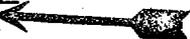


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 V. A. Moore
 D. Skovholt
 R. Klecker
 M. Jinks (4 encls)
 R. Vollmer
 S. Kari
 W. Miller, DR:AO
 S. Burwell
 H. Gearin
 LWR 2 Branch Chiefs

NOV 15 1974

Docket No. 50-321

Georgia Power Company
 ATTN: Mr. I. S. Mitchell, III
 Vice President & Secretary
 P. O. Box 4545
 Atlanta, Georgia 30302

Gentlemen:

The Atomic Energy Commission has issued the enclosed Amendment No. 5 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit 1. This amendment includes Change No. 6 to the Technical Specifications and is in response to your request of November 14, 1974.

The amendment changes the Technical Specification to specify the limiting flow rate trip setting of 300% flow rather than specify a Ap set point on the trip initiating instrumentation that would correspond to 300% of the design steam flow rate.

Copies of the related Safety Evaluation and Federal Register notice are also enclosed.

Sincerely,

Original signed by
 Voss A. Moore

Voss A. Moore, Assistant Director
 for Light Water Reactors, Group 2
 Directorate of Licensing

Enclosures:

1. Amendment No. 5 to DPR-57
w/Change No. 6 to the
Technical Specifications
2. Safety Evaluation
3. Federal Register Notice

cc: (See next page)

LB

OFFICE	L:GCR	L:LWR 2-1	L:LWR 2			
SURNAME	HGearin:aw	JStoib	VAMoore			
DATE	11/15/74	11/15/74	11/15/74			

NOV 15 1974

cc: Mr. Ruble A. Thomas
Vice President
Southern Services, Inc.
300 Office Park
Birmingham, Alabama 35202

George F. Trowbridge, Esquire
Shaw, Pittman, Fotts & Trowbridge
910 Seventeenth Street
Washington, D. C. 20006

Mr. Harry Majors
Southern Services, Inc.
300 Office Park
Birmingham, Alabama 35202

Mr. D. P. Shannon
Georgia Power Company
Edwin I. Hatch Nuclear Plant
P. O. Box 442
Baxley, Georgia 31513

Mr. John Robins
Office of Planning and Budget
Room 615-C
270 Washington Street, S. W.
Atlanta, Georgia 30334

Mr. G. Wyman Lamb, Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513

Mr. Dave Hopkins
Environmental Protection Agency
1421 Peachtree Street, N. E.
Atlanta, Georgia 30309

bcc: J. R. Buchanan, ORNL
Thomas B. Abernathy, DTIE
ACRS (16)

OFFICE▶						
SURNAME▶						
DATE▶						



UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, D.C. 20545

GEORGIA POWER COMPANY

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 5
License No. DPR-57

1. The Atomic Energy Commission (the Commission) has found that:
 - A. The application for amendment by the Georgia Power Company (the licensee) dated November 14, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended, and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

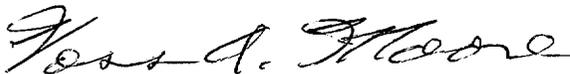
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility License No. DPR-57 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 6."

3. This license amendment is effective as of the date of its issuance.

FOR THE ATOMIC ENERGY COMMISSION



Voss A. Moore, Assistant Director
for Light Water Reactors, Group 2
Directorate of Licensing

Attachment:
Change No. 6 to the
Technical Specifications

Date of Issuance: NOV 15 1974

GEORGIA POWER COMPANY

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 5
License No. DPR-57

1. The Atomic Energy Commission (the Commission) has found that:
 - A. The application for amendment by the Georgia Power Company (the licensee) dated November 14, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended, and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.
-

- 2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility License No. DPR-57 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 6."

- 3. This license amendment is effective as of the date of its issuance.

FOR THE ATOMIC ENERGY COMMISSION

Original signed by
Voss A. Moore

Voss A. Moore, Assistant Director
for Light Water Reactors, Group 2
Directorate of Licensing

Attachment:
Change No. 6 to the
Technical Specifications

Date of Issuance: NOV 15 1974

OFFICE ▶	L:LWR 2-1	L:LWR 2			
SURNAME ▶	JFStolz	VAMoore			
DATE ▶	11/15/74	11/15/74			

Table 3.2-3 (Cont.)

Ref. No. (a)	Instrument	Trip Condition Nomenclature	Required Operable Channels per Trip System (b)	Trip Setting	Remarks
6 10	RCIC Steam Line Flow (Upstream and Downstream Elbow Taps)	High	1	$\leq 300\%$ Flow	Closes isolation valves in RCIC system, trips RCIC turbine
11	RCIC Turbine Exhaust Diaphragm Pressure	High	1	≤ 10 psig	Closes isolation valves in RCIC system, trips RCIC turbine
12	Suppression Chamber Area Air Temperature	High	1	$\leq 90^\circ\text{F} + \text{ambient}$	Closes isolation valves in RCIC system, trips RCIC turbine
13	Suppression Chamber Area Differential Air Temperature	High	1	To be determined during startup	Closes isolation valves in RCIC system, trips RCIC turbine
14	RCIC Logic Power Failure Monitor		1	Not Applicable	Monitors availability of power to logic system

3.2-9

- a. The column entitled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in Table 3.2-3 and items in Table 4.2-3.
- b. Whenever any CCCS subsystem is required to be operable by Section 3.5, there shall be two operable trip systems. If the required number of operable channels cannot be met for one of the trip systems, that system shall be repaired or the reactor shall be placed in the Cold Shutdown Condition within 24 hours after this trip systems is made or found to be inoperable.

3.2.C.9. RCIC Steam Line Pressure Low (Continued)

lation setpoint of ≥ 50 psig is chosen at a pressure below that at which the RCIC turbine can effectively operate.

6 | 10. RCIC Steam Line Flow (High)

RCIC turbine high steam flow could indicate a break in the RCIC turbine steam line. The automatic closure of the RCIC steam line isolation valves prevents the excessive loss of reactor coolant and the release of significant amounts of radioactive materials from the nuclear system process barrier. Upon detection of RCIC turbine high steam flow the RCIC turbine steam line is isolated. The high steam flow trip setting of 300% flow was selected high enough to avoid spurious isolation, i.e., above the high steam flow rate encountered during turbine starts. The setting was selected low enough to provide timely detection of an RCIC turbine steam line break.

11. RCIC Turbine Exhaust Diaphragm Pressure High

High pressure in the RCIC turbine exhaust could indicate that the turbine rotor is not turning, thus allowing reactor pressure to act on the turbine exhaust line. The RCIC steam line isolation valves are automatically closed to prevent overpressurization of the turbine exhaust line. The turbine exhaust diaphragm pressure trip setting of ≤ 10 psig is selected high enough to avoid isolation of the RCIC if the turbine is operating, yet low enough to effect isolation before the turbine exhaust line is unduly pressurized.

12. Suppression Chamber Area Air Temperature High

- As in the RCIC equipment room, and for the same reason, a temperature of 90 F + ambient will initiate a timer to isolate the RCIC turbine steam line.

13. Suppression Chamber Area Differential Air Temperature High

As for the RCIC equipment room differential temperature, and for the same reason, a high differential air temperature between the inlet and outlet ducts which ventilate the suppression chamber area will also initiate a timer to isolate the RCIC turbine steam line.

14. RCIC Logic Power Failure Monitor

The RCIC Logic Power Failure Monitor monitors the availability of power to the logic system. In the event of loss of availability of power to the logic system, an alarm is annunciated in the control room.

SAFETY EVALUATION BY THE DIRECTORATE OF LICENSING

SUPPORTING AMENDMENT NO. 5 TO LICENSE NO. DPR-57

CHANGE NO. 6 TO THE TECHNICAL SPECIFICATIONS

GEORGIA POWER COMPANY

EDWIN I. HATCH NUCLEAR PLANT UNIT 1

DOCKET NO. 50-321

1.0 INTRODUCTION

By letter dated November 14, 1974, Georgia Power Company (the licensee) requested changes to the Technical Specifications appended to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant, Unit 1 (Hatch 1). The requested change involves the high flow trip set point for the RCIC Steam Line. The licensee requests that the high flow trip points now specified in the Technical Specifications be adjusted to reflect the actual measurements made during start-up testing of the RCIC system that correspond to the 300% of design steam flow.

2.0 DISCUSSION

In his letter of November 14, 1974, the licensee points out that the present Technical Specifications require that the isolation valves in the RCIC system be closed and that the RCIC turbine be tripped if the RCIC steam line flow exceeds 300% of the design flow. This flow is measured by pressure sensors which measure the pressure drop (Δp) at an elbow in the steam line. If the pressure drop exceeds the preset amount, RCIC isolation valve closure and turbine trip are automatically initiated. The present Technical Specification set point limitation calls for a Δp of (for 300% flow) less than or equal 90 inches of water. Measurements have been made during start-up testing of the RCIC system which show that the Δp for 300% flow is actually 182 inches of water at the upstream elbow tap and 159 inches of water at the downstream elbow tap. The licensee requested that the trip setting be changed from 90 inches to the above stated trip settings. The licensee further states that the reason for allowing 300% flow is to assure that the system can undergo a fast start. The present Δp set point makes the reliability of an RCIC system fast start questionable.

We conclude that the 90-inch water Δp limit currently specified in the Technical Specifications is incorrect. The bases of the Δp setting as described on page 3.2-17 of the Technical Specification is to limit the RCIC high steam flow to 300% of the design flow. We conclude that the licensee's proposed change in Δp settings will not change the current Technical Specification high steam flow limits for the RCIC system. However, we believe that the Technical Specifications should only specify the RCIC high steam flow limiting condition of 300% flow. The Δp settings corresponding to this limit and the calibration procedures for establishing the Δp settings should be the responsibility of the licensee. Such calibration procedures are available for NRC review. The changes in Technical Specifications are shown by revision marks on pages 3.2-9 and 3.2-57 of Appendix A attached to the license amendment.

3.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Spottswood B. Burwell

Spottswood B. Burwell, Project Manager
Light Water Reactors Branch 2-1
Directorate of Licensing

John F. Stolz

John F. Stolz, Chief
Light Water Reactors Branch 2-1
Directorate of Licensing

Dated: NOV 15 1974

UNITED STATES ATOMIC ENERGY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U. S. Atomic Energy Commission (the Commission) has issued Amendment No. 5 to Facility Operating License No. DPR-57 issued to the Georgia Power Company which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant Unit 1, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

The amendment changes the Technical Specification to specify the limiting flow rate trip setting of 300% rather than specify a Δp set point on the trip initiating instrumentation that would correspond to 300% of the design steam flow rate.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

For further details with respect to this action, see (1) the application for amendment dated November 14, 1974, (2) Amendment No. 5 to License No. DPR-57, with any attachments, and (3) the

Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Atomic Energy Commission, Washington, D. C. 20545, Attention: Deputy Director for Reactor Projects, Directorate of Licensing - Regulation.

Dated at Bethesda, Maryland, this 15th day of November, 1974.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by
John F. Stolz

John F. Stolz, Chief
Light Water Reactor Projects Branch 2-1
Directorate of Licensing