

APR 17 1975

Docket No. 50-321

Georgia Power Company
Oglethorpe Electric Membership Corporation
ATTN: I. S. Mitchell, III
Vice President & Secretary
Georgia Power Company
Atlanta, Georgia 30302

Gentlemen:

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The Commission has issued the enclosed Amendment No. 10 to Facility Operating License No. DPR-57 for the Edwin I. Hatch Nuclear Plant Unit 1. The amendment incorporates Change No. 10 in the Technical Specifications in accordance with your request of January 28, 1975, and the requirements of our December 27, 1974 Order.

The amendment modifies the Technical Specifications to allow an increase in the rates for maximum average planar linear heat generation over the range from 0 to 5000 MWD/T.

A copy of the related Safety Evaluation and Federal Register Notice are also enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Enclosures:

- 1. Amendment No. 10
- 2. Safety Evaluation
- 3. Federal Register Notice

cc: See next page

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Georgia Power Company &
Oglethorpe Electric Membership Corporation

cc: w/enclosures

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Mr. G. Wyman Lamb, Chairman
Appling County Commissioners
County Courthouse
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Mr. John Robins
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company and Oglethorpe Electric Membership Corporation (the licensees) dated January 28, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulation; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(1) of Facility License No. DPR-57 is hereby amended to read as follows:

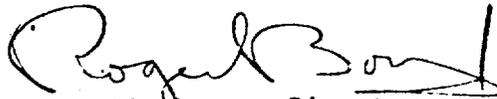


"(1) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 10."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Giambusso, Director
Division of Reactor Licensing
Office of Nuclear Reactor Regulations

Attachment:
Change No. 10
Technical Specifications

Date of Issuance: APR 17 1975

ATTACHMENT TO LICENSE AMENDMENT NO. 10
CHANGE NO. 10 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-57
DOCKET NO. 50-321

Replace pages 3.11-1 and 3.11-2, and 3.11-3. (No changes have been made on 3.11-1).

Replace Figure 3.11-1 with the attached Figure.

3.11 FUEL RODS

Applicability

The Limiting Conditions for Operation associated with the fuel rods apply to those parameters which monitor the fuel rod operating conditions.

Objective

The Objective of the Limiting Conditions for Operation is to assure the performance of the fuel rods.

SpecificationsA. Average Planar Linear Heat Generation Rate (APLHGR)

During steady state power operation, the APLHGR for each type of fuel as a function of average planar exposure shall not exceed the limiting value shown in Figure 3.11-1. If at any time during steady state power operation it is determined that the limiting value for APLHGR is being exceeded, action shall be taken immediately to restore operation to within the prescribed limits.

B. Linear Heat Generation Rate (LHGR)

During steady state power operation, the LHGR as a function of core height shall not exceed the limiting value shown in Figure 3.11-2. If at any time during steady state power operation it is determined that the limiting value for LHGR is being exceeded, action shall be taken immediately to restore operation to within the prescribed limits.

C. Minimum Critical Heat Flux Ratio (MCHFR)

During steady state power operation, MCHFR shall be ≥ 1.9 at rated power and flow. If at any time during steady state power operation it is determined that the limiting value for MCHFR is being exceeded, action shall be taken immediately to restore operation within the prescribed limits.

4.11 FUEL RODS

Applicability

The Surveillance Requirements apply to the parameters which monitor the fuel rod operating conditions.

Objective

The Objective of the Surveillance Requirements is to specify the type and frequency of surveillance to be applied to the fuel rods.

SpecificationsA. Average Planar Linear Heat Generation Rate (APLHGR)

The APLHGR for each type of fuel as a function of average planar exposure shall be determined daily during reactor operation at $\geq 25\%$ rated thermal power.

B. Linear Heat Generation Rate (LHGR)

The LHGR as a function of core height shall be checked daily during reactor operation at $\geq 25\%$ rated thermal power.

C. Minimum Critical Heat Flux Ratio (MCHFR)

MCHFR shall be determined daily during reactor power operation at $\geq 25\%$ rated thermal power and following any change in power level or distribution that would cause operation with a limiting control rod pattern as described in the bases for Specification 3.3.F.

3.11 FUEL RODS

A. Average Planar Linear Heat Generation Rate (APLHGR)

This specification assures that the peak cladding temperature following the postulated design basis loss-of-coolant accident will not exceed the 2300 F limit specified in the Interim Acceptance Criteria (IAC) issued in June 1971 even considering the postulated effects of fuel pellet densification.

The peak cladding temperature following a postulated loss-of-coolant accident is primarily a function of the average heat generation rate of all the rods of a fuel assembly at any axial location and is only dependent secondarily on the rod to rod power distribution within an assembly. Since expected local variations in power distribution within a fuel assembly affect the calculated peak clad temperature by less than ± 20 F relative to the peak temperature for a typical fuel design, the limit on the average linear heat generation rate is sufficient to assure that calculated temperatures are within the IAC limit.

The limiting value for APLHGR is shown in Figure 3.11-1. That portion of the limits from 0 to 5000 MWD/T was verified using the IAC models and GEGAP III pellet-to-cladding gage conductances but is limited as shown to the values specified in Appendix A to the NRC Order of December 27, 1974. The limiting values for APLHGR for exposures greater than 5000 MWD/T is the smaller value of either (1) those specified in Appendix A to the NRC Order of December 27, 1974, or (2) those shown on the curves labeled "(gamma)" on Figures 4-9R1 and 4-9R2 of the GE topical report "Fuel Densification Effects on General Electric Boiling Water Reactor Fuel, "NEDM-10735, Supplements 6, 7, 8, August 1973, and is the result of the calculations presented in Section 4.3.4 of the same report. These calculations were made to determine the effect of densification on peak clad temperature and were performed in accordance with the AEC fuel Densification Model for BWR's which is attached to NEDM-10735, Supplement 6, 7, and 8 as Appendix B.

The possible effects of fuel pellet densification were: (1) creep collapse of the cladding due to axial gap formation; (2) increase in the LHGR because of pellet column shortening; (3) power spikes due to axial gap formation; and (4) changes in stored energy due to increased radial gap size. Calculations show that the clad collapse is conservatively predicted not to occur currently or during the next power operation cycle. Therefore, clad collapse is not considered in the analyses. Since axial thermal expansion of the fuel pellets is greater than axial shrinkage due to densification the analyses of peak clad temperature do not consider any change in LHGR due to pellet column shortening. Although the formation of axial gaps might produce a local power spike at one location on any one rod in a fuel assembly, the increase in local power density would be on the order of only 2% of the axial midplane. Since small local variations in power distribution have a small effect on peak clad temperature, power spikes were not considered in the analysis of loss-of-coolant accidents. Changes in gap size affect the clad temperature by their effect on pellet-clad thermal conductance and fuel pellet stored energy. The pellet-clad thermal conductance assumed for each rod is dependent on the steady state operating linear heat generation rate and the gap size. As specified in the AEC Fuel Densification Model for BWR's, the gap size was calculated assuming that the pellet densified from the measured pellet density to 96.5% of theoretical density. For the most critical rod, the two standard deviation lower bound on initial pellet density was assumed. For the other 48 rods the two standard deviation lower bound on the initial mean "boat" pellet density was assumed.

3.11.A. Average Planar Linear Heat Generation Rate (APLHGR) (Continued)

The curves used to determine pellet-clad thermal conductance as a function of linear heat generation are based on experimental data and predict with a 95% confidence that 90% of the population exceed the predictions.

B. Linear Heat Generation Rate (LHGR)

This specification assures that the linear heat generation rate in any rod is less than the design linear heat generation if fuel pellet densification is postulated. The power spike penalty specified is based on the analysis presented in Section 3.2.1 of the GE topical report NEDM-10735, Supplements 6, 7, and 8, and assumes a linearly increasing variation in axial gaps between core bottom and top, and assures with a 95% confidence, that no more than one fuel rod exceeds the design linear heat generation rate due to power spiking. The LHGR as a function of core height shall be checked daily during reactor operation at $\geq 25\%$ power to determine if fuel burn-up, or control rod movement has caused changes in power distribution. For LHGR to be a limiting value below 25% rated thermal power, the MTPF would have to be greater than 10 which is precluded by a considerable margin when employing any permissible control rod pattern.

C. Minimum Critical Heat Flux Ratio (MCHFR)

At core thermal power levels less than or equal to 25%, the reactor will be operating at minimum recirculation pump speed and the moderator void content will be very small. For all designated control rod patterns which may be employed at this point, operating plant experience and thermal hydraulic analysis indicated that the resulting MCHFR value is in excess of requirements by a considerable margin. With this low void content, any inadvertent core flow increase would only place operation in a more conservative mode relative to MCHFR. During initial start-up testing of the plant, a MCHFR evaluation will be made at the 25% thermal power level with minimum recirculation pump speed. The MCHFR margin will thus be demonstrated such that future MCHFR evaluations below this power level will be shown to be unnecessary. The daily requirement for calculating MCHFR above 25% rated thermal power is sufficient since power distribution shifts are very slow when there have not been significant power or control rod changes. The requirement for calculating MCHFR when a limiting control rod pattern is approached ensures that MCHFR will be known following a change in power or power shape (regardless of magnitude) that could place operation at a thermal limit.

D. References

1. HNP-2 PSAR Appendix I, Conformance to AEC Interim Acceptance Criteria for Emergency Core Cooling Systems.
2. Ditmore, D. C., Elkins, R. D., "Densification Considerations in BWR Fuel Design and Performance," NEDM-10735, December, 1972.
3. "Fuel Densification Effects on General Electric Boiling Water Reactor Fuel," Supplements 6, 7, and 8, NEDM-10735, August, 1973.
4. NEDO 20181, "GEGAP III A Model for the Prediction of Pellet Cladding Thermal Conductance in BWR Fuel Rods", November, 1973.

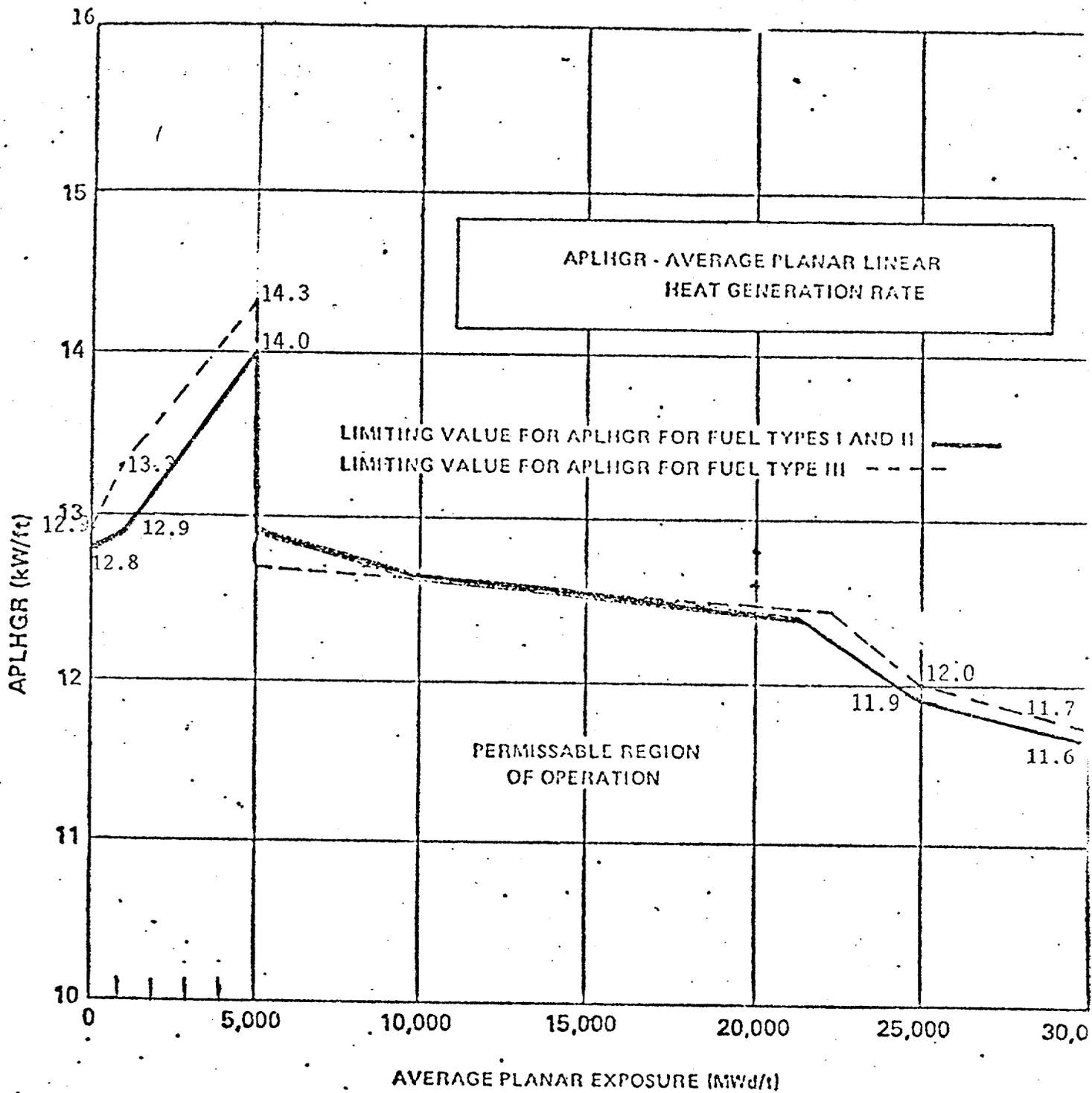


FIGURE 3.11-1 LIMITING VALUE FOR APLHGR

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 10 TO LICENSE NO. DPR-57

(CHANGE NO. 10 TO THE TECHNICAL SPECIFICATIONS)

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

EDWIN I. HATCH NUCLEAR PLANT UNIT 1

DOCKET NO. 50-321

Introduction

By letter dated January 28, 1975, Georgia Power Company proposed a change in the Technical Specifications of Facility Operating License DPR-57 for Edwin I. Hatch Nuclear Plant, Unit 1. The proposed change would replace the current maximum average planar linear heat generation rate (MAPLHGR) curves in the Technical Specifications, over the range from 0 to 5000 MW/T, with revised curves of higher value. The revised curves were computed using the GEGAP III model for pellet-clad gap thermal conductance.

Discussion

General Electric has submitted a report NEDO-20181, "GEGAP III - A Model for the Prediction of Pellet-Clad Thermal Conductance in BWR Fuel Rods," November 1973 with related proprietary information provided in NEDO-20181 Supplement 1 (Proprietary), November 1973. GEGAP III was evaluated by the staff in their report entitled "Supplement 1 to the Technical Report on Densification of General Electric Reactor Fuels," December 14, 1973, and was determined to be suitably conservative for the evaluation of densification effects in BWR fuel.

GEGAP III is a theoretical model which provides an exposure dependent pellet-clad gap thermal conductance by incorporating time-dependent fuel densification, time-dependent gap closure, and gap closure effects due to cladding creepdown. Validity of GEGAP III has been verified by comparison with recent in-reactor experimental results. The accuracy of the evaluation of fuel performance, post-LOCA, was improved with the use of GEGAP III, since previous gap conductance models did not have the capability for calculating the fuel cladding gap conductance as a function of fuel lifetime.



Evaluation

Georgia Power Company has recalculated the fuel temperature response to the design basis loss-of-coolant accident for Edwin I. Hatch Nuclear Plant, Unit 1 using GEGAP III and has determined new MAPLHGR curves for the fuel types presently in use up to an exposure of 5000 MWD/T. Use of the GEGAP III model yields an increase in the calculated pellet-clad gap conductance which in turn causes a decrease in the calculated stored energy in the fuel rods. A reduction in calculated stored energy reduces the calculated peak clad temperature following a postulated LOCA or, conversely, allows a compensating increase in MAPLHGR for a constant calculated peak clad temperature.

The proposed MAPLHGR curves would allow steady state power operation at higher average planar linear heat generation rates. The proposed MAPLHGR curves were verified by Georgia Power Company to maintain a calculated peak clad temperature well within the limitation imposed by the Interim Acceptance Criteria following a postulated design basis LOCA.

However, on August 5, 1974, Georgia Power Company had submitted an ECCS evaluation and proposed changes to the Technical Specifications in accordance with 10 CFR Part 50, Section 50.46. This submittal was subsequently reviewed by the NRC staff and modified by further restriction in an Order for Modification of License dated December 27, 1974. The Order reflected the requirements for operation within the limits and restrictions established in 10 CFR Part 50, Section 50.46 pending completion and approval of the ECCS cooling performance reevaluation using an acceptable evaluation model (the reevaluation is presently scheduled for submission on or prior to July 9, 1975).

Upon examination, it was found that the proposed MAPLHGR curves calculated by Georgia Power Company exceed the values specified in Appendix A to the Order of December 27, 1974 over the range from 0 to 5000 MWD/T; moreover, the values currently in the Technical Specifications for the range from approximately 22,000 MWD/T and beyond are also in excess of the values specified in Appendix A to the Order. Therefore, operation over the range of burnup contemplated must be limited to the lower values specified in the December 27, 1974 Order. This correct application of the Order to the licensee's request for an amendment necessitates our modifying the proposed changes to the Technical Specifications as shown by the solid line in the attached Figure 3.11.1.

Conclusion

We have concluded, based on the considerations discussed above, that:

- (1) there is reasonable assurance that the health and safety of the public will not be endangered by operations in the proposed manner, and
- (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: APR 17 1975

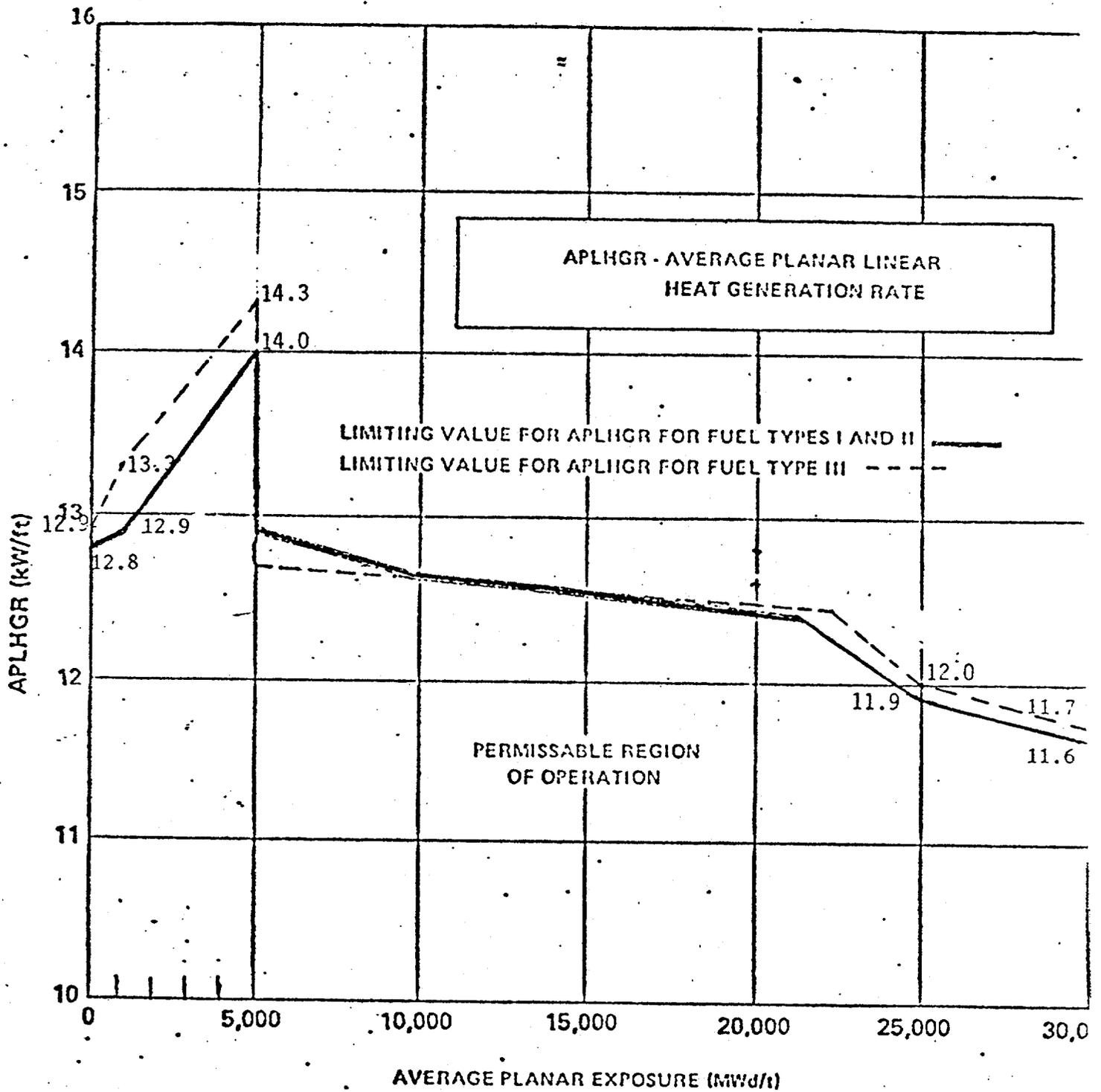


FIGURE 3.11-1 LIMITING VALUE FOR APLHGR

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 10 to Facility Operating License No. DPR-57 issued to Georgia Power Company and Oglethorpe Electric Membership Corporation which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit 1, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

The amendment permits modification to the Technical Specifications to allow an increase in the maximum average planar linear heat generation.

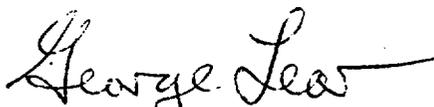
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Notice of proposed Issuance of Amendment to Facility Operating License in connection with this action was published in the FEDERAL REGISTER on March 17, 1975 (40 F.R. 12163). No request for a hearing or petition for leave to intervene was filed following notice of the proposed action.

For further details with respect to this action, see (1) the application for amendment dated January 28, 1975, (2) Amendment No. 10 to License No. DPR-57, with Change No. 10 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W. Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland APR 17 1975

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

APR 17 1975

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 PCollins

Docket No. 50-321

Georgia Power Company
 Oglethorpe Electric Membership Corporation
 ATTN: I. S. Mitchell, III
 Vice President and Secretary
 Georgia Power Company
 Atlanta, Georgia 30302

Gentlemen:

The Commission has requested the Office of the Federal Register to publish the enclosed two items entitled "Notice of Proposed Issuance of Amendment to Facility Operating License" for the Edwin I. Hatch Nuclear Plant No. 1. The proposed amendments would allow all low pressure core and containment cooling systems to be inoperable provided that the reactor is in the Cold Shutdown Condition and no work is being performed which has the potential for draining the reactor pressure vessel, and would modify the safety system settings for the APRM High-High Flux Scram Trip, the APRM Rod Block Trip and the RBM High Flux Block in accordance with your requests dated February 3 and March 3, 1975.

Sincerely,

George Lear, Chief
 Operating Reactors Branch #3
 Division of Reactor Licensing

Enclosures:
 Federal Register Notice (2)

cc: See next page

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Clearance
4-24-75

OFFICE →	ORB#3	ORB#3	ORB#3			
SURNAME →	SATeets rdg	DNbridges	GLear Gk			
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PRELIMINARY DETERMINATION - NOTICING OF
PROPOSED LICENSING AMENDMENT

Licensee: Georgia Power Company
Hatch 1
Docket No. 50-321

Request for: Authorization to allow all low pressure core and containment cooling systems to be inoperable provided that the reactor is in the Cold Shutdown Condition and no work is being performed which has the potential for draining the reactor pressure vessel.

Request Date: February 3, 1975

Proposed Action: (x) Pre-notice Recommended
() Post-notice Recommended
() Determination delayed pending completion of Safety Evaluation

Basis for Decision: An authorization to allow all the low pressure core cooling systems to be inoperable removes a backup system for cooling irradiated fuel during the shutdown condition. This proposed mode of operation tends to increase the probability of an accident significantly. Thus, this licensing action clearly meets the criteria for a pre-notice as illustrated in enclosure 3a of RPOP 601.

CONCURRENCES

DATE

1. *Walter H. Bridges* *11/27/75*
D. N. Bridges
2. *George Lear* *3/27/75*
George Lear
3. *Kent R. Goller* *3/27/75*
K. R. Goller
4. *Stephen H. Lewis* *4/9/75*
OELD

*Language suggestions
not put in notice*

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

NOTICE OF PROPOSED ISSUANCE OF AMENDMENT

TO FACILITY OPERATING LICENSE

The Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-57 issued to Georgia Power Company and Oglethorpe Electric Membership Corporation (the licensee), for operation of the Edwin I. Hatch Nuclear Plant Unit 1, located in Appling County, Georgia.

The amendment would revise ~~the provisions in the~~ Technical Specifications ^{to allow} ~~relating to~~ all low pressure core and containment cooling systems to be inoperable provided that the reactor is in the cold shutdown condition and no work is being performed which has the potential for draining the reactor pressure vessel. Incorporation of this change will permit certain major maintenance to be performed such as draining the pressure suppression pool should it become necessary, in accordance with the licensee's application for amendment dated February 3, 1975.

Prior to issuance of the proposed license amendment, the Commission will have made the findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

By _____ the licensee may file a request for a hearing and any person whose interest may be affected by this proceeding may file a request for a hearing in the form of a petition for leave to intervene with respect to the issuance of the amendment to the subject facility operating license. Petitions for leave to intervene must be filed under oath or

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

NOTICE OF PROPOSED ISSUANCE OF AMENDMENT

TO FACILITY OPERATING LICENSE

The Nuclear Regulatory Commission (the Commission is considering issuance of an amendment to Facility Operating License No. DPR-57 issued to Georgia Power Company and Oglethorpe Electric Membership Corporation (the licensee), for operation of the Edwin I. Hatch Nuclear Plant Unit 1, located in Appling County, Georgia.

The amendment would revise the Technical Specifications to allow all low pressure core and containment cooling systems to be inoperable provided that the reactor is in the cold shutdown condition and no work is being performed which has the potential for draining reactor vessel. Incorporation of this change will permit certain major maintenance to be performed such as draining the pressure suppression pool should it become necessary, in accordance with the licensee's application for amendment dated February 3, 1975.

Prior to issuance of the proposed license amendment, the Commission will have made the findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

By 5/19/75 the licensee may file a request for a hearing and any person whose interest may be affected by this proceeding may file a request for a hearing in the form of a petition for leave to intervene with respect to the issuance of the amendment to the subject facility operating license. Petitions for leave to intervene must be filed under oath or

affirmation in accordance with the provisions of Section 2.714 of 10 CFR Part 2 of the Commission's regulations. A petition for leave to intervene must set forth the interest of the petitioner in the proceeding, how that interest may be affected by the results of the proceeding, and the petitioner's contentions with respect to the proposed licensing action. Such petitions must be filed in accordance with the provisions of this FEDERAL REGISTER notice and Section 2.714, and must be filed with the Secretary of the Commission, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, and to G. F. Trowbridge, Esquire, Shaw, Pittman, Potts, Trowbridge & Madden, Barr Building, 910 17th Street, N. W., Washington, D. C. 20006, the attorney for the licensee.

A petition for leave to intervene must be accompanied by a supporting affidavit which identifies the specific aspect or aspects of the proceeding as to which intervention is desired and specifies with particularity the facts on which the petitioner relies as to both his interest and his contention with regard to each aspect on which intervention is requested. Petitions stating contentions relating only to matters outside the Commission's jurisdiction will be denied.

All petitions will be acted upon by the Commission or licensing board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel. Timely petitions will be considered to determine whether a hearing should be noticed or another appropriate order issued regarding the disposition of the petitions.

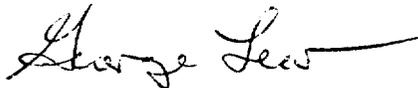
In the event that a hearing is held and a person is permitted to intervene, he becomes a party to the proceeding and has a right to participate

fully in the conduct of the hearing. For example, he may present evidence and examine and cross-examine witnesses.

For further details with respect to this action, see the application for amendment dated February 3, 1975, which is available for public inspection at the Commission's Public Document Room 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. The license amendment and the Safety Evaluation, when issued, may be inspected at the above locations and a copy may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this APR 11 1975

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

PRELIMINARY DETERMINATION - NOTICING OF
PROPOSED LICENSING AMENDMENT

Licensee: Georgia Power Company
Hatch 1
Docket No. 50-321

Request for: Modified safety system settings for (1) the Average Power Range Monitor (APRM) High-High Flux Scram Trip, (2) for the APRM Rod Block Trip, and (3) for the Rod Block Monitor (RBM) High Flux Block.

Request Date: March 3, 1975

Proposed Action: (X) Pre-notice Recommended
() Post-notice Recommended
() Determination delayed pending completion of Safety Evaluation

Basis for Decision: Georgia Power Company has requested modified values for limiting safety system (Reactor Power) settings. These changes in the safety system settings have a significant effect on both the probability and consequences of an accident. Thus, this licensing action meets the criteria for a pre-noticed license amendment as described in enclosure 3a to RPOP 601.

CONCURRENCES

DATE

1. Walter A. Carlson March 25 1975
for D. N. Bridges
2. George Lear 3/27/75
George Lear
3. Karl R. Goller 3/28/75
K. R. Goller
4. Stephen H. Lewis 4/9/75
OELD

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-321

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION

NOTICE OF PROPOSED ISSUANCE OF AMENDMENT

TO FACILITY OPERATING LICENSE

The Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-57 issued to Georgia Power Company and Oglethorpe Electric Membership Corporation (the licensee), for operation of the Edwin I Hatch Nuclear Plant Unit 1, located in Appling County, Georgia.

The amendment would revise the provisions in the Technical Specifications relating to the safety system settings for the Average Power Range Monitor (APRM) High-High Flux Scram Trip, for the APRM rod block trip, and for the Rod Block Monitor (RBM) High Flux rod block. These safety system settings are established as a function of reactor coolant flow and the proposed values represent updated correlations, in accordance with the licensee's application for amendment dated March 3, 1975.

Prior to issuance of the proposed license amendment, the Commission will have made the findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

By 5/19/75 the licensee may file a request for a hearing and any person whose interest may be affected by this proceeding may file a request for a hearing in the form of a petition for leave to intervene with respect to the issuance of the amendment to the subject facility operating license. Petitions for leave to intervene must be filed under oath or affirmation in accordance with the provisions of Section 2.714 of 10 CFR Part 2 of the Commission's regulations. A petition for leave to intervene

must set forth the interest of the petitioner in the proceeding, how that interest may be affected by the results of the proceeding, and the petitioner's contentions with respect to the proposed licensing action. Such petitions must be filed in accordance with the provisions of this FEDERAL REGISTER notice and Section 2.714, and must be filed with the Secretary of the Commission, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Docketing and Service Section, by the above date. A copy of the petition and/or request for a hearing should be sent to the Executive Legal Director, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, and to G. F. Trowbridge, Esquire, Shaw, Pittman, Potts, Trowbridge & Madden, Barr Building, 910 17th Street, N. W., Washington, D. C. 20006, the attorney for the licensee.

A petition for leave to intervene must be accompanied by a supporting affidavit which identifies the specific aspect or aspects of the proceeding as to which intervention is desired and specifies with particularity the facts on which the petitioner relies as to both his interest and his contention with regard to each aspect on which intervention is requested. Petitions stating contentions relating only to matters outside the Commission's jurisdiction will be denied.

All petitions will be acted upon by the Commission or licensing board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel. Timely petitions will be considered to determine whether a hearing should be noticed or another appropriate order issued regarding the disposition of the petitions.

In the event that a hearing is held and a person is permitted to intervene, he becomes a party to the proceeding and has a right to participate

fully in the conduct of the hearing. For example, he may present evidence and examine and cross-examine witnesses.

For further details with respect to this action, see the application for amendment dated March 3, 1975, which is available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. The license amendment and the Safety Evaluation, when issued, may be inspected at the above locations and a copy may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 11th day of April, 1975

FOR THE NUCLEAR REGULATORY COMMISSION


George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing