

May 3, 1995

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
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SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M90634)

Dear Mr. Opeka:

The Commission has issued the enclosed Amendment No. 187 to Facility Operating License No. DPR-65 for the Millstone Nuclear Power Station, Unit No. 2, in response to your application dated October 18, 1994, supplemented February 21, 1995.

The amendment changes Surveillance Requirement 4.6.1.2.a (Overall Integrated Containment Leakage Rate Tests) by revising the surveillance interval for Type A tests from 40 ± 10 months to approximately equal intervals during each 10-year inservice period. The amendment also removes a note that expired upon completion of Cycle 11 refueling outage.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Guy S. Vissing, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures: 1. Amendment No. 187 to DPR-65
2. Safety Evaluation

cc w/encs: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, reading "Guy S. Vissing", is written over a horizontal line.

Guy S. Vissing, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-336

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cc w/encls: See next page

Mr. John F. Opeka
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Millstone Nuclear Power Station
Unit 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY
THE CONNECTICUT LIGHT AND POWER COMPANY
THE WESTERN MASSACHUSETTS ELECTRIC COMPANY
DOCKET NO. 50-336
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 187
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated October 18, 1994, supplemented February 21, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 187, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: May 3, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 187

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 6-2

Insert

3/4 6-2

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of $\leq L_a$, 0.50 percent by weight of the containment air per 24 hours at P_a , 54 psig.
- b. A combined leakage rate of $\leq 0.60 L_a$ for all penetrations and valves subject to Type B and C tests when pressurized to P_a .
- c. A combined leakage rate of $\leq 0.017 L_a$ for all penetrations identified in Table 3.6-1 as secondary containment bypass leakage paths when pressurized to P_a .

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60 L_a$, or (c) with the combined bypass leakage rate exceeding $0.017 L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50.

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at approximately equal intervals during shutdown at P_a (54 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 187

TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By letter dated October 18, 1994, as supplemented February 21, 1995, the Northeast Nuclear Energy Company (the licensee) submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 2 Technical Specifications (TSs). The proposed amendment would require three Type A overall Integrated Containment Leakage Tests be conducted at approximately equal intervals during shutdowns during each 10-year service period. For the third Type A test for the second 10-year period, it would be conducted during the thirteenth refueling outage extending the second 10-year service period to the end of the thirteenth refueling outage. The amendment would also change the Containment Leakage Bases by reflecting the conditions of a proposed exemption to 10 CFR 50, Appendix J, that would remove the requirement that the third Type A test for each 10-year period be conducted when the plant is shutdown for the 10-year plant inservice inspection. The February 21, 1995, letter withdrew the action related to conducting the third Type A test for the second 10-year period during the thirteenth refueling outage and the reference to a proposed exemption to 10 CFR 50, Appendix J, that would remove the requirement that the third Type A test for each 10-year period be conducted when the plant is shutdown for the 10-year plant inservice inspection. The referenced proposed exemption was withdrawn by another letter dated February 21, 1995.

2.0 EVALUATION

The second 10-year inservice inspection period is between June 1985 and June 1995. The 10-year inservice inspection will be performed in March 1995 during shutdown for the twelfth refueling outage as required by the current TS. Allowing the tests to be conducted at approximate equal intervals during the next 10-year inspection period would place the next test to be conducted at an

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interval of 2-3 months greater than equal intervals as the proposed amendment would specify. This is considered not significant considering the following:

Type A tests are performed to ensure that the total leakage from containment does not exceed the maximum allowable primary containment leakage rate at the design pressure. This assures compliance with the dose limits of 10 CFR Part 100.

The proposed change to Surveillance Requirement 4.6.1.2.a of the Millstone Unit No. 2 TSs will increase the flexibility for scheduling the Type A tests. It does not modify the maximum allowable leakage rate at the design containment pressure, does not impact the design basis of the containment, and does not make any physical or operational changes to existing plant structures, systems, or components. Further, the proposed change is consistent with the requirement in 10 CFR Part 50, Appendix J.

Historically, Type A tests have a relatively low failure rate where Type B and C testing (local leakage rate tests) could not detect the leakage path. Most Type A test failures are attributed to failures of Type B or C components (containment penetrations and isolation valves). Type B and C components are tested per Surveillance Requirement 4.6.1.2.d of the Millstone Unit No. 2 TSs. These tests are required to be conducted at intervals no greater than 24 months. These local leakage rate tests provide assurance that containment integrity is maintained. The Type B and C tests will continue to be performed in accordance with the requirements of Surveillance Requirement 4.6.1.2.d.

The previous Type A, B, and C tests demonstrate that Millstone Unit No. 2 has maintained control of containment integrity by maintaining a conservative margin between the acceptance criterion and the "As-Found" and "As-Left" leakage results. Based on this, the Millstone Unit No. 2 containment is considered to be in sound condition.

Based on the above, the staff has determined the proposed changes to the TSs to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 16191). Accordingly, the amendment meets the eligibility criteria for

categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Vissing

Date: May 3, 1995