
17	HPCI Logic Power Failure Monitor		1	Not Applicable	Monitors availability of power to logic system.

a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in Table 3.2-2 and items in Table 4.2-2.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 20 TO FACILITY OPERATING LICENSE NO. DPR-57

(CHANGE NO. 20 TO TECHNICAL SPECIFICATIONS)

EDWIN I. HATCH NUCLEAR PLANT UNIT 1

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

DOCKET NO. 50-321

Introduction

By letter dated September 11, 1975, Georgia Power Company (GPC) requested a change to the Technical Specifications, Appendix A to Facility Operating License No. DPR-57 for Edwin I. Hatch Nuclear Plant Unit 1. The proposed change will revise the high steam flow trip setting associated with the High Pressure Coolant Injection (HPCI) system.

Discussion

Steam flow in the HPCI turbine steam line is sensed by two differential pressure instruments, each of which monitors the differential pressure across a piping elbow in the steam line. High steam flow, as detected by pressure switches associated with the differential pressure instruments, could indicate a break in the HPCI turbine steam line. Actuation of the pressure switches at a differential pressure corresponding to 300% rated steam flow initiates automatic closure of the HPCI steam line isolation valves to prevent excessive loss of reactor coolant inventory and to minimize the release of radioactive materials. The high steam flow trip setting was selected at a flow rate high enough to avoid spurious isolations and low enough to provide timely detection of a HPCI turbine steam line break.

Evaluation

steam line instrument differential pressure at 100% rated steam flow. Actual testing of the HPCI turbine with 100% rated steam flow has shown that the value which was assumed for the differential pressure at 100% steam flow is inaccurate. Consequently, the HPCI high steam flow trip setting is also in error and requires revision. GPC has proposed a new trip setting of 216" H₂O differential pressure; this proposed setting is based upon the extrapolation of actual test data to a differential pressure which corresponds to 300% rated HPCI turbine steam flow.

We conclude that the proposed change is acceptable on the basis that it establishes a trip setting which corresponds more exactly to the design criterion of 300% rated HPCI turbine steam flow.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the changes does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: OCT 2 4 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

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The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated September 11, 1975, (2) Amendment No. 20 to License No. DPR-57, with Change No. 20 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 21st day of October, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter A. Paulson

Walter A. Paulson, Acting Chief
Operating Reactors Branch #3
Division of Reactor Licensing