

MAR 5 1975

Docket No. 50-321

Georgia Power Company
Oglethorpe Electric Membership
Corporation
ATTN: I. S. Mitchell, III
Vice President and Secretary
Georgia Power Company
Atlanta, Georgia 30302

Gentlemen:

The Commission has issued the enclosed Amendment No. 9 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit 1. The amendment incorporates Change No. 8 to the Technical Specifications and is in accordance with your request of December 2, 1974.

The amendment permits a revision to the Technical Specifications relating to the surveillance requirement for testing of personnel air lock to the primary reactor containment.

A copy of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

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George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Enclosures:

- 1. Amendment No. 9
- 2. Safety Evaluation
- 3. Federal Register Notice

cc: See next page

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I. S. Mitchell, III

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cc: w/enclosures

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company & Oglethorpe Electric Membership Corporation (the licensees) dated December 2, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(1) of Facility License No. DPR-57 is hereby amended to read as follows:



"(1) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 8.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Karl R. Goller

Karl R. Goller, Assistant Director
for Operating Reactors
Division of Reactor Licensing

Attachment:
Change No. 8 to the
Technical Specifications

Date of Issuance: MAR 5 1975

ATTACHMENT TO LICENSE AMENDMENT NO. 9
CHANGE NO.8 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-57
DOCKET NO. 50-321

Replace pages 3.7-5 and 3.7-6 with the attached revised pages.
(No changes have been made on page 3.7-5). Add page 3.7-6a (continuation
only of page 3.7-6.)

4.7.A.2.b(2)

- (c) The acceptance criteria for subsequent peak pressure tests shall require the new L_{am} not to exceed L_a .
- (d) The allowable operational leak rate, L_{aO} , which shall be met prior to resumption of power operation following a test (either as measured or following repairs and retest) shall not exceed $0.75 L_a$.

c. Corrective Action for Type A Tests

If leak repairs are necessary to meet the allowable operational leak rate, the integrated leak rate test need not be repeated provided local leakage measurements are conducted and the leak rate differences prior to and after repairs, when corrected to the test pressure and deducted from the integrated leak rate measurements, yield a leak rate value not in excess of the allowable operational leak rate.

d. Frequency for Type A Tests

After the initial preoperational leak rate test, two integrated leak rate tests shall be performed at approximately equal intervals between the major shutdowns for inservice inspection conducted at ten-year intervals. In addition, an integrated leak rate test shall be performed at the end of the ten-year interval, which may coincide with the inservice inspection shutdown period.

e. Type B Tests - Leak Tests of Penetrations with Seals and Bellows
(Tables 3.7-2 and 3.7-3)

- (1) Primary containment components which seal or penetrate the pressure containing boundary of the containment shall be tested at a pressure not less than P_a . These components shall be tested at each major refueling shutdown or at intervals not to exceed two years.

4.7.A.2.e. Type B Tests - Leak Tests of
Seals and Penetrations
(Table 3.7-2 and 3.7-3)
(Continued).

- (2) (a) The personnel air lock shall be tested at intervals not to exceed six months at P_a by pressurizing the compartment between the two air lock doors.

During intervals of door use when containment integrity is required, the door seals shall be tested at 10 psig after each opening.

- (b) Personnel air lock leakage shall not exceed $0.05 L_a$.

f. Type C Tests-Local Leak Tests
of Containment Isolation Valves
(Table 3.7-1 and 3.7-4)

Containment isolation valves (except for main steam line isolation valves) shall be tested at a pressure not less than P_a . Type C tests shall be performed at each major refueling shutdown or at intervals not to exceed two years.

g. Acceptance Criteria for Type B
and Type C Tests

The Combined leakage rate of components subject to Type B and C tests (except for main steam line isolation valves) shall not exceed $0.6 L_a$.

h. Main Steam Line Isolation Valves

The main steam line isolation valves shall be tested at a pressure of 28 psig for leakage at least once per operating cycle. If a total leak rate of 11.5 scf per hour for any one main steam line isolation valve is exceeded, repairs and retest shall be performed to correct this condition.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 9 TO LICENSE NO. DPR-57

(CHANGE NO. 8 TO THE TECHNICAL SPECIFICATIONS)

GEORGIA POWER COMPANY

EDWIN I HATCH NUCLEAR PLANT UNIT 1

DOCKET NO. 50-321

Introduction

By correspondence dated December 2, 1974, Georgia Power Company requested changes to the Technical Specifications appended to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit 1 (Hatch 1). Georgia Power Company requested that the surveillance requirement for testing of the personnel air lock to the reactor containment be modified.

Discussion

Present technical specifications require that the personnel air lock to the primary reactor containment be tested at a peak design pressure of 59 psig after each opening and at intervals not to exceed six months. In addition, present technical specifications require testing of the air lock door seals at 10 psig once per operating cycle. Air lock testing after each opening is extremely cumbersome and time consuming, requiring on the order of 12 hours per test (2 hours preparation and 10 hours of pressure testing).

The applicant has proposed that the testing program for the personnel air lock be modified by increasing the emphasis on testing the air lock door seals. Specifically, the applicant proposes (1) that the personnel air lock be tested at intervals not to exceed six months (deleting the provision for testing after each opening) by pressurizing the compartment between the doors and (2) that the air lock door seals be tested once per operating cycle (per current technical specifications) and during periods of door use, when containment integrity is required, testing the door seals after each opening or every three days whichever is longer.

Evaluation

As a result of our review and with the acquiescence of the licensee, we have deleted the licensee's proposal for a three day interval for door seal testing after opening of the personnel air lock and the requirement that the door seals be tested once per operating cycle. The latter



requirement is superseded by the requirement for door seal testing after each opening. This modification of the licensee's proposal conforms with the requirements specified in 10 CFR 50, Appendix J, "Primary Containment Leakage Testing for Water-Cooled Power Reactors".

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: MAR 5 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 9 to Facility Operating License No. DPR-57 issued to Georgia Power Company & Ogelthorpe Electric Membership Corporation (the licensees) which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit 1, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

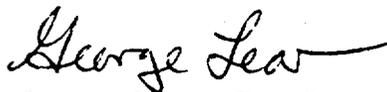
The amendment permits revision to the Technical Specifications relating to the surveillance requirement for testing of the personnel air lock to the reactor containment.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated December 2, 1974, (2) Amendment No. 9 to License No. DPR-57, with Change No. 8, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this MAR 5 1975

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing