

Mr. Neil S. Carns
 Senior Vice President
 and Chief Nuclear Officer
 Northeast Nuclear Energy Company
 c/o Ms. Patricia A. Loftus
 Director - Nuclear Licensing Services
 P.O. Box 128
 Waterford, CT 06385

October 27, 1997

SUBJECT: ISSUANCE OF AMENDMENT RELATING TO THE CONTAINMENT PRESSURE
 TECHNICAL SPECIFICATIONS - MILLSTONE NUCLEAR POWER STATION,
 UNIT NO. 2 (TAC NO. M99503)

Dear Mr. Carns:

The Commission has issued the enclosed Amendment No. 209 to Facility Operating License No. DPR-65 for the Millstone Nuclear Power Station, Unit No. 2, in response to your application dated September 2, 1997.

The amendment changes the Technical Specifications (TSs) by modifying the maximum allowed primary containment internal pressure during normal operation from 2.1 psig to 1.0 psig. The TS Bases are also updated to reflect the new maximum allowed primary containment internal pressure during normal operation.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:
 Daniel G. McDonald Jr., Sr. Project Manager
 Special Projects Office - Licensing
 Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures: 1. Amendment No. 209 to DPR-65
 2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 27, 1997

Mr. Neil S. Carns
Senior Vice President
and Chief Nuclear Officer
Northeast Nuclear Energy Company
c/o Ms. Patricia A. Loftus
Director - Nuclear Licensing Services
P.O. Box 128
Waterford, CT 06385

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Sincerely,

A handwritten signature in black ink, appearing to read "Daniel G. McDonald Jr.", with a stylized flourish at the end.

Daniel G. McDonald Jr., Sr. Project Manager
Special Projects Office - Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-336

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2. Safety Evaluation

cc w/encls: See next page

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Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY
THE CONNECTICUT LIGHT AND POWER COMPANY
AND WESTERN MASSACHUSETTS ELECTRIC COMPANY

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 209
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Northeast Nuclear Energy Company, et al. (the licensees) dated September 2, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

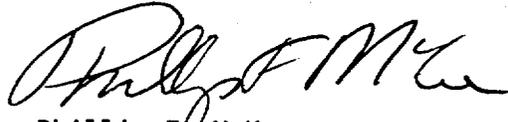
2. Accordingly, Facility Operating License No. DPR-65 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 209, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance to be implemented within 30 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee
Deputy Director for Licensing
Special Projects Office
Office of Nuclear Reactor Regulation

Attachment: Changes to Technical
Specifications

Date of Issuance: October 27, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 209

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Operating License and Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 6-8

B 3/4 6-2

Insert

3/4 6-8

B 3/4 6-2

CONTAINMENT SYSTEMS

INTERNAL PRESSURE

LIMITING CONDITION FOR OPERATION

3.6.1.4 Primary containment internal pressure shall be maintained between -12 inches Water Gauge and +1.0 PSIG. |

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the containment internal pressure in excess of or below the limits above, restore the internal pressure to within the limits within 1 hour or be in HOT STANDBY within the next 4 hours; go to COLD SHUTDOWN within the next 36 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.4 The primary containment internal pressure shall be determined to within the limits at least once per 12 hours.

CONTAINMENT SYSTEMS

BASES

3/4.6.1.4 INTERNAL PRESSURE

The limitations on containment internal pressure ensure that the containment peak pressure does not exceed the design pressure of 54 psig during MSLB or LOCA conditions.

The maximum peak pressure is obtained from a MSLB event. The limit of 1.0 psig for initial positive containment pressure will limit the total pressure to less than the design pressure and is consistent with the accident analyses.

3/4.6.1.5 AIR TEMPERATURE

The limitation on containment air temperature ensures that the containment air temperature does not exceed the worst case combined LOCA/MSLB air temperature profile and the liner temperature of 289°F. The containment air and liner temperature limits are consistent with the accident analyses.

3/4.6.1.6 CONTAINMENT STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the containment vessel will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to ensure that the vessel will withstand the design pressure of 54 psig in the event of a LOCA or MSLB. The measurement of containment tendon lift off force, the visual and metallurgical examination of tendons, anchorages and liner and the Type A leakage tests are sufficient to demonstrate this capability.

The surveillance requirements for demonstrating the containment's structural integrity are in compliance with the recommendations of Regulatory Guide 1.35 "Inservice Surveillance of UngROUTed Tendons in Prestressed Concrete Containment Structures."



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 209

TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By letter dated September 2, 1997, the Northeast Nuclear Energy Company, et al. (the licensee) submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 2, Technical Specifications (TSs). The proposed changes would modify the TSs for the maximum allowed primary containment internal pressure during normal operation.

Specifically, the proposed changes would modify TS 3.6.1.4, "Containment System Internal Pressure," from 2.1 pounds per square inch gauge (psig) to 1.0 psig. The TS Bases, Section 3/4.6.1.4, would also be modified to reflect the new maximum allowed internal primary containment pressure of 1.0 psig.

2.0 BACKGROUND

The licensee is in the process of reanalyzing the main steamline break accident for Millstone, Unit No. 2, and has determined that it is necessary to require a lower initial primary containment internal pressure as an initial condition in performing the analysis. This initial maximum primary containment internal pressure of 1.0 psig, combined with a proposed plant modification to add cavitating ventures in the auxiliary feedwater (AFW) discharge lines to each steam generator, should result in the reanalysis indicating that the peak primary containment internal pressure following a main steamline break will not exceed the design pressure of 54 psig.

The reanalysis of the main steamline break is required to be completed and approved prior to the restart of Millstone, Unit No. 2.

3.0 EVALUATION

The previous main steamline break analysis assumed that the AFW flow, at the initiation of an event, was the result of a single failure. However, the use of the AFW to supply the steam generators is normal, up to approximately 3 percent power, in accordance with the current operating procedures. The omission of not including the AFW flows was identified in Licensee Event Report (LER) 97-006-00, dated February 11, 1997. The licensee committed in the LER to implement corrective actions prior to restart from the current extended outage. The corrective actions necessary include the plant modification and reanalysis, as previously discussed.

The reanalysis will determine the effect of various AFW flow rates to a steam generator, assuming a main steamline break inside containment, and the resulting primary containment peak pressure. Using the existing TS maximum primary containment initial pressure of 2.1 psig could provide results that could exceed the 54 psig design limit. As previously noted, reducing the maximum primary containment initial pressure requirement in the TSs to 1.0 psig will result in a reduction of the peak primary containment pressure resulting from a main steamline break inside of containment.

Therefore, to ensure that the lower initial primary containment maximum pressure is maintained, the NRC staff has determined that the proposed change to TS 3.6.1.4 from a maximum initial pressure of 2.1 psig to 1.0 psig is acceptable. The NRC staff has also determined that the proposed update of the TS Bases, Section 3/4.6.1.4, to reflect the change in the primary containment maximum initial pressure is also acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (62 FR 50007 dated September 24, 1997). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. McDonald

Date: October 27, 1997